Enclosure 3

Generic Communications - Master Table

GENERIC COMMUNICATIONS: MASTER TABLE

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 71-002	PWR Reactor Trip Circuit Breakers	NA	Addressed to specific plant(s).
B 71-003	Catastrophic Failure of Main Steam Line Relief Valve Headers	NA	Addressed to specific plant(s).
B 72-001	Failed Hangers for Emergency Core Cooling System Suction Header	NA	Addressed to specific plant(s).
B 72-002	Simultaneous Actuation of a Safety Injection Signal on Both Units of a Dual Unit Facility	NA	Addressed to specific plant(s).
B 72-003	Limitorque Valve Operator Failures	NA	Addressed to specific plant(s).
B 73-001	Faulty Overcurrent Trip Delay Device in Circuit Breakers for Engineered Safety Systems	C	TVA: letter dated April 4, 1973 NRC: IR 390/391 75-5
B 73-002	Malfunction of Containment Purge Supply Valve Switch	C	TVA: letter dated August 22, 1973 NRC: IR 390/391 75-5
B 73-003	Defective Hydraulic Snubbers and Restraints	C	TVA: letter dated February 7, 1985 NRC: IR 390/391 85-08
B 73-004	Defective Bergen-Patterson Hydraulic Shock Absorbers	C	TVA: memo dated February 7, 1985 NRC: IR 390/391 85-08
B 73-005	Manufacturing Defect in BWR Control Rods	NA	Boiling Water Reactor
B 73-006	Inadvertent Criticality in a BWR	NA	Boiling Water Reactor
B 74-001	Valve Deficiencies	C	TVA: letter dated April 15, 1974 NRC: IR 390/391 75-5
B 74-002	Truck Strike Possibility	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 74-003	Failure of Structural or Seismic Support Bolts on Class I Components	CI 06	TVA: memo dated January 22, 1985 NRC: IR 390/391 85-08 --- Approach accepted in IR 50-390/85-08 and 50-391/85-08 (March 29, 1985). Unit 2 Action: Implement per NUREG-0577 as was done for Unit 1. --- --- REVISION 06 UPDATE: Corrective action for this item consisted of a bolting reheat treatment program for both units; it has been completed.
B 74-004	Malfunction of Target Rock Safety Relief Valves	NA	Boiling Water Reactor
B 74-005	Shipment of an Improperly Shielded Source	NA	Does not apply to power reactor.
B 74-006	Defective Westinghouse Type W-2 Control Switch Component	C	TVA: letter dated October 18, 1974 NRC: IR 390/391 75-6
B 74-007	Personnel Exposure – Irradiation Facility	NA	Does not apply to power reactor.
B 74-008	Deficiency in the ITE Molded Case Circuit Breakers, Type HE-3	C	TVA: letter dated August 21, 1974 NRC: IR 390/391 75-5
B 74-009	Deficiency in GE Model 4KV Magne-Blast Circuit Breakers	C	TVA: letter dated September 20, 1974 NRC: IR 390/391 76-6
B 74-010	Failures in 4-Inch Bypass Pipe at Dresden 2	NA	Boiling Water Reactor
B 74-011	Improper Wiring of Safety Injection Logic at Zion 1 & 2	C	NRC: IR 390/391 75-6
B 74-012	Incorrect Coils in Westinghouse Type SG Relays at Trojan	C	NRC: IR 390/391 75-5
B 74-013	Improper Factory Wiring on GE Motor Control Centers at Fort Calhoun	C	TVA: letter dated December 24, 1974 NRC: IR 390/391 75-5

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 74-014	BWR Relief Valve Discharge to Suppression Pool	NA	Boiling Water Reactor
B 74-015	Misapplication of Cutler-Hammer Three Position Maintained Switch Model No. 10250T	C 06	TVA: letter dated May 5, 1975 NRC: IR 390/391 75-5 --- Unit 2 Action: Install modified A3 Cutler-Hammer 10250T switches. --- REVISION 06 UPDATE: It has been confirmed that WBN Unit 2 never had the faulty switches. --- NRC Inspection Report 391/2010-605 closed B 74-015.
B 74-016	Improper Machining of Pistons in Colt Industries (Fairbanks-Morse) Diesel-Generators	C	TVA: letter dated January 2, 1975 NRC: IR 390/391 75-3
B 75-001	Through-Wall Cracks in Core Spray Piping at Dresden-2	NA	Boiling Water Reactor
B 75-002	Defective Radionics Radiograph Exposure Devices and Source Changers	NA	Does not apply to power reactor.
B 75-003	Incorrect Lower Disc Spring and Clearance Dimension in Series 8300 and 8302 ASCO Solenoid Valves	CI	TVA: letter dated May 16, 1975 NRC: IR 390/391 75-6 --- NRC accepted in IR 50-390/75-6 and 50-391/75-6 (August 21, 1975). Unit 2 Action: Modify valves not modified at factory.
B 75-004	Cable Fire at BFNPP	CI	NRC: IR 390/391 85-08 Closed to Fire Protection CAP --- Part of Fire Protection CAP

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 75-005	Operability of Category I Hydraulic Shock and Sway Suppressors	CI	TVA: letter dated June 16, 1975 NRC: IR 390/391 75-6 --- NRC accepted in IR 50-390/75-6 and 50-391/75-6 (August 21, 1975). Unit 2 Action: Install proper suppressors.
B 75-006	Defective Westinghouse Type OT-2 Control Switches	CI 06	TVA: letter dated July 31, 1975 NRC: IR 390/85-25 and 391/85-20 --- Unit 2 Action: Inspect Westinghouse Type OT-2 control switches. [WAS "NOTE 3."] --- --- REVISION 06 UPDATE: All Unit 2 Type OT-2 switches procured or refurbished are inspected and tested.
B 75-007	Exothermic Reaction in Radwaste Shipment	NA	Does not apply to power reactor.
B 75-008	PWR Pressure Instrumentation	S 02	NRC: IR 390/391 85-08 --- Unit 2 Action: Ensure that Technical Specifications and Site Operating Instructions address importance of maintaining temperature and pressure within prescribed limits. --- --- REVISION 02 UPDATE: Developmental Revision B of the Unit 2 Technical Specifications (TS) was submitted on February 2, 2010. Adherence to Pressure and Temperature limits is required by the following portions of the Unit 2 TS: 1.1 [definition of "PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR)"]; 3.4.3 ["RCS Pressure and Temperature (P/T) Limits"]; 3.4.12 ["Cold Overpressure Mitigation System (COMS)"]; and 5.9.6 ["Reactor Coolant System (RCS) PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR)"].
B 76-001	BWR Isolation Condenser Tube Failure	NA	Boiling Water Reactor

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 76-002	Relay Coil Failures – GE Types HFA, HGA, HKA, HMA Relays	CI	Unit 2 Action: Repair or replace relays before preoperational tests.
B 76-003	Relay Malfunctions – GE Type STD Relays	C	TVA: letter dated May 17, 1976 NRC: IR 390/391 76-6
B 76-004	Cracks in Cold Worked Piping at BWRs	NA	Boiling Water Reactor
B 76-005	Relay Failures – Westinghouse BFD Relays	C	TVA: letter dated June 7, 1976 NRC: IR 390/391 85-08
B 76-006	Diaphragm Failures in Air Operated Auxiliary Actuators for Safety/Relief Valves	C	TVA: memo dated January 25, 1985 NRC: IR 390/391 85-08
B 76-007	Crane Hoist Control Circuit Modifications	C	TVA: letter dated October 29, 1976 NRC: IR 390/391 85-08
B 76-008	Teletherapy Units	NA	Does not apply to power reactor.
B 77-001	Pneumatic Time Delay Relay Setpoint Drift	C	TVA: letter dated July 1, 1977 NRC: IR 390/391 85-08
B 77-002	Potential Failure Mechanism in Certain Westinghouse AR Relays with Latch Attachments	C	TVA: letter dated November 11, 1977 NRC: IR 390/391 85-08
B 77-003	On-Line Testing of the Westinghouse Solid State Protection System	CI	Unit 2 Action: Include necessary periodic testing in test procedures.
B 77-004	Calculation Error Affecting The Design Performance of a System for Controlling pH of Containment Sump Water Following a LOCA	S 02	TVA: letter dated January 23, 1978 NRC: IR 390/78-11 and 391/78-09 --- Unit 2 Action: Ensure Technical Specifications includes limit on Boron concentration. --- --- REVISION 02 UPDATE: Developmental Revision B of the Unit 2 Technical Specifications (TS) was submitted on February 2, 2010. TS Surveillance Requirement 3.6.11.5 requires verification that the boron

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
			concentration is within a specified range.
B 77-005 and B 77-005 A	Electrical Connector Assemblies	C	TVA: letter dated January 17, 1978 NRC: IR 390/78-11 and 391/78-09
B 77-006	Potential Problems with Containment Electrical Penetration Assemblies	C	Item was applicable only to units with operating license at the time the item was issued. --- NRC: IR 390/391 85-08
B 77-007	Containment Electrical Penetration Assemblies at Nuclear Power Plants Under Construction	C	TVA: letter dated January 20, 1978 NRC: IR 390/78-11 and 391/78-09
B 77-008	Assurance of Safety and Safeguards During an Emergency – Locking Systems	C	Item concerns a multi-unit issue that was completed for both units. --- TVA: letter dated March 1, 1978 NRC: IR 390/78-11 and 391/78-09
B 78-001	Flammable Contact – Arm Retainers in GE CR120A Relays	C	TVA: letter dated March 20, 1978 NRC: IR 390/78-11 and 391/78-09
B 78-002	Terminal Block Qualification	C	TVA: letter dated March 1, 1978 NRC: IR 390/78-11 and 391/78-09
B 78-003	Potential Explosive Gas Mixture Accumulations Associated with BWR Offgas System Operations	NA	Boiling Water Reactor
B 78-004	Environmental Qualification of Certain Stem Mounted Limit Switches Inside Reactor Containment	CI	TVA: letter dated December 19, 1978 NRC: IR 390/82-13 and 391/82-10 Closed to EQ Program --- IR 50-390/82-13 and 50-391/82-10 (April 22, 1982) accepted approach. Unit 2 Action: Ensure NAMCO switches have been replaced.
B 78-005	Malfunctioning of Circuit Breaker Auxiliary Contact Mechanism – GE Model CR105X	C	TVA: letter dated June 12, 1978 NRC: IR 390/78-17 and 391/78-15
B 78-006	Defective Cutler-Hammer Type M Relays With DC Coils	C	NRC: IR 390/78-22 and 391/78-19

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ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 78-007	Protection Afforded by Air-Line Respirators and Supplied-Air Hoods	NA	Item was applicable only to units with operating license at the time the item was issued.
B 78-008	Radiation Levels from Fuel Element Transfer Tubes	NA	Item was applicable only to units with operating license at the time the item was issued. --- NRC: IR 390/391 85-08
B 78-009	BWR Drywell Leakage Paths Associated with Inadequate Drywell Closures	NA	Boiling Water Reactor
B 78-010	Bergen-Patterson Hydraulic Shock Suppressor Accumulator Spring Coils	C	TVA: letter dated August 14, 1978 NRC: IR 390/78-22 and 391/78-19
B 78-011	Examination of Mark I Containment Torus Welds	NA	Boiling Water Reactor
B 78-012	Atypical Weld Material in Reactor Pressure Vessel Welds	C	TVA: Westinghouse letter dated October 29, 1979 NRC: IR 390/391 81-04
B 78-013	Failures in Source Heads Kay Ray, Inc. Gauges Models 7050, 7050B, 7051, 7051B, 7060, 7060B, 7061 and 7061B	NA	Does not apply to power reactor.
B 78-014	Deterioration of Buna-N Components in ASCO Solenoids	NA	Boiling Water Reactor
B 79-001	Environmental Qualification of Class 1E Equipment	C	NRC: IR 390/80-06 and 391/80-05
B 79-002	Pipe Support Base Plate Designs Using Concrete Expansion Anchor Bolts	CI	NRC review of HAAUP Program in NUREG-1232, SSER6, and SSER8. Unit 2 Actions: Addressed in CAP/SP. Conduct a complete review of affected support calculations, and perform the necessary revisions to design documents and field modifications to achieve compliance.
B 79-003	Longitudinal Weld Defects in ASME SA-312 Type 304 SS Pipe Spools Manufactured by Youngstown Welding & Engineering	C	TVA: letter dated July 16, 1981 NRC: IRs 390/82-21 and 391/82-17; 390/84-35 and 391/84-33

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 79-004	Incorrect Weights for Swing Check Valves Manufactured by Velan Engineering Corporation	C	TVA: letter dated October 20, 1980 NRC: IR 390/83-15 and 391/83-11
B 79-005	Nuclear Incident at TMI	NA	Applies only to Babcock and Wilcox designed plants
B 79-006	Review of Operational Errors and System Misalignments Identified During the Three Mile Island Incident	C	NRC: IR 390/80-06 and 391/80-05
B 79-007	Seismic Stress Analysis of Safety-Related Piping	C	TVA: letter dated May 31, 1979 NRC: IR 390/79-30 and 391/79-25
B 79-008	Events Relevant to BWRs Identified During TMI Incident	NA	Boiling Water Reactor
B 79-009	Failure of GE Type AK-2 Circuit Breaker in Safety Related Systems	CI 06	TVA: letter dated June 20, 1979 --- Unit 2 Action: Complete preservice preventive maintenance on AK-2 Circuit Breakers. [WAS "NOTE 3."] --- --- REVISION 06 UPDATE: It has been confirmed that AK-2 Circuit Breakers are not used on Unit 2.
B 79-010	Requalification Training Program Statistics	NA	Item was applicable only to units with operating license at the time the item was issued.
B 79-011	Faulty Overcurrent Trip Device in Circuit Breakers for Engineering Safety Systems	C	TVA: letter dated July 20, 1979 NRC: IR 390/79-30 and 391/79-25
B 79-012	Short Period Scrams at BWR Facilities	NA	Boiling Water Reactor
B 79-013	Cracking in Feedwater Piping	C	Item was applicable only to units with operating license at the time the item was issued. --- TVA: letter dated December 1, 1983 NRC: IR 390/391 85-08

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ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 79-014	Seismic Analysis for As-Built Safety-Related Piping Systems	CI	NRC review of HAAUP Program in NUREG-1232, SSER6, and SSER8. Unit 2 Actions: * Addressed in CAP/SP. * Initiate a Unit 2 hanger walkdown and hanger analysis program similar to the program for Unit 1. * Complete re-analysis of piping and associated supports as necessary. * Perform modifications as required by re-analysis.
B 79-015	Deep Draft Pump Deficiencies	C	TVA: letter dated January 24, 1992 NRC: IR 390/391 95-70
B 79-016	Vital Area Access Controls	NA	Item was applicable only to units with operating license at the time the item was issued. --- NRC: IR 390/80-06 and 391/80-05
B 79-017	Pipe Cracks in Stagnant Borated Water Systems at PWR Plants	NA	Item was applicable only to units with operating license at the time the item was issued. NRC: IR 390/80-06 and 391/80-05; NUREG/ CR 5286
B 79-018	Audibility Problems Encountered on Evacuation of Personnel from High-Noise Areas	NA	Item was applicable only to units with operating license at the time the item was issued. --- NRC: IR 390/80-06 and 391/80-05
B 79-019	Packaging of Low-Level Radioactive Waste for Transport and Burial	NA	Item was applicable only to units with operating license at the time the item was issued. --- NRC: IR 390/80-06 and 391/80-05
B 79-020	Packaging, Transport and Burial of Low-Level Radioactive Waste	NA	Item was applicable only to units with operating license at the time the item was issued. --- NRC: IR 390/80-06 and 391/80-05

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ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 79-021	Temperature Effects on Level Measurements	C 06	Reviewed in 7.2.5 of both the original 1982 SER and SSER14. Unit 2 Action: Update accident calculation. --- CONFIRMATORY ISSUE - address IEB 79-21 to alleviate temperature dependence problem associated with measuring SG water level In SSER14, NRC concurred with TVA's assessment to not insulate the steam generator water level instrument reference leg. Unit 2 Action: Update accident calculation. --- --- REVISION 06 UPDATE: The calculations were updated. --- NRC Inspection Report 391/2010-605 closed B 79-021.
B 79-022	Possible Leakage of Tubes of Tritium Gas Used in Time Pieces for Luminosity	NA	Does not apply to power reactor. --- NRC: IR 390/80-06 and 391/80-05
B 79-023	Potential Failure of Emergency Diesel Generator Field Exciter Transformer	C	TVA: letter dated October 29, 1979 NRC: IR 390/80-06 and 391/80-05
B 79-024	Frozen Lines	CI	Unit 2 Actions: * Insulate the section of piping in the containment spray full-flow test line that is exposed to outside air. * Confirm installation of heat tracing on the sensing lines off the feedwater flow elements.
B 79-025	Failures of Westinghouse BFD Relays in Safety-Related Systems	C	TVA: letter dated January 4, 1980 NRC: IR 390/80-03 and 391/80-02
B 79-026	Boron Loss from BWR Control Blades	NA	Boiling Water Reactor
B 79-027	Loss of Non-Class 1E I & C Power System Bus During Operation	CI	TVA responded to the Bulletin on March 1, 1982. Reviewed in 7.5.3 of the original 1982 SER. Unit 2 Action: Issue appropriate emergency procedures.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 79-028	Possible Malfunction of NAMCO Model EA180 Limit Switches at Elevated Temperatures	C	TVA: letter dated April 1, 1993 NRC: IR 390/391 93-32
B 80-001	Operability of ADS Valve Pneumatic Supply	NA	Boiling Water Reactor
B 80-002	Inadequate QA for Nuclear Supplied Equipment	NA	Boiling Water Reactor
B 80-003	Loss of Charcoal from Standard Type II, 2 Inch, Tray Adsorber Cells	C	TVA: letter dated March 21, 1980 NRC: IR 390/80-15 and 391/80-12
B 80-004	Analysis of a PWR Main Steam Line Break with Continued Feedwater Addition	CI 06	IR 50-390/85-60 and 50-391/85-49 (December 6, 1985) required completion of actions that included determination of temperature profiles inside and outside of containment following a MSLB for Unit 1. Unit 2 Action: Complete analysis for Unit 2. REVISION 06 UPDATE: The analysis for Unit 2 was completed.
B 80-005	Vacuum Condition Resulting in Damage to Chemical Volume Control System Holdup Tanks	CI	Closed in IR 50-390/84-59 and 50-391/84-45. Unit 2 Action: Complete surveillance procedures for Unit 2.
B 80-006	Engineered Safety Feature Reset Control	CI	TVA response dated March 11, 1982. Reviewed in 7.3.5 of the original 1982 SER. Unit 2 Action: Perform verification during the preoperational testing.
B 80-007	BWR Jet Pump Assembly Failure	NA	Boiling Water Reactor
B 80-008	Examination of Containment Liner Penetration Welds	C	TVA: letter dated July 8, 1980 NRC: IR 390/391 81-19
B 80-009	Hydramotor Actuator Deficiencies	C	TVA: letter dated January 15, 1981 NRC: NUREG/ CR 5291; IR 390/391 85-08; IR 390/85-60 and 391/85-49

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ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 80-010	Contamination of Nonradioactive System and Resulting Potential for Unmonitored, Uncontrolled Release of Radioactivity to Environment	CI 06	Unit 2 Actions: 2) Include proper monitoring of non-radioactive systems in procedures. --- --- REVISION 06 UPDATE: Chemistry procedure CM-3.01 (System Chemistry Specification) includes a radiation monitoring system for non-radioactive systems and provides appropriate surveillance limits. Additionally, it provides required actions if the surveillance limits are not met.
B 80-010	Contamination of Nonradioactive System and Resulting Potential for Unmonitored, Uncontrolled Release of Radioactivity to Environment	CI 06	Unit 2 Actions: 1) Correct deficiencies involving monitoring of systems. --- --- REVISION 06 UPDATE: Chemistry procedure CM-3.01 (System Chemistry Specification) includes a radiation monitoring system for non-radioactive systems and provides appropriate surveillance limits. Additionally, it provides required actions if the surveillance limits are not met.
B 80-011	Masonry Wall Design	CI	NRC accepted all but completion of corrective actions in IR 50-390/93-01 and 50-391/93-01(February 25, 1993) and closed for Unit 1 in IR 50-390/95-46 (August 1, 1995). Unit 2 Action: Complete implementation for Unit 2.
B 80-012	Decay Heat Removal System Operability	CI	NRC: IR 390/391 85-08; NUREG/CR 4005 --- Unit 2 Action: Implement operating instructions and abnormal operating instructions (AOIs) for RHR. [WAS "NOTE 3."]
B 80-013	Cracking in Core Spray Spargers	NA	Boiling Water Reactor
B 80-014	Degradation of Scram Discharge Volume Capability	NA	Boiling Water Reactor
B 80-015	Possible Loss of Emergency Notification System with Loss of Offsite Power	C	Item concerns a multi-unit issue that was completed for both units. --- NRC: IR 390/391 85-08
B 80-016	Potential Misapplication of Rosemount, Inc. Models 1151 and 1152 Pressure Transmitters With Either "A" or "D" Output Codes	C	TVA: letter dated August 29, 1980 NRC: IR 390/391 81-17

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ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 80-017	Failure of 76 of 185 Control Rods to Fully Insert During a Scram at a BWR	NA	Boiling Water Reactor
B 80-018	Maintenance of Adequate Minimum Flow Thru Centrifugal Charging Pumps Following Secondary Side High Energy Rupture	CO 06	IR 50-390/85-60 and 50-391/85-49 (Unit 1) Unit 2 Action: Implement design and procedure changes. --- REVISION 06 UPDATE: NRC Inspection Report 391/2011-604 closed B 80-018.
B 80-019	Mercury-Wetted Matrix Relay in Reactor Protective Systems of Operating Nuclear Power Plants Designed by CE	C	TVA: letter dated September 4, 1980 NRC: NUREG/CR 4933; IR 390/391 81-17
B 80-020	Failure of Westinghouse Type W-2 Spring Return to Neutral Control Switches	CI 06	Unit 2 Action: Modify switches. --- REVISION 06 UPDATE: The switches were modified. --- NRC Inspection Report 391/2011-604 closed B 80-020.
B 80-021	Valve Yokes Supplied by Malcolm Foundry Co., Inc.	C	TVA: letter dated May 6, 1981 NRC: 390/391 85-08
B 80-022	Automation Industries, Model 200-520-008 Sealed-Source Connectors	NA	Does not apply to power reactor.
B 80-023	Failures of Solenoid Valves Manufactured by Valcor Engineering Corporation	C	TVA: letter dated March 31, 1981 NRC: IR 390/391 81-17; NUREG/CR 5292

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ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 80-024	Prevention of Damage Due to Water Leakage Inside Containment (10/17/80 Indian Point 2 Event)	CI 06	Unit 2 Action: Confirm that the reactor cavity can not be flooded, resulting in the partial or total submergence of the reactor vessel unnoticed by the reactor operators. REVISION 06 UPDATE: It was confirmed that the reactor cavity can not be flooded, resulting in the partial or total submergence of the reactor vessel unnoticed by the reactor operators.
B 80-025	Operating Problems with Target Rock Safety-Relief Valves at BWRs	NA	Boiling Water Reactor
B 81-001	Surveillance of Mechanical Snubbers	NA	NRC: IR 390/391 81-17
B 81-002	Failure of Gate Type Valves to Close Against Differential Pressure	C	TVA: letter dated September 30, 1983 NRC: IR 390/391 84-03
B 81-003	Flow Blockage of Cooling Water to Safety System Components by Asiatic Clams and Mussels	C	TVA: letters dated July 21, 1981 and March 21, 1983 NRC: IR 390/391 81-17
B 82-001	Alteration of Radiographs of Welds in Piping Subassemblies	C	NRC: IR 390/391 85-08
B 82-002	Degradation of Threaded Fasteners in the Reactor Coolant Pressure Boundary of PWR Plants	CI 06	TVA: memo dated February 6, 1985 NRC: IR 390/391 85-08 Approach accepted in IR 50-390/85-08 and 50-391/85-08 (March 29, 1985). Unit 2 Action: Implement same approach as Unit 1. REVISION 06 UPDATE: The boric acid corrosion program applies to both units.
B 82-003	Stress Corrosion Cracking in Thick-Wall, Large Diameter, Stainless Steel, Recirculation System Piping at BWR Plants	NA	Boiling Water Reactor

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ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 82-004	Deficiencies in Primary Containment Electrical Penetration Assemblies	C	TVA: letter dated January 24, 1983 NRC: IR 390/83-10 and 391/83-08
B 83-001	Failure of Trip Breakers (Westinghouse DB-50) to Open on Automatic Trip Signal	C	NRC: IRs 390/391 85-08 and 390/391 92-13
B 83-002	Stress Corrosion Cracking in Large-Diameter Stainless Steel Recirculation System Piping at BWR Plants	NA	Boiling Water Reactor
B 83-003	Check Valve Failures in Raw Water Cooling Systems of Diesel Generators	NA	Addressed by Inservice Testing for Construction Permit holders
B 83-004	Failure of the Undervoltage Trip Function of Reactor Trip Breakers	C 06	NRC: IR 390/391 85-08 --- Unit 2 Action: Install new undervoltage attachment with wider grooves on the reactor trip breakers. --- --- REVISION 06 UPDATE: New breakers have been installed on Unit 2. --- NRC Inspection Report 391/2011-602 closed B 83-004.
B 83-005	ASME Nuclear Code Pumps and Spare Parts Manufactured by the Hayward Tyler Pump Company	C	TVA: letter dated September 7, 1983 NRC: IR 390/85-03 and 391/85-04; NUREG/CR 5297
B 83-006	Nonconforming Material Supplied by Tube-Line Facilities	CI 04	TVA: letter dated February 2, 1984 NRC: IR 390/391 84-03; NUREG/CR 4934 --- NRC SER for both units dated September 23, 1991, provided an alternate acceptance for fittings supplied by Tube-Line. Unit 2 Action: Implement as necessary. --- --- REVISION 04 UPDATE: NRC Inspection Report Nos. 50-390/90-02 and 50-391/90-02 found the

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ITEM	TITLE	* REV	ADDITIONAL INFORMATION
			proposed alternative to ASME code paragraph NA-3451 (a) to be acceptable. It noted that TVA must revise the FSAR to document this deviation from ASME Section III requirements. TVA letter to NRC dated October 11, 2007, stated the Unit 1 exemption is applicable to Unit 2 and was submitted to the NRC as being required for Unit 2 construction. Final action was to incorporate the exemption in the Unit 2 FSAR. This exemption is documented in Unit 2 FSAR Section 3.2 in paragraph 3.2.3.2 and Table 3.2-2a as explained in Note 4. of the table.
B 83-007	Apparently Fraudulent Products Sold by Ray Miller, Inc.	C	TVA: letter dated March 22, 1984 NRC: IR 390/85-03 and 391/85-04
B 83-008	Electrical Circuit Breakers With an Undervoltage Trip Feature in Safety-Related Applications Other Than the Reactor Trip System	C	TVA: letter dated March 29, 1984 NRC: IR 390/84-35 and 391/84-33
B 84-001	Cracks in BWR Mark 1 Containment Vent Headers	NA	Boiling Water Reactor
B 84-002	Failure of GE Type HFA Relays In Use In Class 1E Safety Systems	C	TVA: letter dated July 10, 1984 NRC: IR 390/391 84-42 and IR 390/84-77 and 391/84-54
B 84-003	Refueling Cavity Water Seal	CI	Reviewed in IR 390/93-11. Unit 2 Action: Ensure appropriate abnormal operating instructions (AOIs) are used for Unit 2.
B 85-001	Steam Binding of Auxiliary Feedwater Pumps	CI	TVA: letter dated January 27, 1986 NRC: IR 390/391 90-20 --- NRC accepted approach in letter dated July 20, 1988, and reviewed response in Appendix EE of SSER16. Unit 2 Action: Procedures and hardware will be in place to ensure recognition of indications of steam binding and maintenance of system operability until check valves are repaired and back leakage stopped.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 85-002	Undervoltage Trip Attachment of Westinghouse DB-50 Type Reactor Trip Breakers	C 06	Unit 2 Action: Install automatic shunt trip on the Westinghouse DS-416 reactor trip breakers on Unit 2. --- --- REVISION 06 UPDATE: New breakers (including an automatic shunt trip) have been installed on Unit 2. --- NRC Inspection Report 391/2011-602 closed B 85-002.
B 85-003	Motor-Operated Valve Common Mode Failures During Plant Transients Due to Improper Switch Settings	C	Superseded by GL 89-10
B 86-001	Minimum Flow Logic Problems That Could Disable RHR Pumps	NA	Boiling Water Reactor
B 86-002	Static "O" Ring Differential Pressure Switches	C	TVA: letter dated November 20, 1986 NRC: IR 390/391/90-24
B 86-003	Potential Failure of Multiple ECCS Pumps Due to Single Failure of Air-Operated Valve in Minimum Flow Recirculation Line	C	TVA: letter dated November 14, 1986 NRC: IR 390/391/87-03
B 86-004	Defective Teletherapy Timer That May Not Terminate Treatment Dose	NA	Does not apply to power reactor.
B 87-001	Thinning of Pipe Walls in Nuclear Power Plants	C	TVA: letter dated September 18, 1987 NRC: NUREG/CR 5287 --- Closed to GL 89-08

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 87-002	Fastener Testing to Determine Conformance with Applicable Material Specifications	CI 03	TVA: letters dated April 15, 1988, July 6, 1988, September 12, 1988, and January 27, 1989 NRC: letter dated August 18, 1989 --- NRC closed in letter dated August 18, 1989. Unit 2 Action: Complete for Unit 2, using information used for Unit 1, as applicable. --- --- REVISION 03 UPDATE: Unit 2 has completed fastener testing as required by this Bulletin.
B 88-001	Defects in Westinghouse Circuit Breakers	C	TVA: letter dated November 15, 1991 NRC: IR 390/391 93-01
B 88-002	Rapidly Propagating Fatigue Cracks in Steam Generator Tubes	CI	NRC acceptance letter dated June 7, 1990, for both units. Unit 2 Actions: * Evaluate E/C data to determine anti-vibration bar penetration depth; * perform T/H analysis to identify susceptible tubes; * modify, if necessary.
B 88-003	Inadequate Latch Engagement in HFA Type Latching Relays Manufactured by General Electric (GE) Company	C	TVA: letter dated April 13, 1992 NRC: IR 390/391 92-13
B 88-004	Potential Safety-Related Pump Loss	CI	NRC acceptance letter dated May 24, 1990, for both units. Unit 2 Actions: * Perform calculations, and * install check valves to prevent pump to pump interaction.
B 88-005	Nonconforming Materials Supplied by Piping Supplies, Inc. and West Jersey Manufacturing Company	CI	NRC reviewed in Appendix EE of SSER16. Unit 2 Actions: * Complete review to locate installed WJM material, and * perform in-situ hardness testing for Unit 2.
B 88-006	Actions to be Taken for the Transfer of Model No. SPEC 2-T Radiographic Exposure Device	NA	Does not apply to power reactor.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 88-007	Power Oscillations in BWRs	NA	Boiling Water Reactor
B 88-008	Thermal Stresses in Piping Connected to Reactor Cooling Systems	CI	NRC acceptance letter dated September 19, 1991, for both units. Unit 2 Action: Implement program to prevent thermal stratification.
B 88-009	Thimble Tube Thinning in Westinghouse Reactors	CI 06	Reviewed in Appendix EE of SSER16. Unit 2 Action: TVA letter dated March 11, 1994, for both units committed to establish a program and inspect the thimble tubes during the first refueling outage. --- --- REVISION 06 UPDATE: Unit 2 is installing the Westinghouse In-core, Information, Surveillance, and Engineering (WINCISE) system. Westinghouse has analyzed WINCISE to exhibit essentially no wear due to vibrations, and should there be a breach of the thimble tube there would not be a loss of into the seal table room, Therefore, the thimble tubes for WINCISE do not need eddy current testing.
B 88-010	Nonconforming Molded-Case Circuit Breakers	CI	Unit 2 Action: Replace those circuits not traceable to a circuit breaker manufacturer.
B 88-011	Pressurizer Surge Line Thermal Stratification	CI	NRC SER on "Leak-Before-Break" (April 28, 1993) and reviewed in Appendix EE of SSER16. Unit 2 Actions: * Complete modifications to accommodate Surge Line thermal movements, and * incorporate a temperature limitation during heatup and cooldown operations into Unit 2 procedures.
B 89-001	Failure of Westinghouse Steam Generator Tube Mechanical Plugs	C 06	NRC acceptance letter dated September 26, 1991 for both units. Unit 2 Action: Remove SG tube plugs. --- --- REVISION 06 UPDATE: The SG tube plugs were removed. --- NRC Inspection Report 391/2011-602 closed B 89-001.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 89-002	Stress Corrosion Cracking of High-Hardness Type 410 Stainless Steel Internal Preloaded Bolting in Anchor Darling Model S350W Swing Check Valves or Valves of Similar Nature	CI 06	NRC reviewed in Appendix EE of SSER16. Unit 2 Actions: * Replace the flapper assembly hold-down bolts fabricated on the 14 (12 valves are installed) Atwood and Morrell Mark No. 47W450-53 check valves. * Replacement bolts are to be fabricated from ASTM F593 Alloy 630. * A review of the remaining Unit 2 safety related swing check valves will be performed. --- --- REVISION 06 UPDATE: * Bolts fabricated from ASTM F593 Alloy 630 have been procured. * The review of the remaining Unit 2 safety related swing check valves was completed. Needed corrective actions were initiated.
B 89-003	Potential Loss of Required Shutdown Margin During Refueling Operations	CI	TVA: letter dated June 19, 1990 NRC: IR 390/391 94-04 and letter dated June 22, 1990 --- NRC acceptance letter dated June 22, 1990. Unit 2 Action: Ensure that requirements for fuel assembly configuration, fuel loading and training are included in Unit 2.
B 90-001	Loss of Fill-Oil in Transmitters Manufactured by Rosemount	CO 06	Unit 2 Action: Implement applicable recommendations from this Bulletin including identification of potentially defective transmitters and an enhanced surveillance program which monitors transmitters for loss of fill oil. --- --- REVISION 06 UPDATE: NRC Inspection Report 391/2011-603 closed B 90-001.
B 90-002	Loss of Thermal Margin Caused by Channel Box Bow	NA	Boiling Water Reactor
B 91-001	Reporting Loss of Criticality Safety Controls	NA	Does not apply to power reactor.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 92-001	Failure of Thermo-Lag 330 Fire Barrier System to Maintain Cabling in Wide Cable Trays and Small Conduits Free From Fire Damage	NA 02	REVISION 02 UPDATE: This bulletin was provided for information only to plants with construction permits. See Generic Letter 92-08 for Thermo-lag related actions.
B 92-002	Safety Concerns Related to "End of Life" of Aging Theratronics Teletherapy Units	NA	Does not apply to power reactor.
B 92-003	Release of Patients After Brachytherapy	NA	Does not apply to power reactor.
B 93-001	Release of Patients After Brachytherapy Treatment with Remote Afterloading Devices	NA	Does not apply to power reactor.
B 93-002	Debris Plugging of Emergency Core Cooling Suction Strainers	C 02	Boiling Water Reactor REVISION 02 UPDATE: In Rev. 01, this was characterized as "NA - BWR only". This Bulletin was provided for Information to holders of construction permits. No WBN response was found. B-93-02 was closed in IR 50-390/94-04 and 50-391/94-04.
B 93-003	Resolution of Issues Related to Reactor Vessel Water Level Instrumentation in BWRs	NA	Boiling Water Reactor
B 94-001	Potential Fuel Pool Draindown Caused by Inadequate Maintenance Practices at Dresden Unit 1	NA	Addressed to holders of licenses for nuclear power reactors that are permanently shut down with spent fuel in the spent fuel pool
B 94-002	Corrosion Problems in Certain Stainless Steel Packagings Used to Transport Uranium Hexafluoride	NA	Does not apply to power reactor.
B 95-001	Quality Assurance Program for Transportation of Radioactive Material	NA	Does not apply to power reactor.
B 95-002	Unexpected Clogging of a Residual Heat Removal Pump Strainer While Operating in Suppression Pool Cooling Mode	NA	Boiling Water Reactor

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 96-001, first part	Control Rod Insertion Problems (PWR)	CI 04	NRC acceptance letter for Unit 1 dated July 22, 1996 – Initial response for Unit 2 on September 7, 2007. Unit 2 Action: Issue Emergency Operating Procedure. --- --- REVISION 02 UPDATE: Unit 2 will load all new RFA-2 fuel for the initial fuel load. --- --- REVISION 03 UPDATE: NRC issued the Safety Evaluation (corrected) for Bulletin 1996-001 on May 3, 2010. --- --- REVISION 04 UPDATE: Corrected status from "OV" to "CI" due to NRC issuance of Safety Evaluation as noted in Revision 03 update.
B 96-001, last part	Control Rod Insertion Problems (PWR)	CI 06	NRC acceptance letter for Unit 1 dated July 22, 1996 – Initial response for Unit 2 on September 7, 2007. Unit 2 Action: and provide core map. --- --- REVISION 03 UPDATE: NRC issued the Safety Evaluation (corrected) for Bulletin 1996-001 on May 3, 2010. --- --- REVISION 04 UPDATE: Corrected status from "OV" to "CI" due to NRC issuance of Safety Evaluation as noted in Revision 03 update. --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC letter dated May 3, 2010 (ADAMS Accession No. ML101200035) required Confirmatory Action (See Appendix HH)" --- The applicable item from SER22, Appendix HH for this item is Open

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
			Item 5, "Verify timely submittal of pre-startup core map and perform technical review. (TVA letter dated September 7, 2007, ADAMS Accession No. ML072570676)." TVA to NRC letter dated April 6, 2011 provided the following response to Open Item 5: "Attachment 1 provides the requested core map."
B 96-002	Movement of Heavy Loads over Spent Fuel, Over Fuel in the Reactor, or Over Safety-Related Equipment	CI 06	NRC closure letter dated May 20, 1998. Unit 2 Action: Unit 2 Heavy Loads Program will be in compliance with NUREG-0612. --- --- REVISION 02 UPDATE: NRC issued the Safety Evaluation for Bulletin 1996-002 on March 4, 2010. --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC letter dated March 4, 2010 (ADAMS Accession No. ML100480062)"
B 96-003	Potential Plugging of ECCS Suction Strainers by Debris in BWRs	NA	Boiling Water Reactor
B 96-004	Chemical, Galvanic, or Other Reactions in Spent Fuel Storage and Transportation Casks	NA	Info
B 97-001	Potential for Erroneous Calibration, Dose Rate, or Radiation Exposure Measurements with Certain Victoreen Model 530 and 531SI Electrometer/Dosemeters	NA	Does not apply to power reactor.
B 97-002	Puncture Testing of Shipping Packages Under 10 CFR Part 71	NA	Does not apply to power reactor.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 01-001	Circumferential Cracking of Reactor Pressure Vessel (RPV) Head Penetration Nozzles	C 06	NRC acceptance letter dated November 20, 2001 (Unit 1) – Initial response for Unit 2 on September 7, 2007. Unit 2 Action: Perform baseline inspection. --- --- REVISION 02 UPDATE: Unit 2 Actions: * Perform baseline inspection. * Evaluate or repair as necessary. --- --- REVISION 03 UPDATE: NRC issued the Safety Evaluation for Bulletin 2001-001 on June 30, 2010. --- --- REVISION 04 UPDATE: Corrected status from "OV" to "CI" due to NRC issuance of Safety Evaluation as noted in Revision 03 update. --- --- REVISION 06 UPDATE: The baseline inspection was performed with evaluations and repairs as necessary. --- SSER22 contained the following for NRC Action: "Closed. See NRC Letter dated June 30, 2010 (ADAMS Accession No. ML 100539515)" --- NRC Inspection Report 391/2011-602 closed B 01-001.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 02-001	RPV Head Degradation and Reactor Coolant Pressure Boundary Integrity	C 06	NRC review of Unit 1's 15 day response in letter dated May 20, 2002 – Initial response for Unit 2 on September 7, 2007. Unit 2 Action: Perform baseline inspection. --- --- REVISION 02 UPDATE: Unit 2 Actions: * Perform baseline inspection. * Evaluate or repair as necessary. --- --- REVISION 03 UPDATE: NRC issued the Safety Evaluation for Bulletin 2002-001 on June 30, 2010. --- --- REVISION 04 UPDATE: Corrected status from "OV" to "CI" due to NRC issuance of Safety Evaluation as noted in Revision 03 update. --- --- REVISION 06 UPDATE: The baseline inspection was performed with evaluations and repairs as necessary. --- SSSER22 contained the following for NRC Action: "Closed. See NRC Letter dated June 30, 2010 (ADAMS Accession No. ML 100539515)" --- NRC Inspection Report 391/2011-602 closed B 02-001.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 02-002	RPV Head and Vessel Head Penetration Nozzle Inspection Programs	C 06	NRC acceptance letter dated December 20, 2002 (Unit 1) – Initial response for Unit 2 on September 7, 2007. Unit 2 Action: Perform baseline inspection. --- --- REVISION 02 UPDATE: Unit 2 Actions: * Perform baseline inspection. * Evaluate or repair as necessary. --- --- REVISION 03 UPDATE: NRC issued the Safety Evaluation for Bulletin 2002-002 on June 30, 2010. --- --- REVISION 04 UPDATE: Corrected status from "OV" to "CI" due to NRC issuance of Safety Evaluation as noted in Revision 03 update. --- --- REVISION 06 UPDATE: The baseline inspection was performed with evaluations and repairs as necessary. --- SSSER22 contained the following for NRC Action: "Closed. See NRC Letter dated June 30, 2010 (ADAMS Accession No. ML 100539515)" --- NRC Inspection Report 391/2011-602 closed B 02-002.
B 03-001	Potential Impact of Debris Blockage on Emergency Sump Recirculation at PWRs	NA	TVA: letter dated September 7, 2007

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 03-002	Leakage from RPV Lower Head Penetrations and Reactor Coolant Pressure Boundary Integrity (PWRs)	CI 06	NRC acceptance letter dated October 6, 2004 (Unit 1) – Initial response for Unit 2 on September 7, 2007. Unit 2 Action: Perform baseline inspection. --- --- REVISION 02 UPDATE: NRC issued the Safety Evaluation for Bulletin 2003-002 on January 21, 2010. Unit 2 Actions: * Perform baseline inspection. * Evaluate or repair as necessary. --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML093631061)"
B 03-003	Potentially Deficient 1-inch Valves for Uranium Hexaflouride Cylinders	NA	Does not apply to power reactor.
B 03-004	Rebaselining of Data in the Nuclear Management and Safeguards System	C	TVA: letter dated December 18, 2003 --- Item concerns a multi-unit issue that was completed for both units.
B 04-001	Inspection of Alloy 82/182/600 Materials Used in the Fabrication of Pressurizer Penetrations and Steam Space Piping Connections at PWRs	CI 06	Initial response for Unit 2 on September 7, 2007. Unit 2 Actions: * Provide details of pressurizer and penetrations, and * apply Material Stress Improvement Process. --- --- REVISION 02 UPDATE: TVA provided details of the pressurizer and penetrations on September 29, 2008. This letter committed to: Prior to placing the pressurizer in service, TVA will apply the Material Stress Improvement Process (MSIP) to the Pressurizer Power Operated Relief Valve connections, the safety relief valve connections, the spray line nozzle and surge line nozzle connections.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
			TVA will perform a bare metal visual (BMV) inspection of the upper pressurizer Alloy 600 locations at the first refueling outage. --- --- REVISION 03 UPDATE: April 1, 2010, letter committed to: TVA will perform NDE prior to and after performance of the MSIP. If circumferential cracking is observed in either pressure boundary or non-pressure boundary portions of any locations covered under the scope of the bulletin, TVA will develop plans to perform an adequate extent-of-condition evaluation, and TVA will discuss those plans with cognizant NRC technical staff prior to starting Unit 2. After performing the BMV inspection during the first refueling outage, if any evidence of apparent reactor coolant pressure boundary leakage is discovered, then NDE capable of determining crack orientation will be performed in order to accurately characterize the flaw, the orientation, and extent. TVA will develop plans to perform an adequate extent of condition evaluation, and plans to possibly expand the scope of NDE to other components in the pressurizer will be discussed with NRC technical staff prior to restarting of Unit 2. --- --- REVISION 04 UPDATE: NRC issued the Safety Evaluation for Bulletin 2004-001 on August 4, 2010. --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC Letter dated August 4, 2010 (ADAMS Accession No. ML102080017)"
B 05-001	Material Control and Accounting at Reactors and Wet Spent Fuel Storage Facilities	C	TVA: letters dated March 21, 2005 and May 11, 2005 --- Item concerns a multi-unit issue that was completed for both units.
B 05-002	Emergency Preparedness and Response Actions for Security-Based Events	C	TVA: letters dated January 20, 2006 and August 16, 2006. --- Item concerns a multi-unit issue that was completed for both units.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
B 07-001	Security Officer Attentiveness	C 06	Item concerns a multi-unit issue that was completed for both units. REVISION 05 UPDATE: The NRC closed this bulletin via letter dated March 25, 2010 (ADAMS Accession No. ML100770549). REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC Letter dated March 25, 2010 (ADAMS Accession No. ML 100770549)"
C 76-001	Crane Hoist Control Circuit Modifications	C	See B 76-007 for additional information.
C 76-002	Relay Failures - Westinghouse BF (AC) and BFD (DC) Relays	C	TVA: letter dated November 22, 1976 informed NRC that these relay types are not used in Class 1E circuits. NRC: IR 50/390/76-11 and 50/391/76-11
C 76-003	Radiation Exposures in Reactor Cavities	NA	Info
C 76-004	Neutron Monitor and Flow Bypass Switch Malfunctions	NA	Boiling Water Reactor
C 76-005	Hydraulic Shock And Sway Suppressors - Maintenance of Bleed and Lock-Up Velocities on ITT Grinnell's Model Nos. - Fig. 200 And Fig. 201, Catalog Ph-74-R	C	TVA: letter dated January 7, 1977 informed NRC that no Grinnell shock suppressors or sway braces have been or will be installed at WBN.
C 76-006	Stress Corrosion Cracks in Stagnant, Low Pressure Stainless Piping Containing Boric Acid Solution at PWRs	NA	Item was applicable only to units with operating license at the time the item was issued.
C 76-007	Inadequate Performance by Reactor Operating and Support Staff Members	NA	Item was applicable only to units with operating license at the time the item was issued.
C 77-001	Malfunctions of Limitorque Valve Operators	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
C 77-002a	Potential Heavy Spring Flooding (CP)	NA	Item was applicable only to units with operating license at the time the item was issued.
C 77-003	Fire Inside a Motor Control Center	NA	Info
C 77-004	Inadequate Lock Assemblies	NA	Info
C 77-005	Fluid Entrapment in Valve Bonnets	NA	Info
C 77-006	Effects of Hydraulic Fluid on Electrical Cables	NA	Info
C 77-007	Short Period During Reactor Startup	NA	Boiling Water Reactor
C 77-008	Failure of Feedwater Sample Probe	NA	Item was applicable only to units with operating license at the time the item was issued.
C 77-009	Improper Fuse Coordination in BWR Standby Liquid Control System Control Circuits	NA	Boiling Water Reactor
C 77-010	Vacuum Conditions Resulting in Damage to Liquid Process Tanks	NA	Item was applicable only to units with operating license at the time the item was issued.
C 77-011	Leakage of Containment Isolation Valves with Resilient Seats	NA	Info
C 77-012	Dropped Fuel Assemblies at BWR Facilities	NA	Boiling Water Reactor
C 77-013	Reactor Safety Signals Negated During Testing	NA	Info
C 77-014	Separation of Contaminated Water Systems from Noncontaminated Plant Systems	NA	Info
C 77-015	Degradation of Fuel Oil Flow to the Emergency Diesel Generator	NA	Info
C 77-016	Emergency Diesel Generator Electrical Trip Lock-Out Features	NA	Info
C 78-001	Loss of Well Logging Source	NA	Does not apply to power reactor.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
C 78-002	Proper Lubricating Oil for Terry Turbines	NA	Info
C 78-003	Packaging Greater Than Type A Quantities of Low Specific Activity Radioactive Material for Transport	NA	Info
C 78-004	Installation Errors That Could Prevent Closing of Fire Doors	NA	Info
C 78-005	Inadvertent Safety Injection During Cooldown	NA	Info
C 78-006	Potential Common Mode Flooding of ECCS Equipment Rooms at BWR Facilities	NA	Info
C 78-007	Damaged Components of a Bergen-Paterson Series 25000 Hydraulic Test Stand	NA	Info
C 78-008	Environmental Qualification of Safety-Related Electrical Equipment at Nuclear Power Plants	NA	Info
C 78-009	Arcing of General Electric Company Size 2 Contactors	NA	Info
C 78-010	Control of Sealed Sources in Radiation Therapy	NA	Does not apply to power reactor.
C 78-011	Recirculation MG Set Overspeed Stops	NA	Boiling Water Reactor
C 78-012	HPCI Turbine Control Valve Lift Rod Bending	NA	Boiling Water Reactor
C 78-013	Inoperability of Service Water Pumps	NA	Info
C 78-014	HPCI Turbine Reversing Chamber Hold Down Bolting	NA	Boiling Water Reactor
C 78-015	Tilting Disc Check Valves Fail to Close with Gravity in Vertical Position	NA	Info
C 78-016	Limitorque Valve Actuators	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
C 78-017	Inadequate Guard Training/Qualification and Falsified Training Records	NA	Info
C 78-018	UL Fire Test	NA	Info
C 78-019	Manual Override (Bypass) of Safety System Actuation Signals	NA	Info
C 79-001	Administration of Unauthorized Byproduct Material to Humans	NA	Does not apply to power reactor.
C 79-002	Failure of 120 Volt Vital AC Power Supplies	NA	Info
C 79-003	Inadequate Guard Training - Qualification and Falsified Training Records	NA	Info
C 79-004	Loose Locking Nut on Limitorque Valve Operators	NA	Info
C 79-005	Moisture Leakage in Stranded Wire Conductors	NA	Info
C 79-006	Failure to Use Syringe and Bottle Shields in Nuclear Medicine	NA	Does not apply to power reactor.
C 79-007	Unexpected Speed Increase of Reactor Recirculation MG Set Resulted in Reactor Power Increase	NA	Boiling Water Reactor
C 79-008	Attempted Extortion - Low Enriched Uranium	NA	Fuel facilities and operating reactors at the time the item was issued
C 79-009	Occurrences of Split or Punctured Regulator Diaphragms in Certain Self Contained Breathing Apparatus	NA	Info
C 79-010	Pipefittings Manufactured from Unacceptable Material	NA	Info
C 79-011	Design/Construction Interface Problem	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
C 79-012	Potential Diesel Generator Turbocharger Problem	NA	Info
C 79-013	Replacement of Diesel Fire Pump Starting Contactors	NA	Info
C 79-014	Unauthorized Procurement and Distribution of XE-133	NA	Does not apply to power reactor.
C 79-015	Bursting of High Pressure Hose and Malfunction of Relief Valve O-Ring in Certain Self-Contained Breathing Apparatus	NA	Item was applicable only to units with operating license at the time the item was issued.
C 79-016	Excessive Radiation Exposures to Members of the General Public and a Radiographer	NA	Does not apply to power reactor.
C 79-017	Contact Problem in SB-12 Switches on General Electric Company Metalclad Circuit Breakers	NA	Info
C 79-018	Proper Installation of Target Rock Safety-Relief Valves	NA	Boiling Water Reactor
C 79-019	Loose Locking Devices on Ingersoll-Rand Pumps	NA	Info
C 79-020	Failure of GTE Sylvania Relay Type PM Bulletin 7305 Catalog 5U12-11-AC with a 120V AC Coil	NA	Info
C 79-021	Prevention of Unplanned Releases of Radioactivity	NA	Info
C 79-022	Stroke Times for Power Operated Relief Valves	NA	Info
C 79-023	Motor Starters and Contactors Failed to Operate	C 01	The Circular did not require a response. TVA reported a nonconformance under 10 CFR 50.55e on January 17, 1980, that four motor starters of this type had been located in the 480V control and auxiliary vent boards at WBN. Gould factory representatives supervised the replacement of the carrier assemblies in accordance with the Gould instructions. The starters with replaced carriers were acceptable. NRC IR 50-390/80-03 and 50-391/80-02 reviewed and closed the associated nonconformance reports.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
C 79-024	Proper Installation and Calibration of Core Spray Pipe Break Detection Equipment on BWRs	NA	Boiling Water Reactor
C 79-025	Shock Arrestor Strut Assembly Interference	C 01	The Circular did not require a response. TVA reported a nonconformance under 10 CFR 50.55e on March 6, 1980, that a review had determined that nine installed supports had brackets with the potential of hindering full function of the support. Additional supports that were not installed had the same potential problem. TVA initially determined that the supports would be modified in accordance with a vendor approved drawing. TVA subsequently determined that no actual problem existed and no field work was required. NRC IR 50-390/83-15 and 50-391/83-11 reviewed and closed the associated nonconformance reports.
C 80-001	Service Advice for GE Induction Disc Relays	NA	Info
C 80-002	Nuclear Power Plant Staff Work Hours	NA	Info
C 80-003	Protection from Toxic Gas Hazards	NA	Info
C 80-004	Securing of Threaded Locking Devices on Safety-Related Equipment	NA	Info
C 80-005	Emergency Diesel-Generator Lubricating Oil Addition and Onsite Supply	NA	Info
C 80-006	Control and Accountability Systems for Implant Therapy Sources	NA	Does not apply to power reactor.
C 80-007	Problems with HPCI Turbine Oil System	NA	Boiling Water Reactor
C 80-008	BWR Technical Specification Inconsistency - RPS Response Time	NA	Boiling Water Reactor
C 80-009	Problems with Plant Internal Communications Systems	NA	Info
C 80-010	Failure to Maintain Environmental Qualification of Equipment	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
C 80-011	Emergency Diesel Generator Lube Oil Cooler Failures	NA	Info
C 80-012	Valve-Shaft-to-Actuator Key May Fall Out of Place when Mounted Below Horizontal Axis	NA	Info
C 80-013	Grid Strap Damage in Westinghouse Fuel Assemblies	NA	Info
C 80-014	Radioactive Contamination of Plant Demineralized Water System and Resultant Internal Contamination of Personnel	NA	Info
C 80-015	Loss of Reactor Coolant Pump Cooling and Natural Circulation Cooldown	NA	Info
C 80-016	Operational Deficiencies in Rosemount Model 510DU Trip Units and Model 1152 Pressure Transmitters	NA	Info
C 80-017	Fuel Pin Damage Due to Water Jet from Baffle Plate Corner	NA	Info
C 80-018	10 CFR 50.59 Safety Evaluations for Changes to Radioactive Waste Treatment Systems	NA	Info
C 80-019	Noncompliance with License Requirements for Medical Licensees	NA	Does not apply to power reactor.
C 80-020	Changes in Safe-Slab Tank Dimensions	NA	Info
C 80-021	Regulation of Refueling Crews	NA	Item was applicable only to units with operating license at the time the item was issued.
C 80-022	Confirmation of Employee Qualifications	NA	Info
C 80-023	Potential Defects in Beloit Power Systems Emergency Generators	NA	Info
C 80-024	AECL Teletherapy Unit Malfunction	NA	Does not apply to power reactor.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
C 80-025	Case Histories of Radiography Events	NA _ _ .	Does not apply to power reactor.
C 81-001	Design Problems Involving Indicating Pushbutton Switches Manufactured by Honeywell Incorporated	NA _ _ .	Info
C 81-002	Performance of NRC-Licensed Individuals while on Duty	NA _ _ .	Item was applicable only to units with operating license at the time the item was issued.
C 81-003	Inoperable Seismic Monitoring Instrumentation	NA _ _ .	Info
C 81-004	The Role of Shift Technical Advisors and Importance of Reporting Operational Events	NA _ _ .	Info
C 81-005	Self-Aligning Rod End Bushings for Pipe Supports	NA _ _ .	Info
C 81-006	Potential Deficiency Affecting Certain Foxboro 10 to 50 Milliampere Transmitters	NA _ _ .	Info
C 81-007	Control of Radioactively Contaminated Material	NA _ _ .	Info
C 81-008	Foundation Materials	NA _ _ .	Info
C 81-009	Containment Effluent Water that Bypasses Radioactivity Monitor	NA _ _ .	Info
C 81-010	Steam Voiding in the Reactor Coolant System During Decay Heat Removal Cooldown	NA _ _ .	Item was applicable only to units with operating license at the time the item was issued.
C 81-011	Inadequate Decay Heat Removal During Reactor Shutdown	NA _ _ .	Boiling Water Reactor
C 81-012	Inadequate Periodic Test Procedure of PWR Reactor Protection System	NA _ _ .	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
C 81-013	Torque Switch Electrical Bypass Circuit for Safeguard Service Valve Motors	C 01	The Circular did not require a response. TVA reported a nonconformance under 10 CFR 50.55e on April 4, 1986 (NCR W367-P), that required closing torque switches were found improperly wired. This issue (Torque switch and overload relay bypass capability for active safety related valves) is part of the Electrical Issues Corrective Action Program for WBN Unit 2.
C 81-014	Main Steam Isolation Valve Failures to Close	NA	Info
C 81-015	Unnecessary Radiation Exposures to the Public and Workers During Events Involving Thickness and Level Measuring Devices	NA	Info
GL 77-001	Intrusion Detection Systems Handbook	NA	Info
GL 77-002	Fire Protection Functional Responsibilities	NA	Info
GL 77-003	Transmittal of NUREG-0321, "A Study of the Nuclear Regulatory Commission Quality Assurance Program"	NA	Info
GL 77-004	Shipments of Contaminated Components From NRC Licensed Power Facilities to Vendors & Service Companies	NA	Info
GL 77-005	Nonconformity of Addressees of Items Directed to the Office of Nuclear Reactor Regulation	NA	Info
GL 77-006	Enclosing Questionnaire Related to Steam Generators	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 77-007	Reliability of Standby Diesel Generator Units	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 77-008	Revised Intrusion Detection Handbook and Entry Control Systems Handbook	NA	Info
GL 78-001	Correction to Letter of 12/15/77 [GL 77-07]	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 78-002	Asymmetric Loads Background and Revised Request for Additional Information	C	NRC: Reviewed in SSER15 – Appendix C (June 1995). Resolved by approval of leak-before-break analysis.

*** = See last page for status code definition.**

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 78-003	Request For Information on Cavity Annulus Seal Ring	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 78-004	GAO Blanket Clearance for Letter Dated 12/09/77 [GL 77-06]	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 78-005	Internal Distribution of Correspondence – Asking for Comments on Mass Mailing System	NA	Info
GL 78-006	This GL was never issued.	NA	
GL 78-007	This GL was never issued.	NA	
GL 78-008	Enclosing NUREG-0408 Re Mark I Containments, and Granting Exemption from GDC 50 and Enclosing Sample Notice	NA	Boiling Water Reactor
GL 78-009	Multiple-Subsequent Actuations of Safety/Relief Valves Following an Isolation Event	NA	Boiling Water Reactor
GL 78-010	Guidance on Radiological Environmental Monitoring	NA	Info
GL 78-011	Guidance on Spent Fuel Pool Modifications	NA	Info
GL 78-012	Notice of Meeting Regarding "Implementation of 10 CFR 73.55 Requirements and Status of Research ..."	NA	Info
GL 78-013	Forwarding of NUREG-0219	NA	Info
GL 78-014	Transmittal of Draft NUREG-0219 for Comment	NA	Info
GL 78-015	Request for Information on Control of Heavy Loads Near Spent Fuel	NA	See GL 81-007.
GL 78-016	Request for Information on Control of Heavy Loads Near Spent Fuel Pools	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 78-017	Corrected Letter on Heavy Loads Over Spent Fuel	NA	Info
GL 78-018	Corrected Letter on Heavy Loads Over Spent Fuel	NA	Duplicate of GL 81-007
GL 78-019	Enclosing Sandia Report SAND 77-0777, "Barrier Technology Handbook"	NA	Info
GL 78-020	Enclosing – "A Systematic Approach to the Conceptual Design of Physical Protection Systems for Nuclear Facilities	NA	Info
GL 78-021	Transmitting NUREG/CR-0181, "Concerning Barrier and Penetration Data Needed for Physical Security System Assessment"	NA	Info
GL 78-022	Revision to Intrusion Detection Systems and Entry Control Systems Handbooks and Nuclear Safeguards Technology Handbook	NA	Info
GL 78-023	Manpower Requirements for Operating Reactors	NA	Info
GL 78-024	Model Appendix I Technical Specifications and Submittal Schedule For BWRs	NA	Boiling Water Reactor
GL 78-025	This GL was never issued.	NA	
GL 78-026	Excessive Control Rod Guide Tube Wear	NA	Applies only to Babcock and Wilcox designed plants
GL 78-027	Forwarding of NUREG-0181	NA	Info
GL 78-028	Forwarding pages omitted from 07/11/78 letter [GL 78-24]	NA	Boiling Water Reactor
GL 78-029	Notice of PWR Steam Generator Conference	NA	Info
GL 78-030	Forwarding of NUREG-0219	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 78-031	Notice of Steam Generator Conference Agenda	NA	Info
GL 78-032	Reactor Protection System Power Supplies	NA	Boiling Water Reactor
GL 78-033	Meeting Schedule and Locations For Upgraded Guard Qualification	NA	Info
GL 78-034	Reactor Vessel Atypical Weld Material	C	See B 78-12.
GL 78-035	Regional Meetings to Discuss Upgraded Guard Qualifications	NA	Info
GL 78-036	Cessation of Plutonium Shipments by Air Except In NRC Approved Containers	NA	Does not apply to power reactor.
GL 78-037	Revised Meeting Schedule & Locations For Upgraded Guard Qualifications	NA	Info
GL 78-038	Forwarding of 2 Tables of Appendix I, Draft Radiological Effluent Technical Specifications, PWR, and NUREG-0133	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 78-039	Forwarding of 2 Tables of Appendix I, Draft Radiological Effluent Technical Specifications, BWR, and NUREG-0133	NA	Boiling Water Reactor
GL 78-040	Training & Qualification Program Workshops	NA	Info
GL 78-041	Mark II Generic Acceptance Criteria For Lead Plants	NA	Boiling Water Reactor
GL 78-042	Training and Qualification Program Workshops	NA	Info
GL 79-001	Interservice Procedures for Instructional Systems Development - TRADOC	NA	Info
GL 79-002	Transmitting Rev. to Entry Control Systems Handbook (SAND 77-1033), Intrusion Detection Handbook (SAND 76-0554), and Barrier Penetration Database	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 79-003	Offsite Dose Calculation Manual	NA	Info
GL 79-004	Referencing 4/14/78 Letter - Modifications to NRC Guidance "Review and Acceptance of Spent Fuel Pool Storage and Handling"	NA	Info
GL 79-005	Information Relating to Categorization of Recent Regulatory Guides by the Regulatory Requirements Review Committee	NA	Info
GL 79-006	Contents of the Offsite Dose Calculation Manual	NA	Info
GL 79-007	Seismic (SSE) and LOCA Responses (NUREG-0484)	NA	Info
GL 79-008	Amendment to 10 CFR 73.55	NA	Info
GL 79-009	Staff Evaluation of Interim Multiple-Consecutive Safety-Relief Valve Actuations	NA	Boiling Water Reactor
GL 79-010	Transmitting Regulatory Guide 2.6 for Comment	NA	Does not apply to power reactor.
GL 79-011	Transmitting "Summary of Operating Experience with Recalculating Steam Generators, January 1979," NUREG-0523	NA	Info
GL 79-012	ATWS - Enclosing Letter to GE, with NUREG-0460, Vol. 3	NA	Info
GL 79-013	Schedule for Implementation and Resolution of Mark I Containment Long Term Program	NA	Info
GL 79-014	Pipe Crack Study Group - Enclosing NUREG-0531 and Notice	NA	Info
GL 79-015	Steam Generators - Enclosing Summary of Operating Experience with Recirculating Steam Generators, NUREG-0523	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 79-016	Meeting Re Implementation of Physical Security Requirements	NA	Info
GL 79-017	Reliability of Onsite Diesel Generators at Light Water Reactors	NA	Info
GL 79-018	Westinghouse Two-Loop NSSS	NA	Addressed to specific plant(s).
GL 79-019	NRC Staff Review of Responses to Bs 79-06 and 79-06a	NA	Addressed to specific plant(s).
GL 79-020	Cracking in Feedwater Lines	C	See B 79-13.
GL 79-021	Enclosing NUREG/CR-0660, Enhancement of on Site Emergency Diesel Generator Reliability"	NA	Info
GL 79-022	Enclosing NUREG-0560, "Staff Report on the Generic Assessment of Feedwater Transients in PWRs Designed by B&W"	NA	Applies only to Babcock and Wilcox designed plants
GL 79-023	NRC Staff Review of Responses to B 79-08	NA	Boiling Water Reactor
GL 79-024	Multiple Equipment Failures in Safety-Related Systems	NA 01	GL 79-24 provided a discussion of an inadvertent reactor scram and safety injection during monthly surveillance tests of the safeguards system at a PWR facility. The GL requested a review to determine if similar errors had or could have occurred at other PWRs. The GL further requested a review of management policies and procedures to assure that multiple equipment failures in safety-related systems will be vigorously pursued and analyzed to identify significant reduction in the ability of safety systems to function as required. A response was requested within 30 days of receipt of the GL with the results of these reviews. TVA does not have a record of receiving or responding to this GL. Thus, TVA concluded that this item was applicable only to PWRs with an operating license at the time the GL was issued.
GL 79-025	Information Required to Review Corporate Capabilities	NA	Info
GL 79-026	Upgraded Standard Technical Specification Bases Program	NA	Info
GL 79-027	Operability Testing of Relief and Safety Relief Valves	NA	Boiling Water Reactor

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 79-028	Evaluation of Semi-Scale Small Break Experiment	NA	Info
GL 79-029	Transmitting NUREG-0473, Revision 2, Draft Radiological Effluent Technical Specifications	NA	Info
GL 79-030	Transmitting NUREG-0472, Revision 2, Draft Radiological Technical Specifications	NA	Info
GL 79-031	Submittal of Copies of Response to 6/29/79 NRC Request [79-25]	NA	Info
GL 79-032	Transmitting NUREG-0578, "TMI-2 Lessons Learned"	NA	Info
GL 79-033	Transmitting NUREG-0576, "Security Training and Qualification Plans"	NA	Info
GL 79-034	New Physical Security Plans (FR 43280-285)	NA	Does not apply to power reactor.
GL 79-035	Regional Meetings to Discuss Impacts on Emergency Planning	NA	Info
GL 79-036	Adequacy of Station Electric Distribution Systems Voltages	CI	This GL tracked compliance with BTP PSB-1, "Adequacy of Station Electric Distribution System Voltages." Unit 2 Action: Perform verification during the preoperational testing.
GL 79-037	Amendment to 10 CFR 73.55 Deferral from 8/1/79 to 11/1/79	NA	Info
GL 79-038	BWR Off-Gas Systems - Enclosing NUREG/CR-0727	NA	Boiling Water Reactor
GL 79-039	Transmitting Division 5 Draft Regulatory Guide and Value Impact Statement	NA	Does not apply to power reactor.
GL 79-040	Follow-up Actions Resulting from the NRC Staff Reviews Regarding the TMI-2 Accident	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 79-041	Compliance with 40 CFR 190, EPA Uranium Fuel Cycle Standard	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 79-042	Potentially Unreviewed Safety Question on Interaction Between Non-Safety Grade Systems and Safety Grade Systems	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 79-043	Reactor Cavity Seal Ring Generic Issue	NA	Addressed to specific plant(s).
GL 79-044	Referencing 6/29/79 Letter Re Multiple Equipment Failures	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 79-045	Transmittal of Reports Regarding Foreign Reactor Operating Experiences	NA	Info
GL 79-046	Containment Purge and Venting During Normal Operation – Guidelines for Valve Operability	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 79-047	Radiation Training	NA	Info
GL 79-048	Confirmatory Requirements Relating to Condensation Oscillation Loads for the Mark I Containment Long Term Program	NA	Boiling Water Reactor
GL 79-049	Summary of Meetings Held on 9/18-20/79 to Discuss Potential Unreviewed Safety Question on Systems Interaction for B&W Pl	NA	Info
GL 79-050	Emergency Plans Submittal Dates	NA	Info
GL 79-051	Follow-up Actions Resulting from the NRC Staff Reviews Regarding the TMI-2 Accident	NA 01	GL 79-51 provided follow-up actions resulting from the Three Mile Island Unit 2 accident. GL 79-51 was provided for planning and guidance purposes. Its principal element was a report titled "TMI-2 Lessons Learned Task Force Status Report and Short-Term Recommendations" (NUREG-0573). This GL and the NUREG were superseded by GL 80-90 and NUREG-0737. See GL 80-90 for further information.
GL 79-052	Radioactive Release at North Anna Unit 1 and Lessons Learned	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 79-053	ATWS	NA	Info
GL 79-054	Containment Purging and Venting During Normal Operation	NA	Addressed to specific plant(s).

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 79-055	Summary of Meeting Held on October 12, 1979 to Discuss Responses to Bulletins 79-05C and 79-06C and HPI Termination Criteria	NA	Info
GL 79-056	Discussion of Lessons Learned Short Term Requirements	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 79-057	Acceptance Criteria for Mark I Long Term Program	NA	Boiling Water Reactor
GL 79-058	ECCS Calculations on Fuel Cladding	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 79-059	This GL was never issued.	NA	
GL 79-060	Discussion of Lessons Learned Short Term Requirements	NA	Info
GL 79-061	Discussion of Lessons Learned Short Term Requirements	NA	Info
GL 79-062	ECCS Calculations on Fuel Cladding	NA	Item was applicable only to units with operating license at the time the item was issued. Duplicate of GL 79-058
GL 79-063	Upgraded Emergency Plans	C 01	GL 79-63 advised applicants for licenses of proposed rulemaking that NRC concurrence in State and local emergency plans would be a condition for issuing an operating license. TVA responded to GL 79-63 on January 3, 1980, and confirmed the intent to revise the Emergency Plan to address the NRC requirements.
GL 79-064	Suspension of All Operating Licenses (PWRs)	NA	Info
GL 79-065	Radiological Environmental Monitoring Program Requirements - Enclosing Branch Technical Position, Revision 1	NA	Info
GL 79-066	Additional Information Re 11/09/79 Letter on ECCS Calculations [GL 79-62]	NA	Info
GL 79-067	Estimates for Evacuation of Various Areas Around Nuclear Power Reactors	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 79-068	Audit of Small Break LOCA Guidelines	NA	Info
GL 79-069	Cladding Rupture, Swelling, and Coolant Blockage as a Result of a Reactor Accident	NA	Info
GL 79-070	Environmental Monitoring for Direct Radiation	NA	Info
GL 80-001	NUREG-0630, "Cladding, Swelling and Rupture - Models For LOCA Analysis"	NA	Info
GL 80-002	QA Requirements Regarding Diesel Generator Fuel Oil	C	TVA: FSAR 9.5.4.2
GL 80-003	BWR Control Rod Failures	NA	Boiling Water Reactor
GL 80-004	B 80-01, "Operability of ADS Valve Pneumatic Supply"	NA	Boiling Water Reactor
GL 80-005	B 79-01b, "Environmental Qualification of Class 1E Equipment"	NA	Info
GL 80-006	Issuance of NUREG-0313, Rev 1, "Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping"	NA	Boiling Water Reactor
GL 80-007	This GL was never issued.	NA	
GL 80-008	B 80-02. "Inadequate Quality Assurance for Nuclear Supplied Equipment"	NA	Boiling Water Reactor
GL 80-009	Low Level Radioactive Waste Disposal	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 80-010	Issuance of NUREG-0588, "Interim Staff Position On Equipment Qualifications of Safety-Related Electrical Equipment"	NA	Info
GL 80-011	B 80-03, "Loss of Charcoal From Standard Type II, 2 Inch, Tray Absorber Cells"	C 01	GL 80-11 transmitted Bulletin 80-03. TVA responded to B 80-03 on March 21, 1980. See B 80-03 for further information.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 80-012	B 80-04, "Analysis of a PWR Main Steam Line Break With Continued Feedwater Addition"	NA	Info
GL 80-013	Qualification of Safety Related Electrical Equipment	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 80-014	LWR Primary Coolant System Pressure Isolation Valves	S 02	TVA: FSAR 5.2.7.4 NRC: 1.14.2 of SSER 6 --- NRC reviewed in 1.14.2 of SSER6. Unit 2 Action: Incorporate guidance into Technical Specifications. --- --- REVISION 02 UPDATE: Developmental Revision B of the Unit 2 Technical Specifications (TS) was submitted on February 2, 2010. TS Surveillance Requirement 3.4.13.1 verifies RCS operational leakage by performance of an RCS water inventory balance.
GL 80-015	Request for Additional Management and Technical Resources Information	NA	Info
GL 80-016	B 79-01b, "Environmental Qualification of Class 1E Equipment"	NA	Info
GL 80-017	Modifications to BWR Control Rod Drive Systems	NA	Boiling Water Reactor
GL 80-018	Crystal River 3 Reactor Trip From Approximately 100% Full Power	NA	Applies only to Babcock and Wilcox designed plants
GL 80-019	Resolution of Enhanced Fission Gas Release Concern	NA	Info
GL 80-020	Actions Required From OL Applicants of NSSS Designs by W and CE Resulting From NRC B&O Task Force Review of TMI2 Accident	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 80-021	B 80-05, "Vacuum Condition Resulting in Damage to Chemical Volume Control System Holdup Tanks"	CI	Closed in IR 50-390/84-59 and 50-391/84-45. Unit 2 Action: Complete surveillance procedures for Unit 2.
GL 80-022	Transmittal of NUREG-0654, "Criteria For Preparation and Evaluation of Radiological Emergency Response Plan"	NA	Info
GL 80-023	Change of Submittal Date For Evaluation Time Estimates	NA	Info
GL 80-024	Transmittal of Information on NRC "Nuclear Data Link Specifications"	NA	Info
GL 80-025	B 80-06, "Engineering Safety Feature (ESF) Reset Controls"	NA	Info
GL 80-026	Qualifications of Reactor Operators	NA	Info
GL 80-027	B 80-07, "BWR Jet Pump Assembly Failure"	NA	Boiling Water Reactor
GL 80-028	B 80-08, "Examination of Containment Liner Penetration Welds"	C 01	GL 80-28 transmitted Bulletin 80-08. TVA responded to B 80-08 on July 8, 1980. See B 80-08 for further information.
GL 80-029	Modifications to Boiling Water Reactor Control Rod Drive Systems	NA	Boiling Water Reactor
GL 80-030	Clarification of The Term "Operable" As It Applies to Single Failure Criterion For Safety Systems Required by TS	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 80-031	B 80-09, "Hydramotor Actuator Deficiencies"	NA	Info
GL 80-032	Information Request on Category I Masonry Walls Employed by Plants Under CP and OL Review	C 01	GL 80-32 transmitted NRC questions on masonry walls. TVA provided the information requested by letters dated February 12, 1981, for reinforced walls and August 20, 1981, for nonreinforced walls. TVA provided a final response on January 22, 1982. See B 80-11 for further information.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 80-033	Actions Required From OL Applicants of B&W Designed NSSS Resulting From NRC B&O Task Force Review of TMI2 Accident	NA	Applies only to Babcock and Wilcox designed plants
GL 80-034	Clarification of NRC Requirements for Emergency Response Facilities at Each Site	NA	Info
GL 80-035	Effect of a DC Power Supply Failure on ECCS Performances	NA	Boiling Water Reactor
GL 80-036	B 80-10, "Contamination of Non-Radioactive System and Resulting Potential For Unmonitored, Uncontrolled Release to Environment"	NA	Info
GL 80-037	Five Additional TMI-2 Related Requirements to Operating Reactors	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 80-038	Summary of Certain Non-Power Reactor Physical Protection Requirements	NA	Does not apply to power reactor.
GL 80-039	B 80-11, "Masonry Wall Design"	NA	Info
GL 80-040	Transmittal of NUREG-0654, "Report of the B&O Task Force" and Appropriate NUREG-0626, "Generic Evaluation of FW Transient and Small Break LOCA"	NA	Info
GL 80-041	Summary of Meetings Held on April 22 &23, 1980 With Representatives of the Mark I Owners Group	NA	Info
GL 80-042	B 80-12, "Decay Heat Removal System Operability"	NA	Info
GL 80-043	B 80-13, "Cracking In Core Spray Spargers"	NA	Boiling Water Reactor
GL 80-044	Reorganization of Functions and Assignments Within ONRR/SSPB	NA	Info
GL 80-045	Fire Protection Rule	NA	Item was applicable only to units with operating license at the time the item was issued.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 80-046 and GL 80-047	Generic Technical Activity A-12, "Fracture Toughness and Additional Guidance on Potential for Low Fracture toughness and Laminar Tearing on PWR Steam Generator Coolant Pump Supports"	C	No response was required for this GL, and NUREG-0577 states that the lamellar tearing aspect of this issue was resolved by the NUREG. Further, the NUREG states that for plants under review, the fracture toughness issue was resolved.
GL 80-048	Revision to 5/19/80 Letter On Fire Protection [GL 80-45]	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 80-049	Nuclear Safeguards Problems	NA	Info
GL 80-050	Generic Activity A-10, "BWR Cracks"	NA	Boiling Water Reactor
GL 80-051	On-Site Storage of Low-Level Waste	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 80-052	Five Additional TMI-2 Related Requirements - Erata Sheets to 5/7/80 Letter [GL 80-37]	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 80-053	Decay Heat Removal Capability	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 80-054	B 80-14, "Degradation of Scram Discharge Volume Capability"	NA	Boiling Water Reactor
GL 80-055	B 80-15, "Possible Loss of Hotline With Loss of off-Site Power"	NA	Info
GL 80-056	Commission Memorandum and Order on Equipment Qualification	NA	Info
GL 80-057	Further Commission Guidance For Power Reactor Operating Licenses NUREG-0660 and NUREG-0694	NA	Info
GL 80-058	B 80-16, "Potential Misapplication of Rosemount Inc. Models 1151/1152 Pressure Transmitters With "A" Or "D" Output Codes"	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 80-059	Transmittal of Federal Register Notice RE Regional Meetings to Discuss Environmental Qualification of Electrical Equipment	NA	Info
GL 80-060	Request for Information Regarding Evacuation Times	NA	Info
GL 80-061	TMI-2 Lessons Learned	NA	Info
GL 80-062	TMI-2 Lessons Learned	NA	Boiling Water Reactor
GL 80-063	B 80-17, "Failure of Control Rods to Insert During a Scram at a BWR"	NA	Boiling Water Reactor
GL 80-064	Scram Discharge Volume Designs	NA	Boiling Water Reactor
GL 80-065	Request for Estimated Construction Completion and Fuel Load Schedules	NA	Info
GL 80-066	B 80-17, Supplement 1, "Failure of Control Rods to Insert During a Scram at a BWR"	NA	Boiling Water Reactor
GL 80-067	Scram Discharge Volume	NA	Boiling Water Reactor
GL 80-068	B 80-17, Supplement 2, "Failures Revealed by Testing Subsequent to Failure of Control Rods to Insert During a Scram at a BWR"	NA	Boiling Water Reactor
GL 80-069	B 80-18, "Maintenance of Adequate Minimum Flow Through Centrifugal Charging Pumps Following Secondary Side HELB"	NA	Info
GL 80-070	B 80-19, "Failures of Mercury-Wetted Matrix Relays in RPS of Operating Nuclear Power Plants Designed by GE"	NA	Info
GL 80-071	B 80-20, "Failures of Westinghouse Type W-2 Spring Return to Neutral Control Switches"	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 80-072	Interim Criteria For Shift Staffing	NA	Info
GL 80-073	"Functional Criteria For Emergency Response Facilities," NUREG-0696	NA	Info
GL 80-074	Notice of Forthcoming Meeting With Representatives of EPRI to Discuss Program For Resolution of USI A-12, "Fracture Toughness Issue"	NA	Info
GL 80-075	Lessons Learned Tech. Specs.	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 80-076	Notice of Forthcoming Meeting With GE to Discussed Proposed BWR Feedwater Nozzle Leakage Detection System	NA	Info
GL 80-077	Refueling Water Level – Technical Specifications Changes	S 02	Unit 2 Action: Address in Technical Specifications, as appropriate. REVISION 02 UPDATE: Developmental Revision B of the Unit 2 Technical Specifications (TS) was submitted on February 2, 2010. TS LCO 3.9.7 requires the refueling cavity water level to be maintained greater than or equal to 23 feet above the top of the reactor vessel flange during movement of irradiated fuel assemblies within containment.
GL 80-078	Mark I Containment Long-Term Program	NA	Boiling Water Reactor
GL 80-079	B 80-17, Supplement 3, "Failures Revealed by Testing Subsequent to Failure of Control Rods to Insert During a Scram At a BWR"	NA	Boiling Water Reactor
GL 80-080	Preliminary Clarification of TMI Action Plan Requirements	NA	Info
GL 80-081	Preliminary Clarification of TMI Action Plan Requirements - Addendum to 9/5/80 Letter [GL 80-80]	NA	Info
GL 80-082	B 79-01b, Supplement 2, "Environmental Qualification of Class 1E Equipment"	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 80-083	Environmental Qualification of Safety-Related Equipment	NA	Info
GL 80-084	BWR Scram System	NA	Boiling Water Reactor
GL 80-085	Implementation of Guidance From USI A-12, "Potential For LOW Fracture Toughness and Lamellar Tearing On Component Support"	NA	Info
GL 80-086	Notice of Meeting to Discuss Final Resolution of USI A-12	NA	Info
GL 80-087	Notice of Meeting to Discuss Status of EPRI-Proposed Resolution of the USI A-12 Fracture Toughness Issue	NA	Info
GL 80-088	Seismic Qualification of Auxiliary Feedwater Systems	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 80-089	B 79-01b, Supplement 3, "Environmental Qualification of Class 1E Equipment"	NA	Info
GL 80-090	NUREG-0737, TMI (Prior and future GLs, with the exception of certain discrete scopes, have been screened into NUREG list for those applicable to Watts Bar 2)	CI	See NUREG items in this list.
GL 80-091	ODYN Code Calculation	NA	Boiling Water Reactor
GL 80-092	B 80-21, "Valve Yokes Supplied by Malcolm Foundry Company, Inc."	C 01	GL 80-92 transmitted Bulletin 80-21. TVA responded to B 80-21 on May 6, 1981. See B 80-21 for further information.
GL 80-093	Emergency Preparedness	NA	Does not apply to power reactor.
GL 80-094	Emergency Plan	NA	Info
GL 80-095	Generic Technical Activity A-10, NUREG-0619, "BWR Feedwater Nozzle and Control Rod Drive Return Line Nozzle Cracking"	NA	Boiling Water Reactor

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 80-096	Fire Protection	NA	Addressed to specific plant(s).
GL 80-097	B 80-23, "Failures of Solenoid Valves Manufactured by Valcor Engineering Corporation"	NA	Info
GL 80-098	B 80-24, "Prevention of Damage Due to Water Leakage Inside Containment"	NA	Info
GL 80-099	Technical Specifications Revisions For Snubber Surveillance	NA	Info
GL 80-100	Appendix R to 10 CFR 50 Regarding Fire Protection - Federal Register Notice	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 80-101	Inservice Inspection Programs	NA	Addressed to specific plant(s).
GL 80-102	Commission Memorandum and Order of May 23, 1980 (Referencing B 79-01b, Supplement 2 - q.2 & 3 - Sept 30, 1980)	NA	Info
GL 80-103	Fire Protection - Revised Federal Register Notice	NA	Info
GL 80-104	Orders On Environmental Qualification of Safety Related Electrical Equipment	NA	Info
GL 80-105	Implementation of Guidance For USI A-12, "Potential For Low Fracture toughness and Lamellar Tearing On Component Supports"	NA	Info
GL 80-106	Report On ECCS Cladding Models, NUREG-0630	NA	Info
GL 80-107	BWR Scram Discharge System	NA	Boiling Water Reactor
GL 80-108	Emergency Planning	NA	Info
GL 80-109	Guidelines For SEP Soil Structure Interaction Reviews	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 80-110	Periodic Updating of FSARS	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 80-111	B 80-17, Supplement 4, "Failure of Control Rods to Insert During a Scram at a BWR"	NA	Boiling Water Reactor
GL 80-112	B 80-25, "Operating Problems With Target Rock Safety Relief Valves"	NA	Info
GL 80-113	Control of Heavy Loads	C	Superseded by GL 81-007.
GL 81-001	Qualification of Inspection, Examination, Testing and Audit Personnel	NA	Info
GL 81-002	Analysis, Conclusions and Recommendations Concerning Operator Licensing	NA	Info
GL 81-003	Implementation of NUREG-0313, "Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping"	NA	Boiling Water Reactor
GL 81-004	Emergency Procedures and Training for Station Blackout Events	C	Superseded by Station Blackout Rule.
GL 81-005	Information Regarding The Program For Environmental Qualification of Safety-Related Electrical Equipment	NA	Info
GL 81-006	Periodic Updating of Final Safety Analysis Reports (FSARS)	NA	Info
GL 81-007	Control of Heavy Loads	CI	"Movement of Heavy Loads Over Spent Fuel, Over Fuel in the Reactor, or Over Safety-Related Equipment" – NRC closure letter dated May 20, 1998. LICENSE CONDITION – Control of heavy loads (NUREG-0612) The staff concluded in SSER13 that the license condition was no longer necessary based on their review of TVA's response to NUREG-0612 guidelines for Phase I in TVA letter dated July 28, 1993. Unit 2 Action: Unit 2 Heavy Loads Program will be in compliance with NUREG-0612.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 81-008	ODYN Code	NA	Boiling Water Reactor
GL 81-009	BWR Scram Discharge System	NA	Boiling Water Reactor
GL 81-010	Post-TMI Requirements For The Emergency Operations Facility	NA	Info
GL 81-011	BWR Feedwater Nozzle and Control Rod Drive Return Line Nozzle Cracking (NUREG-0619)	NA	Boiling Water Reactor
GL 81-012	Fire Protection Rule	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 81-013	SER For GEXL Correlation For 8X8R Fuel Reload Applications For Appendix D Submittals of The GE topical Report	NA	Boiling Water Reactor
GL 81-014	Seismic Qualification of Auxiliary Feedwater Systems	CI	TVA: FSAR 10.4.9 --- Unit 2 Action: Additional Unit 2 implementing procedures or other activity is required for completion. [WAS "OL."]
GL 81-015	Environmental Qualification of Class 1E Electrical Equipment - Clarification of Staff's Handling of Proprietary Information	NA	Info
GL 81-016	NUREG-0737, Item I.C.1 SER on Abnormal Transient Operating Guidelines (ATOG)	NA	Applies only to Babcock and Wilcox designed plants
GL 81-017	Functional Criteria for Emergency Response Facilities	NA	Info
GL 81-018	BWR Scram Discharge System - Clarification of Diverse Instrumentation Requirements	NA	Boiling Water Reactor
GL 81-019	Thermal Shock to Reactor Pressure Vessels	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 81-020	Safety Concerns Associated With Pipe Breaks in the BWR Scram System	NA	Boiling Water Reactor

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 81-021	Natural Circulation Cooldown	CI	TVA responded December 3, 1981. Unit 2 Action: Issue operating procedures.
GL 81-022	Engineering Evaluation of the H. B. Robinson Reactor Coolant System Leak on 1/29/81	NA	Info
GL 81-023	INPO Plant Specific Evaluation Reports	NA	Info
GL 81-024	Multi-Plant Issue B-56, "Control Rods Fail to Fully Insert"	NA	Boiling Water Reactor
GL 81-025	Change in Implementing Schedule For Submission and Evaluation of Upgraded Emergency Plans	NA	Info
GL 81-026	Licensing Requirements for Pending Construction Permit and Manufacturing License Applications	NA	Applicants with pending Construction Permits
GL 81-027	Privacy and Proprietary Material in Emergency Plans	NA	Info
GL 81-028	Steam Generator Overfill	NA	Info
GL 81-029	Simulator Examinations	NA	Info
GL 81-030	Safety Concerns Associated With Pipe Breaks in the BWR Scram System	NA	Boiling Water Reactor
GL 81-031	This GL was never issued.	NA	
GL 81-032	NUREG-0737, Item II.K.3.44, "Evaluation of Anticipated Transients Combined With Single Failure"	NA	Boiling Water Reactor
GL 81-033	This GL was never issued.	NA	
GL 81-034	Safety Concerns Associated With Pipe Breaks in the BWR Scram System	NA	Boiling Water Reactor

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 81-035	Safety Concerns Associated With Pipe Breaks in the BWR Scram System	NA	Boiling Water Reactor
GL 81-036	Revised Schedule for Completion of TMI Action Plan Item II.D.1, "Relief and Safety Valve Testing"	NA	Info
GL 81-037	ODYN Code Reanalysis Requirements	NA	Boiling Water Reactor
GL 81-038	Storage of Low Level Radioactive Wastes at Power Reactor Sites	NA	Info
GL 81-039	NRC Volume Reduction Policy	NA	Info
GL 81-040	Qualifications of Reactor Operators	NA	Info
GL 82-001	New Applications Survey	NA	Info
GL 82-002	Commission Policy on Overtime	NA	Info
GL 82-003	High Burnup MAPLHGR Limits	NA	Boiling Water Reactor
GL 82-004	Use of INPO See-in Program	NA	Info
GL 82-005	Post-TMI Requirements	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 82-006	This GL was never issued.	NA	
GL 82-007	Transmittal of NUREG-0909 Relative to the Ginna Tube Rupture	NA	Boiling Water Reactor
GL 82-008	Transmittal of NUREG-0909 Relative to the Ginna Tube Rupture	NA	Info
GL 82-009	Environmental Qualification of Safety Related Electrical Equipment	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 82-010	Post-TMI Requirements	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 82-011	Transmittal of NUREG-0916 Relative to the Restart of R. E. Ginna Nuclear Power Plant	NA	Info
GL 82-012	Nuclear Power Plant Staff Working Hours	NA	Info
GL 82-013	Reactor Operator and Senior Reactor Operator Examinations	NA	Info
GL 82-014	Submittal of Documents to the NRC	NA	Info
GL 82-015	This GL was never issued.	NA	
GL 82-016	NUREG-0737 Technical Specifications	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 82-017	Inconsistency of Requirements Between 50.54(T) and 50.15	NA	Info
GL 82-018	Reactor Operator and Senior Reactor Operator Requalification Examinations	NA	Info
GL 82-019	Submittal of Copies of Documentation to NRC - Copy Requirements for Emergency Plans and Physical Security Plans	NA	Info
GL 82-020	Guidance for Implementing the Standard Review Plan Rule	NA	Info
GL 82-021	Fire Protection Audits	NA	Info
GL 82-022	Congressional Request for Information Concerning Steam Generator Tube Integrity	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 82-023	Inconsistency Between Requirements of 10CFR 73.40(d) and Standard Technical Specifications For Performing Audits of Safeguards Contingency Plans	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 82-024	Safety Relief Valve Quencher Loads: BWR MARK II and III Containments	NA	Boiling Water Reactor
GL 82-025	Integrated IAEA Exercise for Physical Inventory at LWRS	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 82-026	NUREG-0744, REV. 1, "Pressure Vessel Material Fracture Toughness"	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 82-027	Transmittal of NUREG-0763, "Guidelines For Confirmatory In-Plant Tests of Safety-Relief Valve Discharge for BWR Plants"	NA	Boiling Water Reactor
GL 82-028	Inadequate Core Cooling Instrumentation System	CO 06	LICENSE CONDITION - Detectors for Inadequate core cooling (II.F.2) In the original SER, the review of the ICC instrumentation was incomplete. The January 24, 1992, letter superseded the previous responses on this issue. TVA letter for Units 1 and 2 dated January 24, 1992, committed to install Westinghouse ICCM-86 and associated hardware. NRC completed the review for Units 1 and 2 in SSER10. For Unit 2 due to obsolescence of the ICCM-86 system, TVA intends to install the Westinghouse Common Q Post-Accident Monitoring System. Unit 2 Action: Install Westinghouse Common Q PAM system. --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. Subsumed as part of NRC staff review of Instrumentation and Controls submitted April 8, 2010."
GL 82-029	This GL was never issued.	NA	
GL 82-030	Filings Related to 10 CFR 50 Production and Utilization Facilities	NA	Info
GL 82-031	This GL was never issued.	NA	
GL 82-032	Draft Steam Generator Report (SAI)	NA	Item was applicable only to units with operating license at the time the item was issued.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 82-033	Supplement to NUREG-0737, "Requirements for Emergency Response Capability"	CI	"Safety Parameter Display System" (SPDS) / "Requirements for Emergency Response Capability" - NRC reviewed in SSER5, SSER6, and 18.2.2 of SSER15. Unit 2 Action: Install SPDS and have it operational prior to start-up after the first refueling outage.
GL 82-034	This GL was never issued.	NA	
GL 82-035	This GL was never issued.	NA	
GL 82-036	This GL was never issued.	NA	
GL 82-037	This GL was never issued.	NA	
GL 82-038	Meeting to Discuss Developments for Operator Licensing Examinations	NA	Info
GL 82-039	Problems With Submittals of Subsequent Information of CURT 73.21 For Licensing Reviews	NA	Info
GL 83-001	Operator Licensing Examination Site Visit	NA	Info
GL 83-002	NUREG-0737 Technical Specifications	NA	Boiling Water Reactor
GL 83-003	This GL was never issued.	NA	
GL 83-004	Regional Workshops Regarding Supplement 1 to NUREG-0737, "Requirements For Emergency Response Capability"	NA	Info
GL 83-005	Safety Evaluation of "Emergency Procedure Guidelines, Revision 2," June 1982	NA	Boiling Water Reactor
GL 83-006	Certificates and Revised Format For Reactor Operator and Senior Reactor Operator Licenses	NA	Info
GL 83-007	The Nuclear Waste Policy Act of 1982	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 83-008	Modification of Vacuum Breakers on Mark I Containments	NA	Boiling Water Reactor
GL 83-009	Review of Combustion Engineering Owners' Group Emergency Procedures Guideline Program	NA	Applies only to Combustion Engineering designed plants
GL 83-010a	Resolution of TMI Action Item II.K.3.5., "Automatic Trip of Reactor Coolant Pumps"	NA	Applies only to Combustion Engineering designed plants
GL 83-010b	Resolution of TMI Action Item II.K.3.5., "Automatic Trip of Reactor Coolant Pumps"	NA	Applies only to Combustion Engineering designed plants
GL 83-010c	Resolution of TMI Action Item II.K.3.5., "Automatic Trip of Reactor Coolant Pumps"	CI	TVA: letters dated January 5, 1984 and June 25, 1984 NRC: letter dated June 8, 1990. --- Unit 2 Action: Incorporate emergency response guidelines into applicable procedures. [WAS "NOTE 3."]
GL 83-010d	Resolution of TMI Action Item II.K.3.5., "Automatic Trip of Reactor Coolant Pumps"	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 83-010e	Resolution of TMI Action Item II.K.3.5., "Automatic Trip of Reactor Coolant Pumps"	NA	Applies only to Babcock and Wilcox designed plants
GL 83-010f	Resolution of TMI Action Item II.K.3.5., "Automatic Trip of Reactor Coolant Pumps"	NA	Applies only to Babcock and Wilcox designed plants
GL 83-011	Licensee Qualification for Performing Safety Analyses in Support of Licensing Actions	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 83-012	Issuance of NRC FORM 398 - Personal Qualifications Statement - Licensee	NA	Info
GL 83-013	Clarification of Surveillance Requirements for HEPA Filters and Charcoal Absorber Units In Standard Technical Specifications on ESF Cleanup Systems	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 83-014	Definition of "Key Maintenance Personnel," (Clarification of Generic Letter 82-12)	NA	Info
GL 83-015	Implementation of Regulatory Guide 1.150, "Ultrasonic Testing of Reactor Vessel Welds During Preservice & Inservice Examinations, Revision 1"	NA	Info
GL 83-016	Transmittal of NUREG-0977 Relative to the ATWS Events at Salem Generating Station, Unit No.1	NA	Info
GL 83-016a	Transmittal of NUREG-0977 Relative to the ATWS Events at Salem Generating Station, Unit No.1	NA	Info
GL 83-017	Integrity of Requalification Examinations for Renewal of Reactor Operator and Senior Reactor Operator Licenses	NA	Info
GL 83-018	NRC Staff Review of the BWR Owners' Group (BWROG) Control Room Survey Program	NA	Boiling Water Reactor
GL 83-019	New Procedures for Providing Public Notice Concerning Issuance of Amendments to Operating Licenses	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 83-020	Integrated Scheduling for Implementation of Plant Modifications	NA	Info
GL 83-021	Clarification of Access Control Procedures for Law Enforcement Visits	NA	Info
GL 83-022	Safety Evaluation of "Emergency Response Guidelines"	NA	Info
GL 83-023	Safety Evaluation of "Emergency Procedure Guidelines"	NA	Applies only to Combustion Engineering designed plants
GL 83-024	TMI Task Action Plan Item I.G.1, "Special Low Power Testing and Training," Recommendations for BWRs	NA	Boiling Water Reactor

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 83-025	This GL was never issued.	NA	
GL 83-026	Clarification Of Surveillance Requirements For Diesel Fuel Impurity Level Tests	NA	Info
GL 83-027	Surveillance Intervals in Standard Technical Specifications	NA	Info
GL 83-028	"Required Actions Based on Generic Implications of Salem ATWS Events: 1.2 – Post Trip Review Data and Information Capability	C	TVA: letters dated November 7, 1983 and December 4, 1987 NRC: IR 50-390, 391/86-04
GL 83-028	"Required Actions Based on Generic Implications of Salem ATWS Events: 2.1 – Equipment Classification and Vendor Interface (Reactor Trip System Components)	CI 06	TVA: letters dated November 7, 1983 and August 24, 1990 NRC: letters dated October 20, 1986 and June 18, 1990 ---------- Unit 2 Action: Ensure that required information on Critical Structures and Components is properly incorporated into procedures. [WAS "NOTE 3."] ---------- ---------- REVISION 06 UPDATE: Confirmed that required information on Critical Structures and Components is properly incorporated into procedures.
GL 83-028	"Required Actions Based on Generic Implications of Salem ATWS Events: 2.2 – Equipment Classification and Vendor Interface (All SR Components)"	CI	Unit 2 Action: Enter engineering component background data in INPO's Equipment Performance and Information Exchange System (EPIX) for Unit 2.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 83-028	"Required Actions Based on Generic Implications of Salem ATWS Events: 3.1 – Post-Maintenance Testing (Reactor Trip System Components)	S 02	TVA: letters dated November 7, 1983, January 17, 1986 and November 1, 1993 NRC: letters dated December 10, 1985, October 27, 1986, and July 2, 1990; IR 390, 391/86-04 --- Unit 2 Action: Test and maintenance procedures and Technical Specifications will include post-maintenance operability testing of safety-related components of the reactor trip system. --- --- REVISION 02 UPDATE: Developmental Revision A of the Unit 2 TS (including the TS Bases) was submitted on March 4, 2009. The Bases for TS Surveillance Requirement 3.0.1 states, in part, "Upon completion of maintenance, appropriate post maintenance testing is required to declare equipment OPERABLE. This includes ensuring applicable Surveillances are not failed and their most recent performance is in accordance with SR 3.0.2."
GL 83-028	"Required Actions Based on Generic Implications of Salem ATWS Events: 3.2 – Post-Maintenance Testing (All SR Components)	S 06	TVA: letters dated November 7, 1983, January 17, 1986 and November 1, 1993 NRC: letters dated December 10, 1985, October 27, 1986, and July 2, 1990; IR 390, 391/86-04 --- Unit 2 Action: Test and maintenance procedures and Technical Specifications will include post-maintenance operability testing of other (than reactor trip system) safety-related components. --- --- REVISION 02 UPDATE: Developmental Revision A of the Unit 2 TS (including the TS Bases) was submitted on March 4, 2009. The Bases for TS Surveillance Requirement 3.0.1 states, in part, "Upon completion of maintenance, appropriate post maintenance testing is required to declare equipment OPERABLE. This includes ensuring applicable Surveillances are not failed and their most recent performance is in accordance with SR 3.0.2." --- --- REVISION 06 UPDATE: Watts Bar's Preventative Maintenance Program is not unit specific; no further action is required for Unit 2.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 83-028	"Required Actions Based on Generic Implications of Salem ATWS Events: 4.1 – Reactor Trip System Reliability (Vendor Related Modifications)	CO 06	TVA: letter dated May 19, 1986 Unit 2 Action: Confirm vendor-recommended DS416 breaker modifications are implemented. REVISION 06 UPDATE: NRC Inspection Report 391/2011-602 closed GL 83-028, Item 4.1.
GL 83-028	"Required Actions Based on Generic Implications of Salem ATWS Events: 4.2 – Reactor Trip System Reliability (Preventive Maintenance and Surveillance Program for Reactor Trip Breakers)	S 02	TVA: letters dated November 7, 1983, February 10, 1986, and May 19, 1986 NRC: letters dated July 26, 1985 and June 18, 1992; SSER 16 Unit 2 Action: Ensure maintenance instruction procedure and Technical Specifications support reliable reactor trip breaker operation. REVISION 02 UPDATE: Developmental Revision B of the Unit 2 TS was submitted on February 2, 2010. Item 17. (Reactor Trip Breakers) of TS Table 3.3.1-1 states the requirement for the reactor trip breakers.
GL 83-028	"Required Actions Based on Generic Implications of Salem ATWS Events: 4.3 – Reactor Trip System Reliability (Automatic Actuation of Shunt Trip Attachment)	C	TVA: letters dated November 7, 1983, March 22, 1985 NRC: IR 50-390/86-04 and 50-391/86-04; letter dated June 18, 1990

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 83-028	"Required Actions Based on Generic Implications of Salem ATWS Events: 4.5 – Reactor Trip System Reliability (Automatic Actuation of Shunt Trip Attachment)	S 02	TVA: letters dated November 7, 1983 and July 26, 1985 NRC: letters dated June 28, 1990 and October 9, 1990; SSERs 5 and 16 --- Unit 2 Action: Address in Technical Specifications, as appropriate. --- --- REVISION 02 UPDATE: Developmental Revision B of the Unit 2 Technical Specifications (TS) was submitted on February 2, 2010. Item 18. (Reactor Trip Breaker Undervoltage and Shunt Trip Mechanisms) of TS Table 3.3.1-1 states the requirement for the shunt trip attachment.
GL 83-029	This GL was never issued.	NA	
GL 83-030	Deletion of Standard Technical Specifications Surveillance Requirement 4.8.1.1.2.d.6 For Diesel Generator Testing	NA	Info
GL 83-031	Safety Evaluation of "Abnormal Transient Operating Guidelines"	NA	Applies only to Babcock and Wilcox designed plants
GL 83-032	NRC Staff Recommendations Regarding Operator Action for Reactor Trip and ATWS	NA	Info
GL 83-033	NRC Positions on Certain Requirements of Appendix R to 10 CFR 50	NA	Info
GL 83-034	This GL was never issued.	NA	
GL 83-035	Clarification of TMI Action Plan Item II.K.3.31	NA	Info
GL 83-036	NUREG-0737 Technical Specifications	NA	Boiling Water Reactor
GL 83-037	NUREG-0737 Technical Specifications	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 83-038	NUREG-0965, "NRC Inventory of Dams"	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 83-039	Voluntary Survey of Licensed Operators	NA	Info
GL 83-040	Operator Licensing Examination	NA	Info
GL 83-041	Fast Cold Starts of Diesel Generators	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 83-042	Clarification to GL 81-07 Regarding Response to NUREG-0612, "Control of Heavy Loads at Nuclear Power Plants"	NA	Info
GL 83-043	Reporting Requirements of 10 CFR 50, Sections 50.72 and 50.73, and Standard Technical Specifications	NA	Info
GL 83-044	Availability of NUREG-1021, "Operator Licensing Examiner Standards"	NA	Info
GL 84-001	NRC Use Of The Terms "Important To Safety" and "Safety Related"	NA	Info
GL 84-002	Notice of Meeting Regarding Facility Staffing	NA	Info
GL 84-003	Availability of NUREG-0933, "A Prioritization of Generic Safety Issues"	NA	Info
GL 84-004	Safety Evaluation of Westinghouse Topical Reports Dealing with Elimination of Postulated Pipe Breaks in PWR Primary Main Loops	NA	Info
GL 84-005	Change to NUREG-1021, "Operator Licensing Examiner Standards"	NA	Info
GL 84-006	Operator and Senior Operator License Examination Criteria For Passing Grade	NA	Does not apply to power reactor.
GL 84-007	Procedural Guidance for Pipe Replacement at BWRs	NA	Boiling Water Reactor

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 84-008	Interim Procedures for NRC Management of Plant-Specific Backfitting	NA	Info
GL 84-009	Recombiner Capability Requirements of 10 CFR 50.44(c)(3)(ii)	NA	Boiling Water Reactor
GL 84-010	Administration of Operating Tests Prior to Initial Criticality	NA	Info
GL 84-011	Inspection of BWR Stainless Steel Piping	NA	Boiling Water Reactor
GL 84-012	Compliance With 10 CFR Part 61 and Implementation of Radiological Effluent Technical Specifications (RETs) and Attendant Process Control Program (PCP)	NA	Info
GL 84-013	Technical Specification for Snubbers	NA	Info
GL 84-014	Replacement and Requalification Training Program	NA	Info
GL 84-015	Proposed Staff Actions to Improve and Maintain Diesel Generator Reliability	NA	Info
GL 84-016	Adequacy of On-Shift Operating Experience for Near Term Operating License Applicants	NA	Info
GL 84-017	Annual Meeting to Discuss Recent Developments Regarding Operator Training, Qualifications, and Examinations	NA	Info
GL 84-018	Filing of Applications for Licenses and Amendments	NA	Does not apply to power reactor.
GL 84-019	Availability of Supplement 1 to NUREG-0933, "A Prioritization of Generic Safety Issues"	NA	Info
GL 84-020	Scheduling Guidance for Licensee Submittals of Reloads That Involve Unreviewed Safety Questions	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 84-021	Long Term Low Power Operation in Pressurized Water Reactors	NA	Info
GL 84-022	This GL was never issued.	NA	
GL 84-023	Reactor Vessel Water Level Instrumentation in BWRs	NA	Boiling Water Reactor
GL 84-024	Certification of Compliance to 10 CFR 50.49, Environmental Qualification of Electric Equipment Important To Safety For Nuclear Power Plants	CI	See Special Program for Environmental Qualification.
GL 85-001	Fire Protection Policy Steering Committee Report	NA	Only issued as draft
GL 85-002	Recommended Actions Stemming From NRC Integrated Program for the Resolution of Unresolved Safety Issues Regarding Steam Generator Tube Integrity	CI	TVA responded to the GL on June 17, 1985. Unit 2 Action: Perform SG inspection.
GL 85-003	Clarification of Equivalent Control Capacity for Standby Liquid Control Systems	NA	Boiling Water Reactor
GL 85-004	Operating Licensing Examinations	NA	Info
GL 85-005	Inadvertent Boron Dilution Events	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 85-006	Quality Assurance Guidance for ATWS Equipment That Is Not Safety-Related	NA	Info
GL 85-007	Implementation of Integrated Schedules for Plant Modifications	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 85-008	10 CFR 20.408 Termination Reports - Format	NA	Info
GL 85-009	Technical Specifications For Generic Letter 83-28, Item 4.3	NA	Info
GL 85-010	Technical Specification For Generic Letter 83-28, Items 4.3 and 4.4	NA	Applies only to Babcock and Wilcox designed plants

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 85-011	Completion of Phase II of "Control of Heavy Loads at Nuclear Power Plants," NUREG-0612	C	See GL 81-07.
GL 85-012	Implementation Of TMI Action Item II.K.3.5, "Automatic Trip Of Reactor Coolant Pumps"	CI	"Implementation of TMI Item II.K.3.5" – Reviewed in 15.5.4 of original 1982 SER; became License Condition 35. The staff determined that their review of Item II.K.3.5 did not have to be completed to support the full power license and considered this license condition resolved in SSER4. The item was further reviewed in Appendix EE of SSER16. Unit 2 Action: Implement modifications as required.
GL 85-013	Transmittal Of NUREG-1154 Regarding The Davis-Besse Loss Of Main And Auxiliary Feedwater Event	NA	Info
GL 85-014	Commercial Storage At Power Reactor Sites Of Low Level Radioactive Waste Not Generated By The Utility	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 85-015	Information On Deadlines For 10 CFR 50.49, "Environmental Qualification Of Electric Equipment Important To Safety At Nuclear Power Plants"	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 85-016	High Boron Concentrations	NA	Info
GL 85-017	Availability Of Supplements 2 and 3 To NUREG-0933, "A Prioritization Of Generic Safety Issues"	NA	Info
GL 85-018	Operator Licensing Examinations	NA	Info
GL 85-019	Reporting Requirements On Primary Coolant Iodine Spikes	NA	Info
GL 85-020	Resolution Of Generic Issue 69: High Pressure Injection/Make-up Nozzle Cracking In Babcock And Wilcox Plants	NA	Applies only to Babcock and Wilcox designed plants
GL 85-021	This GL was never issued.	NA	
GL 85-022	Potential For Loss Of Post-LOCA Recirculation Capability Due To Insulation Debris Blockage	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 86-001	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	NA	Boiling Water Reactor
GL 86-002	Technical Resolution of Generic Issue B-19 - Thermal Hydraulic Stability	NA	Boiling Water Reactor
GL 86-003	Applications For License Amendments	NA	Info
GL 86-004	Policy Statement On Engineering Expertise On Shift	C 01	TVA responded to GL 86-04 on May 29, 1986. TVA provides engineering expertise on shift in the form of a dedicated Shift Technical Advisor (STA) or an STA qualified Senior Reactor Operator.
GL 86-005	Implementation Of TMI Action Item II.K.3.5, "Automatic Trip Of Reactor Coolant Pumps"	NA	Applies only to Babcock and Wilcox designed plants
GL 86-006	Implementation Of TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps"	NA	Applies only to Combustion Engineering designed plants
GL 86-007	Transmittal of NUREG-1190 Regarding The San Onofre Unit 1 Loss of Power and Water Hammer Event	NA	Info
GL 86-008	Availability of Supplement 4 to NUREG-0933, "A Prioritization of Generic Safety Issues"	NA	Info
GL 86-009	Technical Resolution of Generic Issue B-59, (N-1) Loop Operation in BWRs and PWRs	S 02	N-1 Loop operation was addressed in original 1982 SER (4.4.7). Unit 2 Action: Confirm Technical Specifications prohibit (N-1) Loop Operation. REVISION 02 UPDATE: Developmental Revision B of the Unit 2 Technical Specifications (TS) was submitted on February 2, 2010. TS LCO 3.4.4 requires that four Reactor Coolant System loops be operable and in operation during Modes 1 and 2.
GL 86-010	Implementation of Fire Protection Requirements	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 86-010, S1	Fire Endurance Test Acceptance Criteria for Fire Barrier Systems Used to Separate Redundant Safe Shutdown Trains Within the Same Fire Area	NA	Info
GL 86-011	Distribution of Products Irradiated in Research	NA	Does not apply to power reactor.
GL 86-012	Criteria for Unique Purpose Exemption From Conversion From The Use of Heu Fuel	NA	Does not apply to power reactor.
GL 86-013	Potential Inconsistency Between Plant Safety Analyses and Technical Specifications	NA	Applies only to Babcock and Wilcox and Combustion Engineering designed plants
GL 86-014	Operator Licensing Examinations	NA	Info
GL 86-015	Information Relating To Compliance With 10 CFR 50.49, "Environmental Qualification of Electric Equipment Important To Safety For Nuclear Power Plants"	NA	Info
GL 86-016	Westinghouse ECCS Evaluation Models	NA	Info
GL 86-017	Availability of NUREG-1169, "Technical Findings Related to Generic Issue C-8, BWR MSIC Leakage And Treatment Methods"	NA	Boiling Water Reactor
GL 87-001	Public Availability Of The NRC Operator Licensing Examination Question Bank	NA	Info
GL 87-002 and GL 87-003	Verification of Seismic Adequacy of Mechanical and Electrical Equipment in Operating Reactors, USI A-46	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 87-004	Temporary Exemption From Provisions Of The FBI Criminal History Rule For Temporary Workers	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 87-005	Request for Additional Information on Assessment of License Measures to Mitigate and/or Identify Potential Degradation of Mark I Drywells	NA	Boiling Water Reactor

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 87-006	Periodic Verification of Leak Tight Integrity of Pressure Isolation Valves	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 87-007	Information Transmittal of Final Rulemaking For Revisions To Operator Licensing - 10 CFR 55 And Confirming Amendments	NA	Info
GL 87-008	Implementation of 10 CFR 73.55 Miscellaneous Amendments and Search Requirements	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 87-009	Sections 3.0 And 4.0 of Standard Tech Specs on Limiting Conditions For Operation And Surveillance Requirements	NA	Info
GL 87-010	Implementation of 10 CFR 73.57, Requirements For FBI Criminal History Checks	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 87-011	Relaxation in Arbitrary Intermediate Pipe Rupture Requirements	NA	Info
GL 87-012	Loss of Residual Heat Removal While The Reactor Coolant System is Partially Filled	C	This GL was superseded by GL 88-17.
GL 87-013	Integrity of Requalification Examinations At Non-Power Reactors	NA	Does not apply to power reactor.
GL 87-014	Operator Licensing Examinations	NA	Info
GL 87-015	Policy Statement On Deferred Plants	NA	Info
GL 87-016	Transmittal of NUREG-1262, "Answers To Questions On Implementation of 10 CFR 55 On Operators' Licenses"	NA	Info
GL 88-001	NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping	NA	Boiling Water Reactor
GL 88-002	Integrated Safety Assessment Program II	NA	Item was applicable only to units with operating license at the time the item was issued.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 88-003	Resolution of GSI 93, "Steam Binding of Auxiliary Feedwater Pumps"	CI	TVA: letter June 3, 1988. NRC letters dated February 17, 1988 and July 20, 1988 NRC: SSER 16 --- NRC accepted approach in letter dated July 20, 1988, and reviewed response in Appendix EE of SSER16. Unit 2 Action: Procedures and hardware will be in place to ensure recognition of indications of steam binding and maintenance of system operability until check valves are repaired and back leakage stopped.
GL 88-004	Distribution of Gems Irradiated in Research Reactors	NA	Does not apply to power reactor.
GL 88-005	Boric Acid Corrosion of Carbon Steel Reactor Pressure Boundary Components in PWR plants	CI 06	NRC acceptance letter dated August 8, 1990 for both units. Unit 2 Action: Implement program. --- --- REVISION 06 UPDATE: The program has been implemented on Unit 2.
GL 88-006	Removal of Organization Charts from Technical Specification Administrative Control Requirements	NA	Info
GL 88-007	Modified Enforcement Policy Relating to 10 CFR 50.49, "Environmental Qualification of Electrical Equipment Important to Safety for Nuclear Power Plants"	CI	See Special Program for Environmental Qualification.
GL 88-008	Mail Sent or Delivered to the Office of Nuclear Reactor Regulation	NA	Info
GL 88-009	Pilot Testing of Fundamentals Examination	NA	Boiling Water Reactor
GL 88-010	Purchase of GSA Approved Security Containers	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 88-011	NRC Position on Radiation Embrittlement of Reactor Vessel Material and its Impact on Plant Operations	S 02	NRC acceptance letter dated June 29, 1989, for both units. Unit 2 Action: Submit Pressure Temperature curves. REVISION 02 UPDATE: Developmental Revision B of the Unit 2 Technical Specifications (TS) was submitted on February 2, 2010. WCAP-17035-NP "Watts Bar Unit 2 Heatup and Cooldown Limit Curves for Normal Operation and PTLR Support Documentation" was submitted with the TS.
GL 88-012	Removal of Fire Protection Requirements from Technical Specification	NA	Info
GL 88-013	Operator Licensing Examinations	NA	Info
GL 88-014	Instrument Air Supply System Problems Affecting Safety-Related Equipment	CI 04	NRC letter dated July 26, 1990, closing the issue. Unit 2 Action: Complete Unit 2 implementation. REVISION 04 UPDATE: The compressed air system is a common system at Watts Bar; therefore, the requirements for this GL have been satisfied for Unit 2. Watts Bar revised the response in a letter dated July 14, 1995. NRC letter dated July 27, 1995, stated that their conclusion as stated on July 26,1990, had not changed and that their effort was complete.
GL 88-015	Electric Power Systems - Inadequate Control Over Design Process	NA	Info
GL 88-016	Removal of Cycle-Specific Parameter Limits from Technical Specifications	NA	Info
GL 88-017	Loss of Decay Heat Removal	CI	NRC acceptance letter dated March 8, 1995 (Unit 1). Unit 2 Action: Implement modifications to provide RCS temperature, RV level and RHR system performance.
GL 88-018	Plant Record Storage on Optical Disks	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 88-019	Use of Deadly Force by Licensee Guards to Prevent Theft of Special Nuclear Material	NA	Does not apply to power reactor.
GL 88-020	Individual Plant Examination for Severe Accident Vulnerabilities	S 06	Unit 2 Action: Complete evaluation for Unit 2. REVISION 02 UPDATE: The Probabilistic Risk Assessment Individual Plant Examination Summary Report was submitted on February 9, 2010. REVISION 04 UPDATE: The Individual Plant Examination of External Events Design Report was submitted on April 30, 2010. REVISION 06 UPDATE: The NRC issued Requests for Additional Information (RAIs) on November 12, 2010. TVA responded to the RAIs on December 17, 2010, and April 1, 2011.
GL 89-001	Implementation of Programmatic and Procedural Controls for Radiological Effluent Technical Specifications	NA	Info
GL 89-002	Actions to Improve the Detection of Counterfeit and Fraudulently Marketed Products	C 01	GL 89-02 did not require a response. WBN Unit 2 program for procurement and dedication of materials is based in part on and complies with the guidance of GL 89-02. The program is implemented through project procedures.
GL 89-003	Operator Licensing Examination Schedule	NA	Info
GL 89-004	Guidelines on Developing Acceptable Inservice Testing Programs	OV	NRC reviewed in 3.9.6 of SSER14 (Unit 1). Unit 2 Action: Submit an ASME Section XI Inservice Test Program for the first ten year interval six months before receiving an Operating License.
GL 89-005	Pilot Testing of the Fundamentals Examination	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 89-006	Task Action Plan Item I.D.2 – Safety Parameter Display System – 10 CFR 50.54(f)	CI	"Safety Parameter Display System" (SPDS) / "Requirements for Emergency Response Capability" - NRC reviewed in SSER5, SSER6, and 18.2.2 of SSER15. Unit 2 Action: Install SPDS and have it operational prior to start-up after the first refueling outage.
GL 89-007	Power Reactor Safeguards Contingency Planning for Surface Vehicle Bombs	C	TVA: letter dated October 31, 1989 NRC: memo dated June 26, 1990
GL 89-008	Erosion/Corrosion-Induced Pipe Wall Thinning	CI	Unit 1 Flow Accelerated Corrosion Program reviewed in IR 390/94-89 (February 1995). Unit 2 Actions: * Prepare procedure, and * perform baseline inspections.
GL 89-009	ASME Section III Component Replacements	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 89-010	Safety-Related Motor-Operated Valve Testing and Surveillance	CI	NRC accepted approach in September 14, 1990, letter and reviewed in Appendix EE of SSER16. Unit 2 Action: Implement pressure testing and surveillance program for safety-related MOVs, satisfying the intent of GL 89-10.
GL 89-010 or GL 96-005	Involves Main Steam Isolation Valves	NA	Boiling Water Reactor
GL 89-011	Resolution of Generic Issue 101, "Boiling Water Reactor Water Level Redundancy"	NA	Boiling Water Reactor
GL 89-012	Operator Licensing Examination	NA	Info
GL 89-013	Service Water System Problems Affecting Safety-Related Equipment	CI 06	NRC letters dated July 9, 1990 and June 13, 1997, accepting approach. Unit 2 Actions: 1) Implement initial performance testing of the heat exchangers; and 2) Establish eddy current baseline data for the Containment Spray heat exchangers. --- --- REVISION 06 UPDATE: NRC Inspection Report 391/2011-602 closed GL 89-013.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 89-014	Line-Item Improvements in Technical Specifications - Removal of 3.25 Limit on Extending Surveillance Intervals	NA	Info
GL 89-015	Emergency Response Data System	NA	Info
GL 89-016	Installation of a Hardened Wetwell Vent	NA	Boiling Water Reactor
GL 89-017	Planned Administrative Changes to the NRC Operator Licensing Written Examination Process	NA	Info
GL 89-018	Resolution of Unresolved Safety Issues A-17, "Systems Interactions in Nuclear Power Plants"	NA	Info
GL 89-019	Request for Actions Related to Resolution of Unresolved Safety Issue A-47, "Safety Implication of Control Systems in LWR Nuclear Power Plants" Pursuant to 10 CFR 50.54(f)	CI	TVA responded by letter dated March 22, 1990. NRC acceptance letter dated October 24, 1990, for both units. Unit 2 Action: Perform evaluation of common mode failures due to fire.
GL 89-020	Protected Area Long-Term Housekeeping	NA	Does not apply to power reactor.
GL 89-021	Request for Information Concerning Status of Implementation of Unresolved Safety Issue (USI) Requirements	S 06	TVA responded to GL 89-21 with the status of USIs for both units on November 29, 1989. NRC provided an assessment of WBN USI status on May 1, 1990. The NRC assessment included a list of incomplete USIs for WBN. USIs were initially reviewed for WBN in the SER Appendix C. USIs were subsequently reviewed in SSER 15 Appendix C (June 1995) and SSER 16 (September 1995). Unit 2 actions: * Provide a status of WBN Unit 2 USIs. * Complete implementation of USIs. --- --- REVISION 02 UPDATE: Status of USIs was provided by Enclosure 2 of TVA letter dated September 26, 2008. The applicable USIs are either closed, deleted, or captured in either the SER Framework or the Generic Communications Framework, or they are part of the CAPs and SPs. ---

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
			REVISION 06 UPDATE: Updated status of USIs was provided on January 25, 2011.
GL 89-022	Potential For Increased Roof Loads and Plant Area Flood Runoff Depth At Licensed Nuclear Power Plants Due To Recent Change In Probable Maximum Precipitation Criteria Developed by the National Weather Service	C	TVA: letter dated December 16, 1981 Answer to informal question provided in TVA letter dated December 16, 1981, and subsequently included in FSAR. GL did not require a response. No further action required.
GL 89-023	NRC Staff Responses to Questions Pertaining to Implementation of 10 CFR Part 26	NA	Info
GL 90-001	Request for Voluntary Participation in NRC Regulatory Impact Survey	NA	Info
GL 90-002	Alternative Requirements for Fuel Assemblies in the Design Features Section of Technical Specifications	NA	Info
GL 90-003	Relaxation of Staff Position in Generic Letter 83-28, Item 2.2 Part 2 "Vendor Interface for Safety-Related Components"	NA	Info
GL 90-004	Request for Information on the Status of Licensee Implementation of GSIs Resolved with Imposition of Requirements or CAs	C	TVA responded on June 23, 1990
GL 90-005	Guidance for Performing Temporary Non-Code Repair of ASME Code Class 1, 2, and 3 Piping	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 90-006	Resolution of Generic Issues 70, "PORV and Block Valve Reliability," and 94, "Additional LTOP Protection for PWRs"	S 02	NRC letter dated January 9, 1991, accepted TVA's response for both units. Unit 2 Actions: 1) Revise operating instruction and surveillance procedure; and 2) Incorporate testing requirements in the Technical Specifications. REVISION 02 UPDATE: Developmental Revision A of the Unit 2 Technical Specifications (TS) was submitted on March 04, 2009. TS Surveillance Requirement 3.4.11.2 specifies the required testing of each PORV.
GL 90-007	Operator Licensing National Examination Schedule	NA	Info
GL 90-008	Simulation Facility Exemptions	NA	Info
GL 90-009	Alternative Requirements for Snubber Visual Inspection Intervals and Corrective Actions	NA	Info
GL 91-001	Removal of the Schedule for the Withdrawal of Reactor Vessel Material Specimens from Technical Specifications	NA	Info
GL 91-002	Reporting Mishaps Involving LLW Forms Prepared for Disposal	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 91-003	Reporting of Safeguards Events	NA	Info
GL 91-004	Changes in Technical Specification Surveillance Intervals to Accommodate a 24-Month Fuel Cycle	NA	Info
GL 91-005	Licensee Commercial-Grade Procurement and Dedication Programs	NA	Info
GL 91-006	Resolution of Generic Issue A-30, "Adequacy of Safety-Related DC Power Supplies," Pursuant to 10 CFR 50.54(f)	NA	Item was applicable only to units with operating license at the time the item was issued.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 91-007	GI-23, "Reactor Coolant Pump Seal Failures" and Its Possible Effect on Station Blackout	NA	Info
GL 91-008	Removal of Component Lists from Technical Specifications	NA	Info
GL 91-009	Modification of Surveillance Interval for the Electrical Protective Assemblies in Power Supplies for the Reactor Protection System	NA	Boiling Water Reactor
GL 91-010	Explosives Searches at Protected Area Portals	NA	Does not apply to power reactor.
GL 91-011	Resolution of Generic Issues A-48, "LCOs for Class 1E Vital Instrument Buses", and 49, "Interlocks and LCOs for Class 1E Tie Breakers," Pursuant to 10 CFR 50.54	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 91-012	Operator Licensing National Examination Schedule	NA	Info
GL 91-013	Request for Information Related to Resolution of Generic Issue 130, "Essential Service Water System Failures @ Multi-Unit Sites"	NA	Addressed to specific (non-TVA) plants.
GL 91-014	Emergency Telecommunications	NA	Info
GL 91-015	Operating Experience Feedback Report, Solenoid-Operated Valve Problems at U.S. Reactors	NA	Info
GL 91-016	Licensed Operators' and Other Nuclear Facility Personnel Fitness for Duty	NA	Info
GL 91-017	Generic Safety Issue 29, "Bolting Degradation or Failure in Nuclear Power Plants"	NA	Info
GL 91-018	Information to Licensees Regarding Two NRC Inspection Manual Sections on Resolution of Degraded and Nonconforming Conditions and on Operability	NA	GL 91-18 has been superseded by RIS 2005-20.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 91-019	Information to Addressees Regarding New Telephone Numbers for NRC Offices Located in One White Flint North	NA	Info
GL 92-001	Reactor Vessel Structural Integrity	C	By letter dated May 11, 1994, for both units NRC confirmed TVA had provided the information requested in GL 92-01. NRC issued GL 92-01 revision 1, supplement 1 on May 19, 1995. By letter dated July 26, 1996, NRC closed GL 92-01, Revision 1, Supplement 1 for both Watts Bar units.
GL 92-002	Resolution of Generic Issue 79, "Unanalyzed Reactor Vessel (PWR) Thermal Stress During Natural Convection Cooldown"	NA	Info
GL 92-003	Compilation of the Current Licensing Basis: Request for Voluntary Participation in Pilot Program	NA	Info
GL 92-004	Resolution of the Issues Related to Reactor Vessel Water Level Instrumentation in BWRs Pursuant to 10 CFR 50.54(f)	NA	Boiling Water Reactor
GL 92-005	NRC Workshop on the Systematic Assessment of Licensee Performance (SALP) Program	NA	Info
GL 92-006	Operator Licensing National Examination Schedule	NA	Info
GL 92-007	Office of Nuclear Reactor Regulation Reorganization	NA	Info
GL 92-008	Thermo-Lag 330-1 Fire Barriers	OV	TVA configurations for Thermo-Lag 330-1 were reviewed in SSER18 and accepted in NRC letter dated January 6, 1998 (includes a supplemental SE). Unit 2 Actions: 1) Review Watts Bar design and installation requirements for Thermolag 330-1 fire barrier system and evaluate the Thermolag currently installed in Unit 2. 2) Remove and replace, as required, or prepare an approved deviation.
GL 92-009	Limited Participation by NRC in the IAEA International Nuclear Event Scale	NA	Info
GL 93-001	Emergency Response Data System Test Program	NA	Addressed to specific plant(s).

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 93-002	NRC Public Workshop on Commercial Grade Procurement and Dedication	NA	Info
GL 93-003	Verification of Plant Records	NA	Info
GL 93-004	Rod Control System Failure and Withdrawal of Rod Control Cluster Assemblies, 10 CFR 50.54(f)	CO 06	NRC letter dated December 9, 1994, accepted TVA commitments for both units. Unit 2 Action: Implement modifications and testing. REVISION 06 UPDATE: NRC Inspection Report 391/2011-604 closed GL 93-004.
GL 93-005	Line-Item Technical Specifications Improvements to Reduce Surveillance Requirements for Testing During Power Operation	NA	Info
GL 93-006	Research Results on Generic Safety Issue 106, "Piping and the Use of Highly Combustible Gases in Vital Areas"	NA	Info
GL 93-007	Modification of the Technical Specification Administrative Control Requirements for Emergency and Security Plans	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 93-008	Relocation of Technical Specification Tables of Instrument Response Time Limits	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 94-001	Removal of Accelerated Testing and Special Reporting Requirements for Emergency Diesel Generators	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 94-002	Long-Term Solutions and Upgrade of Interim Operating Recommendations for Thermal-Hydraulic Instabilities in BWRs	NA	Boiling Water Reactor
GL 94-003	IGSCC of Core Shrouds in BWRs	NA	Boiling Water Reactor
GL 94-004	Voluntary Reporting of Additional Occupational Radiation Exposure Data	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 95-001	NRC Staff Technical Position on Fire Protection for Fuel Cycle Facilities	NA	Does not apply to power reactor.
GL 95-002	Use of NUMARC/EPRI Report TR-102348, "Guideline on Licensing Digital Upgrades," in Determining the Acceptability of Performing Analog-to-Digital Replacements under 10 CFR 50.59	NA	Info
GL 95-003	Circumferential Cracking of Steam Generator Tubes	CI 06	NRC acceptance letter dated May 16, 1997 (Unit 1) – Initial response for Unit 2 on September 7, 2007. TVA responded to a request for additional information on December 17, 2007. Unit 2 Action: Perform baseline inspection. --- --- REVISION 02 UPDATE: Unit 2 Action: * Perform baseline inspection. * Evaluate or repair as necessary. --- On January 21, 2010, NRC issued the Safety Evaluation for the following Generic Letters: 1995-03, 1995-05, 1997-05, 1997-06, 2004-01, and 2006-01. --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML093631061)." --- 100% of the steam generator tubes have been inspected.
GL 95-004	Final Disposition of the Systematic Evaluation Program Lessons-Learned Issues	NA	Info

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 95-005	Voltage-Based Repair Criteria for Westinghouse Steam Generator Tubes Affected by Outside Diameter Stress Corrosion Cracking	C 06	No specific action or response required by the GL; TVA responded on September 7, 2007. REVISION 02 UPDATE: On January 21, 2010, NRC issued the Safety Evaluation for the following Generic Letters: 1995-03, 1995-05, 1997-05, 1997-06, 2004-01, and 2006-01. REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML093631061)."
GL 95-006	Changes in the Operator Licensing Program	NA	Info
GL 95-007	Pressure Locking and Thermal Binding of Safety-Related Power-Operated Gate Valves	CI 06	Unit 1 SER for GL 95-07 dated Sept 15, 1999 Unit 2 Actions: * Perform evaluation for pressure locking and thermal binding of safety related power-operated gate valves, and * take corrective actions for those valves identified as being susceptible. REVISION 03 UPDATE: April 1, 2010, letter committed to evaluate missing GL 89-10 motor-operated valves for susceptibility to pressure locking and thermal binding. REVISION 04 UPDATE: NRC letter dated July 29, 2010, provided RAIs on the GL. TVA letter dated July 30, 2010, answered the RAIs and provided the following commitments: * EDCRs 53292 and 53287 shall be implemented to eliminate the potential for pressure locking prior to startup. * Valves 2-FCV-63-25 and -26 will be evaluated for impact due to new parameters from the JOG Topical Report MPR 2524A prior to startup.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
			NRC issued the Safety Evaluation for GL 1995-007 on August 12, 2010. --- --- REVISION 06 UPDATE: TVA letter to NRC dated July 30, 2010, documented that none of the missing Watts Bar Unit 2 GL 89-10 valves are GL 95-07 valves. --- SSER22 contained the following for NRC Action: "Closed. NRC Letter dated August 12, 2010 (ADAMS Accession No. ML100190443)"
GL 95-008	10 CFR 50.54(p) Process for Changes to Security Plans Without Prior NRC Approval	NA	Info
GL 95-009	Monitoring and Training of Shippers and Carriers of Radioactive Materials	NA	Info
GL 95-010	Relocation of Selected Technical Specifications Requirements Related to Instrumentation	NA	Info
GL 96-001	Testing of Safety-Related Circuits	CI	TVA responded for both units on April 18, 1996. Unit 2 Action: Implement Recommendations.
GL 96-002	Reconsideration of Nuclear Power Plant Security Requirements Associated with an Internal Threat	NA	Info
GL 96-003	Relocation of the Pressure Temperature Limit Curves and Low Temperature Overpressure Protection System Limits	CI 06	No response required Unit 2 Actions: * Submit Pressure Temperature limits, and * similar to Unit 1, upon approval, incorporate into licensee-controlled document. --- --- REVISION 06 UPDATE: The Pressure and Temperature Limits Report (PTLR) was submitted via TVA to NRC letter dated February 2, 2010. The PTLR was incorporated in the system description for the Reactor Coolant System (WBN2-68-4001).

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 96-004	Boraflex Degradation in Spent Fuel Pool Storage Racks	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 96-005	Periodic Verification of Design-Basis Capability of Safety-Related Motor-Operated Valves	CI	SE of TVA response to GL 96-05 dated July 21, 1999. Unit 2 Actions: * Implement the Joint Owner's Group recommended GL 96-05 MOV PV program, as described in Topical Report No. OG-97-018, and * begin testing during the first refueling outage after startup.
GL 96-006	Assurance of Equipment Operability and Containment Integrity During Design-Basis Accident Conditions	C 06	NRC letter dated April 6, 1999, accepting TVA response for Unit 1. Unit 2 Action: Implement modification to provide containment penetration relief. --- --- REVISION 02 UPDATE: NRC issued the Safety Evaluation for Generic Letter 1996-006 on January 21, 2010. --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML100130227)." --- Modification to provide containment penetration relief was implemented. --- NRC Inspection Report 391/2011-603 closed GL 96-006.
GL 96-007	Interim Guidance on Transportation of Steam Generators	NA	Item was applicable only to units with operating license at the time the item was issued.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 97-001	Degradation of Control Rod Drive Mechanism Nozzle and Other Vessel Closure Head Penetrations	CI 06	NRC acceptance letter dated November 4, 1999 (Unit 1). Unit 2 Action: Provide a report to address the inspection program. REVISION 03 UPDATE: NRC issued the Safety Evaluation for Generic Letter 97-001 on June 30, 2010. REVISION 04 UPDATE: Corrected status from "OV" to "CI" due to NRC issuance of Safety Evaluation as noted in Revision 03 update. REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC Letter dated June 30, 2010 (ADAMS Accession No. ML100539515)"
GL 97-002	Revised Contents of the Monthly Operating Report	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 97-003	Annual Financial Update of Surety Requirements for Uranium Recovery Licensees	NA	Does not apply to power reactor.
GL 97-004	Assurance of Sufficient Net Positive Suction Head for Emergency Core Cooling and Containment Heat Removal Pumps	CI 06	NRC acceptance letter dated June 17, 1998 (Unit 1) – Initial response for Unit 2 on September 7, 2007. Unit 2 Actions: * Install new sump strainers, and * perform other modification-related activities identical to Unit 1. REVISION 02 UPDATE: NRC issued the Safety Evaluation for Generic Letter 1997-004 on February 18, 2010. REVISION 06 UPDATE: See the REVISION 06 UPDATE for GL 04-002 for new commitments.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
			--------- SSER22 contained the following for NRC Action: "Closed. NRC Letter dated February 18, 2010 (ADAMS Accession No. ML100200375)"
GL 97-005	Steam Generator Tube Inspection Techniques	CI 06	NRC acceptance letter dated September 22, 1998 (Unit 1) - Initial response for Unit 2 on September 7, 2007. Unit 2 Action: Employ the same approach used on the original Unit 1 SGs. TVA responded to a request for additional information on December 17, 2007. --- --- REVISION 02 UPDATE: On January 21, 2010, NRC issued the Safety Evaluation for the following Generic Letters: 1995-03, 1995-05, 1997-05, 1997-06, 2004-01, and 2006-01. --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML093631061)"
GL 97-006	Degradation of Steam Generator Internals	CI 06	NRC acceptance letter dated October 19, 1999 (Unit 1) – Initial response for Unit 2 on September 7, 2007. TVA responded to a request for additional information on December 17, 2007. Unit 2 Action: Perform SG inspections during each refueling outage. --- --- REVISION 02 UPDATE: On January 21, 2010, NRC issued the Safety Evaluation for the following Generic Letters: 1995-03, 1995-05, 1997-05, 1997-06, 2004-01, and 2006-01. --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML093631061)"

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 98-001	Year 2000 Readiness of Computer Systems at Nuclear Power Plants	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 98-002	Loss of Reactor Coolant Inventory and Associated Potential for Loss of Emergency Mitigation Functions While in a Shutdown Condition	CI 06	Initial response for Unit 2 on September 7, 2007. Unit 2 Actions: 1) Review the ECCS designs to ensure they do not contain design features which can render them susceptible to common-cause failures; and 2) document the results. --- --- REVISION 02 UPDATE: NRC issued the Safety Evaluation for Generic Letter 1998-002 on March 3, 2010. --- --- REVISION 03 UPDATE: NRC issued the Safety Evaluation for Generic Letter 98-002 on May 11, 2010. This letter noted that it superseded the SE issued by NRC on March 3, 2010. --- April 1, 2010, letter committed to ensure that the guidance added to the Unit 1 procedure as a result of the review of NRC GL 98-02 is incorporated into the Unit 2 procedures. Specifically, when decreasing power, valve HCV-74-34, Refueling Water Return (normally locked closed valve) has a hold order placed with specific release criteria before entry into Mode 4 and to remove the hold order before entry into Mode 3 when returning to power. --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC Letter dated May 11, 2010 (ADAMS Accession No. ML101200155)"
GL 98-003	NMSS Licensees' and Certificate Holders' Year 2000 Readiness Programs	NA	Does not apply to power reactor.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 98-004	Potential for Degradation of the ECCS and the Containment Spray System After a LOCA Because of Construction and Protective Coating Deficiencies and Foreign Material in Containment	CI 06	NRC closure letter dated November 24, 1999 (Unit 1). – Initial response for Unit 2 on September 7, 2007. Unit 2 Actions: * Install new sump strainers, and * perform other modification-related activities identical to Unit 1. --- --- REVISION 02 UPDATE: NRC issued the Safety Evaluation for Generic Letter 1998-004 on February 1, 2010. --- --- REVISION 06 UPDATE: See the REVISION 06 UPDATE for GL 04-002 for new commitments. --- SSER22 contained the following for NRC Action: "Closed. NRC Letter dated February 1, 2010 (ADAMS Accession No. ML100260594)"
GL 98-005	Boiling Water Reactor Licensees Use of the BWRVIP-05 Report to Request Relief from Augmented Examination Requirements on Reactor Pressure Vessel Circumferential Shell Welds	NA	Boiling Water Reactor
GL 99-001	Recent Nuclear Material Safety and Safeguards Decision on Bundling Exempt Quantities	NA	Info
GL 99-002	Laboratory Testing of Nuclear Grade Activated Charcoal	NA	Item was applicable only to units with operating license at the time the item was issued.
GL 03-001	Control Room Habitability	S 06	Initial response for Unit 2 on September 7, 2007 Unit 2 Action: Incorporate TSTF-448 into Technical Specifications. --- --- REVISION 02 UPDATE: NRC issued the Safety Evaluation for Generic Letter 2003-01 on February 1, 2010. ---

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
			Developmental Revision B of the Unit 2 Technical Specifications (TS) was submitted on February 2, 2010. TS Surveillance Requirement 3.7.10.4 requires performance of a Control Room Envelope (CRE) unfiltered air inleakage test in accordance with the CRE Habitability Program. TS 5.7.2.20 provides for the CRE Habitability Program. These portions of the Unit 2 TS were based on the Unit 1 TS which incorporated TSTF-448 per Amendment 70 (NRC approved A70 on 10/08/2008). --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC Letter dated February 1, 2010 (ADAMS Accession No. ML100270076)"
GL 04-001	Requirements for Steam Generator Tube Inspection	CI 06	NRC acceptance letter dated April 8, 2005 (Unit 1) - Initial response for Unit 2 on September 7, 2007. Unit 2 Action: Perform baseline inspection. --- --- REVISION 02 UPDATE: On January 21, 2010, NRC issued the Safety Evaluation for the following Generic Letters: 1995-03, 1995-05, 1997-05, 1997-06, 2004-01, and 2006-01. --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML093631061)" --- 100% of the steam generator tubes have been inspected.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 04-002	Potential Impact of Debris Blockage on Emergency Recirculation During Design Basis Accidents at PWRs	OV 06	NRC Audit Report dated February 7, 2007 (Unit 1) - Initial response for Unit 2 on September 7, 2007. Unit 2 Actions: * Install new sump strainers, and * perform other modification-related activities identical to Unit 1. --- --- REVISION 06 UPDATE: Additional TVA letters concerning GL 2004-02 were sent to the NRC on the following dates: - January 29, 2008, - May 19, 2008, - September 10, 2010, - March 4, 2011, and - April 29, 2011. The March 4, 2011, letter provided a response that superseded previous responses and commitments. It provided the following new commitments: - Unit 2 will install sump modifications per the requirements of Generic Letter (GL) 2004-02 prior to Unit 2 fuel load. - A confirmatory walkdown for loose debris will be performed on Unit 2 after containment work is completed and the containment has been cleaned. This walkdown will be completed prior to startup. - New throttle valves will be installed in the CVCS and SI injection lines to the RCS. The new valves will be opened sufficiently to preclude downstream blockage. - The current Unit 1 TVA protective coating program contains requirements for conducting periodic visual examinations of Coating Service Level I and Level II protective coatings. The Unit 2 program will be the same. - Procedural controls will be put in place at WBN Unit 2 to ensure that potential quantities of post-accident debris are maintained within the bounds of the analyses and design bases that support ECCS and CSS recirculation functions. - TVA will complete the WBN in-vessel downstream effects evaluation discussed in the supplemental response to Generic Letter 2004-02 following issuance of the final NRC Safety Evaluation Report (SER) for Topical Report No. WCAP-16793-NP, "Evaluation of Long-Term Cooling Considering Particulate, Fibrous, and Chemical Debris in the Recirculating Fluid." - The design basis of the modified emergency sump strainer has been incorporated into the plant's current licensing basis. The WBN Unit 2 FSAR will be amended to include this information. --- - Unit 1 and Unit 2 share a common protective coatings program. - Amendment 103 to the Unit 2 FSAR was submitted to the NRC on

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
			March 15, 2010. This amendment included the design basis of the modified emergency sump strainer.
GL 06-001	Steam Generator Tube Integrity and Associated Technical Specifications	S 06	Initial response for Unit 2 on September 7, 2007. Unit 2 Action: Incorporate TSTF-449 into Technical Specifications. --- --- REVISION 02 UPDATE: On January 21, 2010, NRC issued the Safety Evaluation for the following Generic Letters: 1995-03, 1995-05, 1997-05, 1997-06, 2004-01, and 2006-01. --- Developmental Revision B of the Unit 2 Technical Specifications (TS) was submitted on February 2, 2010. TS 5.7.2.12 is the Steam Generator (SG) Program. This program is implemented to ensure that SG tube integrity is maintained. Unit 2 TS 5.7.2.12 was based on Unit 1 TS 5.7.2.12. Unit 1 TS 5.7.2.1.12 was based on TSTF-449 (NRC approved Unit 1 TS A65 on 1/03/2006). --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML093631061) (See Appendix HH)" --- The applicable item from SER22, Appendix HH for this item is Open item 6, "Verify implementation of TSTF-449. (TVA letter dated September 7, 2007, ADAMS Accession No. ML072570676)." TVA to NRC letter dated April 6, 2011 provided the following response to Open Item 6: "Amendment 65 to the Unit 1 TS revised the existing steam generator tube surveillance program and was modeled after TSTF-449, Rev. 4. The NRC approved Amendment 65 via letter dated November 3, 2006, 'Watts Bar Nuclear Plant, Unit 1 - Issuance of Amendment Regarding Steam Generator Tube Integrity (TS-05-10) (TAC No. MC9271).' Revision 82 made the associated changes to the Unit 1 TS Bases. Developmental Revision A to the Unit 2 TS and TS Bases made the equivalent changes to the Unit 2 TS / TS Bases. Affected TS sections include the following: LEAKAGE definition in 1.1, LCO 3.4.13 (RCS Operational LEAKAGE), LCO 3.4.17 (SG Tube Integrity), 5.7.2.12 (Steam Generator (SG) Program), and 5.9.9 (Steam Generator Tube Inspection Report). Developmental Revision A of the Unit 2 TS was submitted to the NRC via letter dated March 4, 2009, 'Watts Bar Nuclear Plant (WBN) Unit 2 -

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
			Operating License Application Update,' (ADAMS Accession number ML090700378)."
GL 06-002	Grid Reliability and the Impact on Plant Risk and the Operability of Offsite Power	CI 06	Initial response for Unit 2 on September 7, 2007. Unit 2 Action: Complete the two unit baseline electrical calculations and implementing procedures. --- --- REVISION 02 UPDATE: NRC issued the Safety Evaluation for Generic Letter 2006-002 on January 20, 2010. --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC Letter dated January 21, 2010 (ADAMS Accession No. ML093631061) (See Appendix HH)" Note that the correct date and ADAMS Accession No. are January 20, 2010, and ML100080768, respectively.
GL 06-003	Potentially Nonconforming Hemyc and MT Fire Barrier Configurations	CI 06	TVA does not rely on Hemyc or MT materials to protect electrical and instrumentation cables or equipment that provide safe shutdown capability during a postulated fire. Unit 2 Action: Addressed in CAP/SP. The Fire Protection Corrective Action Program will ensure Unit 2 conforms with NRC requirements and applicable guidelines. --- --- REVISION 02 UPDATE: NRC issued the Safety Evaluation for Generic Letter 2006-003 on February 25, 2010. --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed. NRC Letter dated February 25, 2010 (ADAMS Accession No. ML100470398)"

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ITEM	TITLE	* REV	ADDITIONAL INFORMATION
GL 07-001	Inaccessible or Underground Power Cable Failures That Disable Accident Mitigation Systems or Cause Plant Transients	CI 06	Initial response for Unit 2 on September 7, 2007. Unit 2 Action: Complete testing of four additional cables. --- --- REVISION 02 UPDATE: NRC issued the Safety Evaluation for Generic Letter 2007-001 on January 26, 2010. --- --- REVISION 04 UPDATE: NRC Inspection Report 391/2010-603 closed GL 2007-001. --- --- REVISION 06 UPDATE: The four additional cables passed the testing. --- SSER22 contained the following for NRC Action: "Closed. NRC Letter dated January 26, 2010 (ADAMS Accession No. ML100120052)"
GL 08-001	Managing Gas Accumulation in Emergency Core Cooling, Decay Heat Removal, and Containment Spray Systems	O 06	Initial response for Unit 2 on October 1, 2008. --- --- REVISION 02 UPDATE: Unit 2 Actions: - TVA will provide a submittal within 45 days of completion of the engineering for the ECCS, RHR, and CSS systems. - WBN Unit 2 will complete the required modifications and provide a submittal consistent with the information requested in the GL 90 days prior to fuel load. --- --- REVISION 06 UPDATE: The submittal was provided in TVA to NRC letter dated March 11, 2011. This submittal satisfied the above Unit 2 actions and generated the following new commitments: - TVA will evaluate adopting the revised ISTS SR 3.5.2.3 (NUREG 1431) at WBN within 6 months of NRC approval of the Traveler. - Complete evaluation of CS pump 2A-A pipe chase horizontal suction

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
			piping for venting. Add a vent valve to this location or conduct periodic UT examinations if necessary. (90 days prior to fuel load.) - Add vent valves to selected locations in the ECCS and RHRS piping to enhance filling and venting. (90 days prior to fuel load.) - Complete walk down survey of ECCS and RHRS piping and evaluate the piping for latent voids that could exceed 5% of the pipe cross sectional area. (90 days prior to fuel load.) - Operating procedures are being revised to improve instructions for filling and venting portions of the ECCS discharge pipe. (90 days prior to fuel load.) - Complete Preoperational tests on ECCS and RHRS systems to confirm Unit 1 operating experience showing no gas intrusion/accumulation issues. (90 days prior to fuel load.) - Periodic venting procedures used to meet SR 3.5.2.3 are being revised to require that, for an extended gas release, a report is entered into the Corrective Action Program. (90 days prior to fuel load.)
NUREG-0737, I.A.1.1	Shift Technical Advisor	**NA**	Not applicable to WBN per SSER16.
NUREG-0737, I.A.1.2	Shift Supervisor Responsibilities	**NA**	Not applicable to WBN per SSER16.
NUREG-0737, I.A.1.3	Shift Manning	**C**	Closed in SSER16.
NUREG-0737, I.A.2.1	Immediate Upgrade of RO and SRO Training and Qualifications	**C**	Closed in SSER16.
NUREG-0737, I.A.2.3	Administration of Training Programs	**C**	Closed in SSER16.
NUREG-0737, I.A.3.1	Revise Scope and Criteria for Licensing Exams	**C**	Closed in SSER16.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
NUREG-0737, I.B.1.2	Independent Safety Engineering Group	OV 06	LICENSE CONDITION - Independent Safety Engineering Group (ISEG) (NUREG-0737, I.B.1.2) Resolved for Unit 1 only in SSER8. Unit 2 action: Implement the alternate ISEG that was approved for the rest of the TVA units including WBN Unit 1 by NRC on August 26, 1999. The function will be performed by the site engineering organizations. --- --- REVISION 06 UPDATE: By letter of March 2, 1999, TVA proposed to eliminate the ISEG function from the fleet-wide nuclear organization. NRC safety evaluation of August 26,1999 shows that the NRC accepted the elimination of the ISEG with alternate organizational responsibilities provided in TVA-NQA-PLN89A and TVA-NPOD89-A. By letter of August 26, 1999, TVA revised Topical Report TVA-NPOD89-A, Rev 8 to describe the alternate organizations responsible for the management and operation of TVA's nuclear projects that replaced the ISEG function. The developmental Unit 2 TS were modeled after the Unit 1 TS. There is no reference to the ISEG. The current revision of TVA-NQA-PLN89-A (24A1) was written to include Unit 2. The current revision of TVA-NPOD89-A (18) was written to include Unit 2.
NUREG-0737, I.C.1	Short Term Accident and Procedure Review	CI	NRC reviewed in Appendix EE of SSER16. Unit 2 Action: Implement upgraded Emergency Operating Procedures, including validation and training.
NUREG-0737, I.C.2	Shift and Relief Turnover Procedures	C	Closed in SSER16.
NUREG-0737, I.C.3	Shift Supervisor Responsibility	C	Closed in SSER16.
NUREG-0737, I.C.4	Control Room Access	C	Closed in SSER16.
NUREG-0737, I.C.5	Feedback of Operating Experience	C	Closed in SSER16.

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NUREG-0737, I.C.6	Verify Correct Performance of Operating Activities	C	Closed in SSER16.
NUREG-0737, I.C.7	NSSS Vendor Revision of Procedures	CI	IR 50-390/391 85-08 closed this item for Unit 1, and NRC also reviewed in Appendix EE of SSER16. Unit 2 Action: Revise power ascension and emergency procedures which were reviewed by Westinghouse.
NUREG-0737, I.C.8	Pilot Monitoring of Selected Emergency Procedures For Near Term Operating Licenses	CI	IR 50-390/391 85-08 closed this item for Unit 1, and NRC also reviewed in Appendix EE of SSER16. Unit 2 Action: Pilot monitor selected emergency procedures for NTOL.
NUREG-0737, I.D.1	Control Room Design Review	CI 06	NRC reviewed in SSER5, SSER6, SSER15, and Appendix EE of SSER16. Unit 2 Actions: * Complete the CRDR process. * Perform rewiring in accordance with ECN 5982. * Take advantage of the completed Human Engineering reviews to ensure appropriate configuration for Unit 2 control panels. See CRDR Special Program. --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed in SSER22, Section 18.2"
NUREG-0737, I.D.2	Plant-Safety-Parameter-Display Console	CI	NRC reviewed in SSER5, SSER6, and 18.2.2 of SSER15. Unit 2 Action: Install SPDS and have it operational prior to start-up after the first refueling outage.
NUREG-0737, I.G.1	Training During Low-Power Testing	C	Closed in SSER16.
NUREG-0737, II.B.1	Reactor Coolant Vent System	CI	LICENSE CONDITION - NUREG-0737, II.B.1, "Reactor Coolant System Vents" - In the original SER, the NRC found TVA's commitment to install reactor coolant vents acceptable pending verification. This was completed for Unit 1 only in SSER5 (IR 390/84-37). Unit 2 Action: Verify installation of reactor coolant vents.

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ITEM	TITLE	* REV	ADDITIONAL INFORMATION
NUREG-0737, II.B.2	Plant Shielding	CI	NRC reviewed in Appendix EE of SSER16. Unit 2 Action: Complete Design Review of EQ of equipment for spaces/systems which may be used in post accident operations.
NUREG-0737, II.B.3	Post-Accident Sampling	S 02	NRC reviewed in 9.3.2 of SSER16. TVA submitted a TS improvement to eliminate requirements for the Post Accident Sampling System using the Consolidated Line Item Improvement Process in a letter dated October 31, 2001. Unit 2 Actions: Unit 2 Technical Specifications will eliminate requirements for the Post-Accident Sampling System. --- --- REVISION 02 UPDATE: Developmental Revision A of the Unit 2 Technical Specifications (TS) was submitted on March 04, 2009. Rev. 0 of the Unit 1 TS contained 5.7.2.6, "Post Accident Sampling." Amendment 34 to the Unit 1 TS (approved by the NRC on January 14, 2002) deleted 5.7.2.6, "Post Accident Sampling." The markup for Unit 2 Developmental Revision A noted that Unit 2 had deleted 5.7.2.6, "Post Accident Sampling" also.
NUREG-0737, II.B.4	Training for Mitigating Core Damage	C	Closed in SSER16.
NUREG-0737, II.D.1	Relief and Safety Valve Test Requirements	CI	NRC reviewed in Technical Evaluation Report attached to Appendix EE of SSER15. Unit 2 Actions: 1) Testing of relief and safety valves; 2) Reanalysis of fluid transient loads for pressurizer relief and safety valve supports and any required modifications; 3) Modifications to pressurizer safety valves, PORVs, PORV block valves and associated piping; and 4) Change motor operated block valves.
NUREG-0737, II.D.3	Valve Position Indication	CI	The design was reviewed in the original 1982 SER and found acceptable pending confirmation of installation of the acoustic monitoring system. In SSER5 (IR 390/84-35), the staff closed the LICENSE CONDITION for Unit 1 only. Unit 2 Action: Verify installation of the acoustic monitoring system to PORV to indicate position.

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ITEM	TITLE	* REV	ADDITIONAL INFORMATION
NUREG-0737, II.E.1.1	Auxiliary Feedwater System Evaluation, Modifications	CI	Reviewed in Appendix EE of SSER16. Unit 2 Action: Perform Auxiliary Feedwater System analysis as it pertains to system failure and flow rate.
NUREG-0737, II.E.1.2	Auxiliary Feedwater System Initiation and Flow	CI	NRC: IR 50-390/84-20 and 50-391/84-16; letters dated March 29, 1985, and October 31, 1995; SSER 16 --- Unit 2 Actions: * Complete procedures, and * qualification testing.
NUREG-0737, II.E.3.1	Emergency Power For Pressurizer Heaters	CI	NRC: letters dated March 29, 1985, and October 31, 1995; SSER 16 --- Reviewed in original 1982 SER. Unit 2 Action: Implement procedures and testing.
NUREG-0737, II.E.4.1	Dedicated Hydrogen Penetrations	C	NRC: IR 50-390/83-27 and 50-391/83-19; SER (NUREG-0847)
NUREG-0737, II.E.4.2	Containment Isolation Dependability	S 02	TVA: letters dated October 29, 1981, and February 25, 1985 NRC: letters dated March 29, 1985, July 12, 1990 and October 31, 1995; SSER 16. --- OUTSTANDING ISSUE for NRC to complete review of information provided by TVA to address Containment Purging During Normal Plant Operation LICENSE CONDITION - Containment isolation dependability In the original 1982 SER, NRC concluded that WBN met all the requirements of NUREG-0737, item II.E.4.2 except subsection (6) concerning containment purging during normal operation. In SSER3, the outstanding issue was closed and the LICENSE CONDITION was left open. NRC completed the review and issued a Technical Evaluation Report for both units on July 12, 1990. NRC concluded that the isolation valves can close against the buildup of pressure in the event of a design basis accident if the lower containment isolation valves are physically blocked to an opening angle of 50 degrees or less. (SSER5) Unit 2 Action: Reflect valve opening restriction in the Technical Specifications.

* = See last page for status code definition.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
			REVISION 02 UPDATE: Developmental Revision B of the Unit 2 Technical Specifications (TS) was submitted on February 2, 2010. TS Surveillance Requirement 3.6.3.7 requires verification that the valves are "blocked to restrict the valve from opening > 50 degrees."
NUREG-0737, II.F.1.2.a.	Accident-Monitoring Instrumentation - Noble Gas	CI	Reviewed in SSER9. Unit 2 Actions: Install Noble gas, Iodine / particulate sampling, and Containment High Range Monitors. --- Unit 2 Action: Install Noble gas monitor for Unit 2.
NUREG-0737, II.F.1.2.b.	Accident-Monitoring Instrumentation - Iodine/Particulate Sampling	CI	Reviewed in SSER9. Unit 2 Actions: Install Noble gas, Iodine / particulate sampling, and Containment High Range Monitors. --- Unit 2 Action: Install Iodine / particulate sampling monitor for Unit 2.
NUREG-0737, II.F.1.2.c.	Accident-Monitoring Instrumentation - Containment High Range Monitoring	CI	Reviewed in SSER9. Unit 2 Actions: Install Noble gas, Iodine / particulate sampling, and Containment High Range Monitors. --- Unit 2 Action: Install high range in-containment monitor for Unit 2.
NUREG-0737, II.F.1.2.d.	Accident-Monitoring Instrumentation - Containment Pressure	CO 06	Reviewed in SSER9. Unit 2 Action: Verify installation of containment pressure indication. --- --- REVISION 06 UPDATE: NRC Inspection Report 391/2011-604 closed NUREG-0737, II.F.1.2.d.
NUREG-0737, II.F.1.2.e.	Accident-Monitoring Instrumentation - Containment Water Level	CI	Reviewed in SSER9. Unit 2 Action: Verify installation of containment water level monitors.

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ITEM	TITLE	* REV	ADDITIONAL INFORMATION
NUREG-0737, II.F.1.2.f.	Accident-Monitoring Instrumentation - Containment Hydrogen	CO 06	Reviewed in SSER9. Unit 2 Action: Verify installation of containment hydrogen accident monitoring instrumentation. REVISION 06 UPDATE: NRC Inspection Report 391/2011-604 closed NUREG-0737, II.F.1.2.F.
NUREG-0737, II.F.2	Instrumentation For Detection of Inadequate Core-Cooling	O	LICENSE CONDITION - Detectors for Inadequate core cooling (II.F.2) In the original SER, the review of the ICC instrumentation was incomplete. The January 24, 1992, letter superseded the previous responses on this issue. TVA letter for Units 1 and 2 dated January 24, 1992, committed to install Westinghouse ICCM-86 and associated hardware. NRC completed the review for Units 1 and 2 in SSER10. For Unit 2 due to obsolescence of the ICCM-86 system, TVA intends to install the Westinghouse Common Q Post-Accident Monitoring System. Unit 2 Action: Install Westinghouse Common Q PAM system.
NUREG-0737, II.G.1	Power Supplies For Pressurizer Relief Valves, Block Valves and Level Indicators	CI 06	Reviewed in original 1982 SER and 8.3.3 of SSER7. Unit 2 Action: Implement modifications such that PORVS and associated Block Valves are powered from same train but different buses. REVISION 06 UPDATE: Modifications were implemented such that PORVS and associated Block Valves are powered from same train but different buses.
NUREG-0737, II.K.1.5	Review ESF Valves	C	NRC: letter dated March 29, 1985; SSER 16
NUREG-0737, II.K.1.10	Operability Status	CI	Unit 2 Action: Confirm multi-unit operation will have no impact on administrative procedures with respect to operability status.
NUREG-0737, II.K.1.17	Trip Per Low-Level B/S	C	NRC: letter dated March 29, 1985; SSER 16

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ITEM	TITLE	* REV	ADDITIONAL INFORMATION
NUREG-0737, II.K.2.13	Effect of High Pressure Injection for Small Break LOCA With No Auxiliary Feedwater	C	LICENSE CONDITION – Effect of high pressure injection for small break LOCA with no auxiliary feedwater (NUREG-0737, II.K.2.13) In SSER4, the staff concluded that there was reasonable assurance that vessel integrity would be maintained for small breaks with an extended loss of all feedwater and that the USI A-49, "Pressurized Thermal Shock," review did not have to be completed to support the full-power license. They considered this condition resolved.
NUREG-0737, II.K.2.17	Voiding in the Reactor Coolant System	C	LICENSE CONDITION – Voiding in the reactor coolant system (NUREG-0737, II.K.2.17) The staff reviewed the generic resolution of this license condition in SSER4 and approved the study in question, thereby resolving this license condition.
NUREG-0737, II.K.3.1	Auto PORV Isolation	C	Reviewed in SSER5 and resolved based on NRC conclusion that there is no need for an automatic PORV isolation system (NRC letter dated June 29, 1990).
NUREG-0737, II.K.3.2	Report on PORV Failures	C	Reviewed in SSER5 and resolved based on NRC conclusion that there is no need for an automatic PORV isolation system (NRC letter dated June 29, 1990).
NUREG-0737, II.K.3.3	Reporting SV/RV Failures/Challenges	C 06	(Action from GL 82-16) – NRC reviewed in Appendix EE of SSER16. Unit 2 Action: Include, as necessary, in Technical Specifications submittal. --- --- REVISION 02 UPDATE: Developmental Revision A of the Unit 2 Technical Specifications (TS) was submitted on March 04, 2009. Rev. 0 of the Unit 1 TS contained 5.9.4 (Monthly Operating Reports) which implemented the above commitment for Unit 1. Amendment 57 to the Unit 1 TS (approved by the NRC on March 21, 2005) deleted this section of the TS. The markup for Unit 2 Developmental Revision A noted that Unit 2 will apply this change, and the Unit 2 TS will contain no requirement for Monthly Operating Reports. --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed in SSER22, Section 13.5.3."

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ITEM	TITLE	* REV	ADDITIONAL INFORMATION
NUREG-0737, II.K.3.5	Auto Trip of RCPS	CI	Reviewed in 15.5.4 of original 1982 SER; became License Condition 35. The staff determined that their review of Item II.K.3.5 did not have to be completed to support the full power license and considered this license condition resolved in SSER4. The item was further reviewed in Appendix EE of SSER16. Unit 2 Action: Implement modifications as required.
NUREG-0737, II.K.3.9	PID Controller	CI 06	Reviewed in original 1982 SER. Unit 2 Action: Set the derivative time constant to zero. REVISION 06 UPDATE: The derivative time constant was set to zero.
NUREG-0737, II.K.3.10	Anticipatory Trip at High Power	S 02	NRC: letter dated October 31, 1995; SSER 16 Unit 2 Action: Unit 2 Technical Specifications and surveillance procedures will address this issue. REVISION 02 UPDATE: Developmental Revision A of the Unit 2 Technical Specifications (TS) was submitted on March 04, 2009. Items 14.a. (Turbine Trip - Low Fluid Oil Pressure) and 14.b. (Turbine Trip - Turbine Stop Valve Closure) of TS Table 3.3.1-1 are the trips of interest. The table and the Bases for these items state that below the P-9 setpoint, these trips do not actuate a reactor trip. Per item 16.d. (Power Range Neutron Flux, P-9) of TS Table 3.3.1-1, the Nominal Trip Setpoint for P-9 is "50% RTP" and the Allowable Value is "< 52.4% RTP."
NUREG-0737, II.K.3.12	Confirm Existence of Anticipatory Reactor Trip Upon Turbine Trip	C	Closed in SSER16.
NUREG-0737, II.K.3.17	Report On Outage of Emergency Core Cooling System	C	LICENSE CONDITION – Report on outage of emergency core cooling system (NUREG-0737, II.K.3.17) In the original 1982 SER, the NRC accepted TVA's commitment to develop and implement a plan to collect emergency core cooling system outage information. In SSER3, the staff accepted a revised commitment from an October 28, 1983, letter to participate in the nuclear power reliability data system and comply with the requirements of 10 CFR 50.73.

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ITEM	TITLE	* REV	ADDITIONAL INFORMATION
NUREG-0737, II.K.3.25	Power On Pump Seals	C 06	NRC reviewed and closed in IR 390/84-35 based on Diesel Generator (DG) power to pump sealing cooling system. Unit 2 Action: Ensure DG power is provided to pump sealing cooling system. --- --- REVISION 06 UPDATE: It was confirmed that DG power is provided to pump sealing cooling system. --- NRC Inspection Report 391/2010-605 closed NUREG-0737, II.K.3.25.
NUREG-0737, II.K.3.30	Small Break LOCA Methods	C 06	TVA: letter dated October 29, 1981 NRC: letters dated March 29, 1985, and July 24, 1986; SSER 16 --- The staff determined in SSER4 that their review of Items II.K.3.30 and II.K.3.31 did not have to be completed to support the full-power license and considered this LICENSE CONDITION resolved in SSER4. In SSER5, the staff further reviewed responses to these items, and concluded that the Units 1 and 2 FSAR methods and analysis met the requirements of II.K.3.30 and II.K.3.31. This item was further reviewed in Appendix EE of SSER16. Unit 2 Action: Complete analysis for Unit 2. --- --- REVISION 06 UPDATE: The analysis has been completed for Unit 2. --- NRC Inspection Report 391/2011-603 closed NUREG-0737, II.K.3.30.
NUREG-0737, II.K.3.31	Plant Specific Analysis	C 06	The staff determined in SSER4 that their review of Items II.K.3.30 and II.K.3.31 did not have to be completed to support the full-power license and considered this LICENSE CONDITION resolved in SSER4. In SSER5, the staff further reviewed responses to these items, and concluded that the Units 1 and 2 FSAR methods and analysis met the requirements of II.K.3.30 and II.K.3.31. This item was further reviewed in Appendix EE of SSER16. Unit 2 Action: Complete analysis for Unit 2. --- --- REVISION 06 UPDATE:

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ITEM	TITLE	* REV	ADDITIONAL INFORMATION
			The analysis has been completed for Unit 2. --- NRC Inspection Report 391/2011-603 closed NUREG-0737, II.K.3.31.
NUREG-0737, III.A.1.1	Emergency Preparedness, Short Term	C	LICENSE CONDITION – Emergency Preparedness (NUREG-0737, III.A.1, III.A.2) The NRC review of Emergency Preparedness in SSER13 superseded the review in the original 1982 SER. In SSER13, the staff concluded that the WBN Radiological Emergency Plan (REP) provided an adequate planning basis for an acceptable state of onsite emergency preparedness, and the LICENSE CONDITION was deleted. The NRC completed the review of the REP in SSER20.
NUREG-0737, III.A.1.2	Upgrade Emergency Support Facilities	C	LICENSE CONDITION – Emergency Preparedness (NUREG-0737, III.A.1, III.A.2) The NRC review of Emergency Preparedness in SSER13 superseded the review in the original 1982 SER. In SSER13, the staff concluded that the WBN Radiological Emergency Plan (REP) provided an adequate planning basis for an acceptable state of onsite emergency preparedness, and the LICENSE CONDITION was deleted. The NRC completed the review of the REP in SSER20.
NUREG-0737, III.A.2	Emergency Preparedness	C	LICENSE CONDITION – Emergency Preparedness (NUREG-0737, III.A.1, III.A.2) The NRC review of Emergency Preparedness in SSER13 superseded the review in the original 1982 SER. In SSER13, the staff concluded that the WBN Radiological Emergency Plan (REP) provided an adequate planning basis for an acceptable state of onsite emergency preparedness, and the LICENSE CONDITION was deleted. The NRC completed the review of the REP in SSER20.
NUREG-0737, III.D.1.1	Primary Coolant Outside Containment	S 02	Resolved for Unit 1 only in SSER10; reviewed in Appendix EE of SSER16. Unit 2 Actions: Include the waste gas disposal system in the leakage reduction program and incorporate in Unit 2 Technical Specifications. --- --- REVISION 02 UPDATE: Developmental Revision B of the Unit 2 Technical Specifications (TS) was submitted on February 2, 2010. TS 5.7.2.4 is the Primary Coolant Sources Outside Containment program. This program provides controls to minimize leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to levels as low as practicable. This program includes the "Waste Gas" system.

ITEM	TITLE	* REV	ADDITIONAL INFORMATION
NUREG-0737, III.D.3.3	In-Plant Iodine Radiation Monitoring	CI	NRC reviewed in Appendix EE of SSER16. Unit 2 Action: Complete modifications for Unit 2.
NUREG-0737, III.D.3.4	Control-Room Habitability	CI 06	TVA: letter dated October 29, 1981 NRC: SSER 16 --- NRC reviewed in SER and in Appendix EE of SSER16. Unit 2 Action: Complete with CRDR completion. --- --- REVISION 06 UPDATE: SSER22 contained the following for NRC Action: "Closed in SSER22, Section 6.4"

STATUS CODE DEFINITIONS

C: CLOSED: Previous staff review of NUREG-0847 and/or supplements has closed the item either for both units at WBN or explicitly for WBN Unit 2.

CI: CLOSED/IMPLEMENTATION: Staff has approved either for both units at WBN or explicitly for WBN Unit 2; there is no change to the approved design; and implementation is recommended through Regional Inspection.

CO: CLOSED - OPEN: Staff has approved closure of the item; however, TVA actions remain to be completed.

CT: CLOSED/TECHNICAL SPECIFICATIONS: Item has been approved either for both units at WBN or explicitly for WBN Unit 2; however, a change to the original approval requires submittal of the Technical Specifications and staff review.

NA: NOT APPLICABLE: Justification as to why a section / subsection is not applicable is provided in the ADDITIONAL INFORMATION column.

O: OPEN: No action or documentation is provided that shows the staff has reviewed the item for WBN Unit 2.

OT: OPEN/TECHNICAL SPECIFICATIONS: No action or documentation is provided that shows the staff has reviewed the item for WBN Unit 2, and the resolution is through submittal of a Technical Specification.

OV: OPEN/VALIDATION: The proposed approach has been approved for Watts Bar Unit 1; the same approach is proposed for use on WBN Unit 2 without change.

S: SUBMITTED: Information has been submitted, and is under review by NRC staff.