

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

August 15, 2011

Mr. David A. Heacock President and Chief Nuclear Officer Virginia Electric and Power Company Innsbrook Technical Center 5000 Dominion Boulevard Glen Allen, VA 23060-6711

SUBJECT: SURRY POWER STATION, UNIT NOS. 1 AND 2, CORRECTION TO AMENDMENTS REGARDING ADMINISTRATIVE CHANGES TO TECHNICAL SPECIFICATIONS 3.12 AND 6.2 (TAC NOS. ME4349 AND ME4350)

Dear Mr. Heacock:

On October 19, 2010, the U.S. Nuclear Regulatory Commission (NRC) issued Amendment No. 270 to Renewed Facility Operating License No. DPR-32 and Amendment No. 269 to Renewed Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2 (SPS), respectively. The amendments made changes to Technical Specification (TS) pages 3.12-12 and 6.2-1. On July 28, 2011, NRC issued Amendment No. 275 for Unit 1 and 275 for Unit 2, making changes to other parts of TS pages 3.12-12 and 6.2-1 than the parts addressed in the October 19, 2010, amendment. When the July 28, 2011, amendments were issued, they did not include the changes that had been made in the October 19, 2010, amendments, therefore TS pages 3.12-12 and 6.2-1 are in error. The NRC staff has determined that this error was inadvertently introduced, and was not the subject of the July 28, 2011, amendment or the associated notice to the public. Thus, according to the NRC's policy established by SECY-96-238 (Agencywide Documents Access and Management System, Accession No. 9611250030), this error can be corrected by a letter.

Enclosed please find correct replacement TS pages 3.12-12 and 6.2-1. This correction does not change any of the conclusions in the safety evaluation associated with the amendments.

D. Heacock

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If you have any questions, please feel free to contact me at 301-415-1438.

Sincerely,

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Karen Cotton, Project Manager Plant Licensing Branch II-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket Nos. 50-280 and 50-281

Enclosure: Corrected TS pages TS 3.12-12 and TS 6.2-1

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- 3. If more than one rod position indicator per group is inoperable, place the control rods under manual control immediately, monitor and record RCS T_{avg} once per hour, verify the position of the control rod assemblies indirectly using the movable incore detectors at least once per 8 hours, and restore inoperable position indicators to OPERABLE status such that a maximum of one position indicator per group is inoperable within 24 hours.
- 4. If one or more rods with inoperable position indicators have been moved in excess of 24 steps in one direction since the last determination of the rod's position, verify the position of the control rod assemblies indirectly using the movable incore detectors within 4 hours or reduce power to less than 50% of RATED POWER within 8 hours.
- 5. If one group step demand counter per bank for one or more banks is inoperable, verify that all rod position indicators for the affected bank(s) are OPERABLE once per 8 hours and verify that the most withdrawn rod and the least withdrawn rod of the affected bank(s) are less than or equal to 12 steps apart once per 8 hours. Alternatively, reduce power to less than 50% of RATED POWER within 8 hours.
- 6. If the requirements of Specification 3.12.E.2, 3.12.E.3, 3.12.E.4, or 3.12.E.5 are not satisfied, then the unit shall be placed in HOT SHUTDOWN within 6 hours.

F. DNB Parameters

- 1. The following DNB related parameters shall be maintained within their limits during POWER OPERATION:
 - Reactor Coolant System $T_{avg} \leq$ the limit specified in the CORE OPERATING LIMITS REPORT
 - Pressurizer Pressure > the limit specified in the CORE OPERATING LIMITS REPORT
 - Reactor Coolant System Total Flow Rate ≥ 273,000 gpm and ≥ the limit specified in the CORE OPERATING LIMITS REPORT

6.2 <u>GENERAL NOTIFICATION AND REPORTING REQUIREMENTS</u> Specification

A. The following action shall be taken for Reportable Events:

A report shall be submitted pursuant to the requirements of Section 50.73 to 10 CFR.

- B. Immediate notifications shall be made in accordance with Section 50.72 to 10 CFR.
- C. CORE OPERATING LIMITS REPORT

Core operating limits shall be established and documented in the CORE OPERATING LIMITS REPORT before each reload cycle or any remaining part of a reload cycle. Parameter limits for the following Technical Specifications are defined in the CORE OPERATING LIMITS REPORT:

- 1. TS 3.1.E Moderator Temperature Coefficient
- 2. TS 3.12.A.1, TS 3.12.A.2 and TS 3.12.A.3 Control Bank Insertion Limits
- 3. TS 3.12.B.1 and TS 3.12.B.2 Power Distribution Limits
- 4. TS 3.12.F DNB Parameters
- 5. TS 2.1 Safety Limit, Reactor Core
- 6. TS 2.3.A.2.d Overtemperature ΔT
- 7. TS 2.3.A.2.e Overpower ΔT
- 8. TS Table 4.1-2A Minimum Frequency for Equipment Tests: Item 22 RCS Flow
- 9. TS 3.12.A.1.a, TS 3.12.A.2.a, TS 3.12.A.3.c and TS 3.12.G Shutdown Margin

Amendment Nos. 275 and 275

D. Heacock

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If you have any questions, please feel free to contact me at 301-415-1438.

Sincerely,

/RA/

Karen Cotton, Project Manager Plant Licensing Branch II-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket Nos. 50-280 and 50-281

Enclosure: Corrected TS pages TS 3.12-12 and TS 6.2-1

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