UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 22, 2011

Mr. Michael J. Pacilio
President and Chief Nuclear Officer
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: THREE MILE ISLAND NUCLEAR STATION, UNIT 1 - ISSUANCE OF AMENDMENT RE: RELOCATION OF EQUIPMENT LOAD LIST FROM TECHNICAL SPECIFICATIONS TO UPDATED FINAL SAFETY ANALYSIS REPORT (TAC NO. ME4732)

Dear Mr. Pacilio:

The Commission has issued the enclosed Amendment No. 276 to Facility Operating License No. DPR-50 for the Three Mile Island Nuclear Station, Unit 1 (TMI-1), in response to your application dated September 22, 2010 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML102650392), as supplemented by letter dated April 7, 2011 (ADAMS Accession No. ML110980268).

The amendment relocates the list of pumps, fans, and valves in Technical Specification (TS) 4.5.1.1b, Sequence and Power Transfer Test, to the TMI-1 Updated Final Safety Analysis Report. In place of the TS equipment listing there will be a more general reference to the permanently-connected and automatically-connected emergency loads which are tested through the load sequencer. In addition, TS 4.5.1.2b, TS 4.5.2.2a, and TS 4.5.2.2b refer to this test and are revised to reflect the proposed change to TS 4.5.1.1b.

A copy of the related safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

Peter Bamford

Peter J. Bamford, Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-289

Enclosures: 1. Amendment No. 276 to DPR-50
2. Safety Evaluation

cc: Distribution via Listserv

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-289

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 276
License No. DPR-50

1. The Nuclear Regulatory Commission (the Commission or NRC) has found that:

   A. The application for amendment by Exelon Generation Company, LLC (the licensee), dated September 22, 2010, as supplemented by letter dated April 7, 2011, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.c.(2) of Renewed Facility Operating License No. DPR-50 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 276, are hereby incorporated in the license. The Exelon Generation Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately and shall be implemented within 30 days. Implementation of the amendment shall include updating the UFSAR in accordance with 10 CFR 50.71(e). This update shall include, but not be limited to, a complete relocation of the equipment load listing from TS 4.5.1.1b to the UFSAR.

FOR THE NUCLEAR REGULATORY COMMISSION

for Harold K. Chernoff, Chief
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the License and
Technical Specifications

Date of Issuance: August 22, 2011

ATTACHMENT TO LICENSE AMENDMENT NO. 276

FACILITY OPERATING LICENSE NO. DPR-50

DOCKET NO. 50-289

Replace the following page of the Renewed Facility Operating License with the revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove	Insert
Page 4	Page 4

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
4-39	4-39
4-41	4-41

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 276 are hereby incorporated in the license. The Exelon Generation Company shall operate the facility in accordance with the Technical Specifications.

(3) Physical Protection

Exelon Generation Company shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822), and the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans[1], submitted by letter dated May 17, 2006, is entitled: "Three Mile Island Nuclear Station Security Plan, Training and Qualification Plan, and Safeguards Contingency Plan, Revision 3." The set contains Safeguards Information protected under 10 CFR 73.21.

Exelon Generation Company shall fully implement and maintain in effect all provisions of the Commission-approved cyber security plan (CSP), including changes made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The Exelon Generation Company CSP was approved by License Amendment No. 275.

(4) Fire Protection

Exelon Generation Company shall implement and maintain in effect all provisions of the Fire Protection Program as described in the Updated FSAR for TMI-1.

Changes may be made to the Fire Protection Program without prior approval by the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire. Temporary changes to specific fire protection features which may be necessary to accomplish maintenance or modifications are acceptable provided that interim compensate measures are implemented.

(5) The licensee shall implement a secondary water chemistry monitoring program to inhibit steam generator tube degradation. This program shall include:

a. Identification of a sampling schedule for the critical parameters and control points for these parameters;
b. Identification of the procedures used to measure the values of the critical parameters;
c. Identification of process sampling points;
d. Procedure for the recording and management of data;

[1] The Training and Qualification Plan and Safeguards Contingency Plan are Appendices to the Security Plan.

4.5 EMERGENCY LOADING SEQUENCE AND POWER TRANSFER, EMERGENCY CORE COOLING SYSTEM & REACTOR BUILDING COOLING SYSTEM PERIODIC TESTING

4.5.1 Emergency Loading Sequence

Applicability: Applies to periodic testing requirements for safety actuation systems.

Objective: To verify that the emergency loading sequence and automatic power transfer is operable.

Specifications:

4.5.1.1 Sequence and Power Transfer Test

a. At the frequency specified in the Surveillance Frequency Control Program, a test shall be conducted to demonstrate that the emergency loading sequence and power transfer is operable.

b. The test will be considered satisfactory if the permanently connected loads and auto-connected emergency loads have been successfully energized on preferred power using the load sequencer and transferred to emergency power.

c. Following successful transfer to the emergency diesel, the diesel generator breaker will be opened to simulate trip of the generator then re-closed to verify block load on the reclosure.

4.5.1.2 Sequence Test

a. At the frequency specified in the Surveillance Frequency Control Program, a test shall be conducted to demonstrate that the emergency loading sequence is operable, this test shall be performed on either preferred power or emergency power.

b. The test will be considered satisfactory if the auto-connected emergency loads have been successfully energized using the load sequencer.

4.5.2 EMERGENCY CORE COOLING SYSTEM

Applicability: Applies to periodic testing requirement for emergency core cooling systems.

Objective: To verify that the emergency core cooling systems are operable.

Specification

4.5.2.1 High Pressure Injection

a. At the frequency specified in the Surveillance Frequency Control Program and following maintenance or modification that affects system flow characteristics, system pumps and system high point vents shall be vented, and a system test shall be conducted to demonstrate that the system is operable.

b. The test will be considered satisfactory if the valves (MU-V-14A/B & 16A/B/C/D) have completed their travel and the make-up pumps are running as evidenced by system flow. Minimum acceptable injection flow must be greater than or equal to 431 gpm per HPI pump when pump discharge pressure is 600 psig or greater (the pressure between the pump and flow limiting device) and when the RCS pressure is equal to or less than 600 psig.

c. Testing which requires HPI flow thru MU-V16A/B/C/D shall be conducted only under either of the following conditions:

1) Indicated RCS temperature shall be greater than 329°F.
2) Head of the Reactor Vessel shall be removed.

4.5.2.2 Low Pressure Injection

a. At the frequency specified in the Surveillance Frequency Control Program and following maintenance or modification that affects system flow characteristics, system pumps and high point vents shall be vented, and a system test shall be conducted to demonstrate that the system is operable. The auxiliaries required for low pressure injection are all included in the emergency loading sequence test specified in 4.5.1.

b. The test will be considered satisfactory if the decay heat pumps have been successfully started and the decay heat injection valves and the decay heat supply valves have completed their travel as evidenced by the control board component operating lights. Flow shall be verified to be equal to or greater than the flow assumed in the Safety Analysis for the single corresponding RCS pressure used in the test.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 276 TO FACILITY OPERATING LICENSE NO. DPR-50

RELOCATION OF EQUIPMENT LOAD LIST FROM TECHNICAL SPECIFICATIONS TO

UPDATED FINAL SAFETY ANALYSIS REPORT

EXELON GENERATION COMPANY, LLC

THREE MILE ISLAND NUCLEAR STATION, UNIT 1

DOCKET NO. 50-289

1.0 INTRODUCTION

By application dated September 22, 2010 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML102650392), as supplemented by letter dated April 7, 2011 (ADAMS Accession No. ML110980268), Exelon Generation Company (Exelon, or the licensee) requested changes to the Technical Specifications (TSs) for Three Mile Island Nuclear Station, Unit 1 (TMI-1). The supplement provided additional information that clarified the application and did not expand the scope of the application as originally noticed in the *Federal Register* on November 30, 2010 (75 FR 74095). Due to changes in the structure of the proposed TS contained in the April 7, 2011, supplement, which resulted in corrections to the original proposed no significant hazards consideration evaluation, the application was re-noticed in the *Federal Register* on May 3, 2011 (76 FR 24928). The revised notice did not change the U.S. Nuclear Regulatory Commission (NRC or Commission) staff's original proposed no significant hazards consideration determination.

The amendment relocates the list of pumps, fans, and valves in TS 4.5.1.1b, Sequence and Power Transfer Test, to the TMI-1 Updated Final Safety Analysis Report (UFSAR). In place of the TS equipment listing there will be a more general reference to the permanently-connected and automatically-connected emergency loads, as applicable, which are tested in accordance with TS 4.5.1. In addition, TS 4.5.1.2b, TS 4.5.2.2a, and TS 4.5.2.2b refer to portions of TS 4.5.1.1b and are therefore revised to reflect the change to TS 4.5.1.1b.

2.0 REGULATORY EVALUATION

The NRC's regulatory requirements related to the content of the TSs are set forth in Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.36, "Technical specifications." This regulation requires, in part, that the TSs include items in the following five specific categories: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation (LCOs); (3) surveillance requirements; (4) design features; and (5) administrative controls. The regulation does not specify the particular requirements to be included in a plant's TSs.

Paragraph 50.36(c)(3) of 10 CFR states that: "Surveillance requirements are requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the limiting conditions for operation will be met."

Requirements for changes to the UFSAR can be found at 10 CFR 50.71(e). Nuclear Energy Institute (NEI) guidance document NEI 98-03, Revision 1, "Guidelines for Updating Final Safety Analysis Reports" (ADAMS Accession No. ML003779028) provides guidance for changes to the UFSAR. The NRC endorsed NEI 98-03, Revision 1, in Regulatory Guide 1.181, "Content of the Updated Final Safety Analysis Report in Accordance with 10 CFR 50.71(e)," dated September 1999 (ADAMS Accession No. ML992930009), stating that it provides methods that are acceptable to the NRC staff for complying with the provisions of 10 CFR 50.71(e). Changes to the UFSAR are controlled in accordance with 10 CFR 50.59, "Changes, tests, and experiments."

3.0 TECHNICAL EVALUATION

3.1 Proposed TS Changes

The current and proposed TS are summarized as follows:

Current TS 4.5.1.1b:

"The test will be considered satisfactory if the following pumps and fans have been successfully started and the following valves have completed their travel on preferred power and transferred to the emergency power.

- M. U. [Makeup] Pump
- D. H. [Decay Heat] Pump and D. H. Injection Valves and D. H. Supply Valves
- R. B. [Reactor Building] Cooling Pump
- R. B. Ventilators
- D. H. Closed Cycle Cooling Pump
- N. S. [Nuclear Services] Closed Cycle Cooling Pump
- D. H. River Cooling Pump
- N. S. River Cooling Pump
- D. H. and N. S. Pump Area Cooling Fan
- Screen House Area Cooling Fan
- Spray Pump. (Initiated in coincidence with a 2 out of 3 R. B. 30 psig Pressure Test Signal.)
- Motor Driven Emergency Feedwater Pump"

Proposed TS 4.5.1.1b:

"The test will be considered satisfactory if the permanently connected loads and auto-connected emergency loads have been successfully energized on preferred power using the load sequencer and transferred to emergency power."

Current TS 4.5.1.2b:

"The test will be considered satisfactory if the pumps and fans listed in 4.5.1.1b have been successfully started and the valves listed in 4.5.1.1b have completed their travel."

Proposed TS 4.5.1.2b:

"The test will be considered satisfactory if the auto-connected emergency loads have been successfully energized using the load sequencer."

Current TS 4.5.2.2a:

"At the frequency specified in the Surveillance Frequency Control program and following maintenance or modification that affects system flow characteristics, system pumps and high point vents shall be vented, and a system test shall be conducted to demonstrate that the system is operable. The auxiliaries required for low pressure injection are all included in the emergency loading sequence specified in 4.5.1."

Proposed TS 4.5.2.2a:

"At the frequency specified in the Surveillance Frequency Control program and following maintenance or modification that affects system flow characteristics, system pumps and high point vents shall be vented, and a system test shall be conducted to demonstrate that the system is operable. The auxiliaries required for low pressure injection are all included in the emergency loading sequence *test* (italics added) specified in 4.5.1."

Current TS 4.5.2.2b:

"The test will be considered satisfactory if the decay heat pumps listed in 4.5.1.1b have been successfully started and the decay heat injection valves and the decay heat supply valves have completed their travel as evidenced by the control board component operating lights. Flow shall be verified to be equal to or greater than the flow assumed in the Safety Analysis for the single corresponding RCS pressure used in the test."

Proposed TS 4.5.2.2b:

"The test will be considered satisfactory if the decay heat pumps have been successfully started and the decay heat injection valves and the decay heat supply valves have completed their travel as evidenced by the control board component operating lights. Flow shall be verified to be equal to or greater than the flow assumed in the Safety Analysis for the single corresponding RCS pressure used in the test."

3.2 Evaluation of Proposed TS Changes to TS 4.5.1.1, "Sequence and Power Transfer Test" and TS 4.5.1.2, "Sequence Test."

3.2.1 TS 4.5.1.1b and 4.5.1.2b

TMI-1 TS 4.5.1, "Emergency Loading Sequence" applies to the periodic testing of safety actuation systems. The objective of the TS 4.5.1 test is to verify that the emergency loading sequence and automatic power transfer function is operable. TS 4.5.1.1b contains a list of pumps, valves and fans that are verified to have started (pumps and fans) or changed position

(valves) and transferred to emergency power to determine satisfactory performance of the test. The proposed changes to TS 4.5.1.1b would relocate the equipment load list to the UFSAR and modify the wording to include reference to the permanently-connected and auto-connected loads. The design features tested during the Sequence and Power Transfer Test and Sequence Test are described in the TMI-1 UFSAR, Sections 1, 7, and 8. The proposed changes do not change the requirement to perform a Sequence and Power Transfer Test and Sequence Test of Engineered Safeguards equipment, nor do they change the performance requirements of the surveillance. Also, the individual component surveillance tests located elsewhere in the TS for the affected components are not being revised.

The proposed relocation of the equipment load list from TS 4.5.1.1b is a change to TS surveillance requirements, and hence 10 CFR 50.36(c)(3) is applicable. This paragraph of 10 CFR does not provide specific guidance on the level of detail to which a surveillance requirement should be written. TMI-1 TS 4.5.1.1b currently provides a list of the specific loads to be tested during the Sequence and Power Transfer Test. This level of TS detail is not required to ensure operability as long as the TS continue to provide the necessary clarity as to which loads must be tested. The requirements of the applicable LCOs and their corresponding surveillance requirements, as well as the definition of OPERABLE (TMI-1 TS 1.3), are adequate to ensure that the systems are maintained in an operable state. Moving the load listing from the TS to the UFSAR does not change the requirement to test the performance of the applicable structures, systems or components and therefore proper verification of acceptable performance is maintained. Thus, the NRC staff concludes that the list of tested equipment in TS is not necessary to ensure that the systems can perform their intended safety function, and is not required to be in the TS to provide adequate protection of public health and safety.

TS 4.5.1.2b is changed to reflect the relocation of the equipment load list from TS 4.5.1.1b and to identify the satisfactory completion of the sequence test by verifying that the auto-connected loads have been energized via the emergency loading sequencer. This is a logical change based on the removal of the TS 4.5.1.1b listing from the TS and therefore, it is acceptable to the NRC staff based on the same rationale as the TS 4.5.1.1b changes described above.

3.2.2 TS 4.5.2.2a and 4.5.2.2b

TS 4.5.2, "Emergency Core Cooling System," applies to the periodic testing requirement for emergency core cooling and is performed to verify that the core cooling systems are operable. TS 4.5.2.2a is changed by adding the word "test" to the following sentence: "The auxiliaries required for low pressure injection are all included in the emergency loading sequence test specified in 4.5.1." This change provides clarification to the sentence. Since the equipment load listing is no longer contained directly in TS 4.5.1, referring to the more general test described in TS 4.5.1 ensures the TS remain accurate, and also ensures that the intent is not changed.

TS 4.5.2.2b is changed by removing the reference to the equipment load list relocated from TS 4.5.1.1b. This change is editorial in nature and is necessary because of the proposed relocation of the list from TS 4.5.1.1b to the UFSAR. Since the listing will no longer be contained in TS 4.5.1.1b, the removal of that phrase is appropriate. Also, there are no other decay heat pumps other than those in the relocated list and thus referring to them by their title needs no further descriptive phrases. The actual pumps referred to in the modified TS 4.5.2.2b do not change and the proposed change is acceptable.

3.3 TS Bases

In the application dated September 22, 2010, and in the supplement dated April 7, 2011, the licensee included modified TS Bases changes. Some of the TS Bases changes presented did not relate to the proposed TS changes. For example, TS Bases changes specifying that the testing may include, " ... any series of sequential, overlapping, or total steps so that the entire connection and loading sequence is verified," were included in the application. The application does not identify how this verbiage supports the proposed relocation. TS Bases changes are made under the control of the licensee's TS Bases control program as specified in TS 6.18. Hence, the NRC staff makes no judgment in this application review as to the adequacy or acceptability of the TS Bases changes submitted.

3.4 Technical Evaluation Conclusion

The NRC staff has reviewed the licensee's submittal and based on the review, the NRC staff concludes that having the list of sequenced loads in the TS is not required to meet the criteria of 10 CFR 50.36(c)(3). The equipment proposed for relocation does not need to be included in the surveillance requirement to ensure the effectiveness of the TSs. Accordingly, this list of equipment may be relocated to the UFSAR, a licensee-controlled document, where future changes would be adequately governed by 10 CFR 50.59. The NRC staff therefore concludes that the proposed changes to the TMI-1 TS 4.5.1.1b and TS 4.5.1.2b are acceptable. The NRC staff also notes that the proposed changes are consistent with NUREG 1430, "Standard Technical Specifications Babcock and Wilcox Plants."

The licensee's proposed change to TS 4.5.2.2a provides sentence clarification, is editorial, and therefore, is acceptable. The change to TS 4.5.2.2b is also editorial in that it reflects the proposed change to TS 4.5.1.1b, and is therefore, acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20, as well as a surveillance requirement. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (75 FR 74095 and 76 FR 24928). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: G. Waig
P. Bamford

Date: August 22, 2011

August 22, 2011

Mr. Michael J. Pacilio
President and Chief Nuclear Officer
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: THREE MILE ISLAND NUCLEAR STATION, UNIT 1 - ISSUANCE OF AMENDMENT RE: RELOCATION OF EQUIPMENT LOAD LIST FROM TECHNICAL SPECIFICATIONS TO UPDATED FINAL SAFETY ANALYSIS REPORT (TAC NO. ME4732)

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Sincerely,
/ra/
Peter J. Bamford, Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-289

Enclosures: 1. Amendment No. 276 to DPR-50
2. Safety Evaluation
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ADAMS Accession No.: ML112150486 * via memo

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NAME	PBamford	ABaxter	REIliott *	MMarsh	HChernoff (REnnis for)
DATE	8/3/2011	8/10/2011	07/19/2011	8/18/2011	8/22/2011

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