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ACCESSION NBR: 9107260174 DOC. DATE: 91/07/19 NOTARIZED: NO DOCKET #
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 AUTH. NAME AUTHOR AFFILIATION
 CROSBY, R.C. Maine Yankee Atomic Power Co.
 NICHOLS, S.E. Maine Yankee Atomic Power Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 91-007-00: on 910625, determined that open outlet valves from component cooling surge tanks not locked. Detailed root cause evaluation in progress. Valves locked & controlling procedures revised. W/910719 ltr.

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 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

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	AEOD/ROAB/DSP	2	2	NRR/DET/ECMB 9H	1	1
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EXTERNAL:	EG&G BRYCE, J.H	3	3	L ST LOBBY WARD	1	1
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Maine Yankee

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July 19, 1991

MN-91-109 SEN-91-205

UNITED STATES NUCLEAR REGULATORY COMMISSION

Attention: Document Control Desk
Washington, DC 20555

Reference: (a) License No. DPR-36 (Docket No. 50-309)

Subject: Maine Yankee Licensee Event Report 91-007-00, Component Cooling
Valves Not Locked Open.

Gentlemen:

Please find enclosed Maine Yankee Licensee Event Report 91-006-00.
This report is submitted in accordance with the requirements of
10 CFR 50.73 (a)(2)(i).

Please contact us should you have any questions regarding this matter.

Very truly yours,

S. E. Nichols

S. E. Nichols, Manager
Nuclear Engineering & Licensing

SEN/sjj

Enclosure

c: Mr. Thomas T. Martin
Mr. Charles S. Marschall
Mr. E. H. Trottier
Mr. Patrick J. Dostie

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LICENSEE EVENT REPORT (LER)

Facility Name(1) Maine Yankee Atomic Power Company	Docket Number(2) 0 5 0 0 0 3 0 9	Page(3) 1 of 2
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Title(4)
Component Cooling Valves Not Locked Open

Event Date(5)			LER Number(6)			Report Date(7)			Other Facilities Involved(8)	
Month	Day	Year	Year	Sequential Number	Revision Number	Month	Day	Year	Facility Names	Docket Number(s)
0	6	2	5	9	1	9	1	1	Maine Yankee	0 5 0 0 0 3 0 9

This Report is Submitted Pursuant to the Requirements of 10 CFR §
(Check one or more of the following) (11)

Operating Mode (9)	7	20.402(b)	-	20.405(c)	-	50.73(a)(2)(iv)	-	73.71(b)
Power Level (10)	I 0 0	20.405(a)(1)(i)	-	50.36(c)(1)	-	50.73(a)(2)(v)	-	73.71(c)
		20.405(a)(1)(ii)	-	50.36(c)(2)	-	50.73(a)(2)(vii)	-	Other (Specify in
		20.405(a)(1)(iii)	X	50.73(a)(2)(i)	-	50.73(a)(2)(viii)(A)	-	Abstract below
		20.405(a)(1)(iv)	-	50.73(a)(2)(ii)	-	50.73(a)(2)(viii)(B)	-	and in Text, NRC
		20.405(a)(1)(v)	-	50.73(a)(2)(iii)	-	50.73(a)(2)(x)	-	Form 366A)

LICENSEE CONTACT FOR THIS LER (12)

NAME Robert C. Crosby, Nuclear Safety Engineer	Telephone Number Area Code 2 0 7 8 8 2 6 3 2 1
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT 13

Cause	System	Com-ponent	Manufac-turer	Reportable to NPRDS	Cause	System	Com-ponent	Manufac-turer	Reportable to NPRDS

Supplemental Report Expected (14)

(If yes, complete Expected Submission Date)	No	Expected Submission Date(15)	Month	Day	Year
X Yes				N/A	

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During normal operation at 100% power, on June 25, 1991, it was determined that the open outlet valves from the component cooling surge tanks were not locked and therefore not in accordance with plant Technical Specification 3.6. This Technical Specification requires Emergency Core Cooling System manual valves be aligned and locked in the position required for safeguards operation. The valves were subsequently locked, controlling procedures were revised, and a review of the component cooling system revealed that no additional unlocked manual valves were required to be locked. A detailed root cause evaluation is in progress to determine why this oversight in procedural controls occurred and to determine additional measures which may be necessary to ensure present and continued compliance with Technical Specification requirements for locked manual valve control.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

Facility Name(1) Maine Yankee Atomic Power Company	Docket Number(2) 05000309	LER Number (6)						Page(3)	
		Year	Sequential Number		Revision Number				
		91	00	7	0	0	2	of 2	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During normal operation at 100% power, on June 25, 1991, it was determined that the open outlet valves (ISV) from the Component Cooling (CC) Surge Tanks (TK) were not locked and therefore not in accordance with plant Technical Specifications.

The component cooling system is an intermediate cooling system, transferring heat from plant equipment to the service water system (BI); and under accident conditions, to remove reactor sensible and decay heat on a recirculation actuation signal (RAS). The system consists of two similar but separate subsystems, for redundancy for its accident function. The subsystems are primary component cooling water (PCCW) and secondary component cooling water (SCCW) systems. Each subsystem is a closed loop consisting of pumps (P), heat exchangers (HX), distribution piping and a surge tank. The function of the surge tanks, as stated in the Final Safety Analysis Report (FSAR), is to "accommodate all system volume changes caused by temperature variations and maintain a positive head at the suction of each pump."

The PCCW surge tank can be isolated from the PCCW pumps' suction header by one manual isolation valve; the SCCW surge tank can be isolated by two valves. These three valves are maintained in the open position to satisfy their FSAR stated function. The three valves were aligned and controlled by procedure as open valves, but they were not locked according to Technical Specification 3.6 requirements. Technical Specification 3.6 requires "the manual valves (for an operable Emergency Core Cooling System) be aligned and locked in the position required for safeguards operation." The valves were subsequently locked and controlling procedures were revised. A review of the component cooling system revealed that no additional unlocked manual valves were required to be locked.

Previous occurrences of Emergency Core Cooling System valves not being procedurally controlled as locked are documented in Occurrence Report 73-5 and Licensee Event Reports 75-4 and 88-02.

A detailed root cause evaluation is in progress to determine why this oversight in procedural controls occurred and to determine additional measures which may be necessary to ensure present and continued compliance with Technical Specification requirements for locked manual valve control. The evaluation should be completed by September 1, 1991. A Licensed Event Report Supplement will be issued by October 1, 1991.