



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**

REGION III
2443 WARRENVILLE ROAD, SUITE 210
LISLE, IL 60532-4352

July 12, 2011

Mr. Michael J. Pacilio
Senior Vice President, Exelon Generation Company, LLC
President and Chief Nuclear Officer (CNO), Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

**SUBJECT: QUAD CITIES NUCLEAR POWER STATION, UNIT 1
NRC POST APPROVAL SITE INSPECTION FOR LICENSE RENEWAL
INSPECTION REPORT 05000254/2011008**

Dear Mr. Pacilio:

On June 3, 2011, the U.S. Nuclear Regulatory Commission (NRC) completed Phase I of the Post-Approval Site Inspection for License Renewal at your Quad Cities Nuclear Power Station, Unit 1. The enclosed report documents the inspection activities, which were discussed on June 3, 2011, with Mr. S. Darrin and other members of your staff.

This inspection was an examination of activities conducted under your renewed license as they relate to the completion of commitments made during the renewed license application process and compliance with the Commission's rules and regulations and the conditions of your operating license. Within these areas, the inspection involved examination of selected procedures and representative records, observations of activities, and interviews with personnel. On the basis of the sample selected for review, there were no findings of significance identified during this inspection. The NRC staff did not identify any instances of incomplete commitments with respect to timeliness or adequacy.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS), accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

Ann Marie Stone, Chief
Engineering Branch 2
Division of Reactor Safety

Docket No. 50-254
License No. DPR-29

Enclosure: Inspection Report 05000254/2011008
w/Attachment: Supplemental Information

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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket No: 50-254

License No: DPR-29

Report No: 05000254/2011008

Licensee: Exelon Generation Company, LLC

Facility: Quad Cities Nuclear Power Station, Unit 1

Location: Cordova, IL

Dates: May 16 – June 3, 2011

Inspectors: C. Tilton, Senior Reactor Engineer (Lead)
T. Bilik, Senior Reactor Engineer

Approved by: Ann Marie Stone, Chief
Engineering Branch 2
Division of Reactor Safety

Enclosure

SUMMARY OF FINDINGS

IR 05000254/2011008; 05/16/2011 – 06/03/2011; Quad Cities Nuclear Power Station, Unit 1;
Post-Approval Site Inspection for License Renewal

The inspection was conducted by two regional based inspectors. No instances were noted of incomplete license renewal commitments with respect to timeliness or adequacy. The NRC's program for overseeing the safe operation of commercial nuclear power reactors as described in NUREG-1649, "Reactor Oversight Process," Revision 4, dated December 2006.

A. NRC-Identified and Self-Revealed Findings

Cornerstone: Initiating Events, Mitigating Systems, Barrier Integrity

No findings were identified.

B. Licensee-Identified Violations

No violations of significance were identified.

REPORT DETAILS

Summary of Plant Status

Quad Cities Nuclear Power Station, Unit 1 was in a refueling outage during the period of this inspection.

4. OTHER ACTIVITIES

4OA5 Other Activities

.1 Post-Approval Site Inspection for License Renewal (Phase I) - IP 71003

a. Inspection Scope

(1) Review of Newly Identified Structures Systems and Components (SSC)

The inspectors discussed the identification of new SSCs, under the purview of Title 10 of the Code of Federal Regulations (CFR) 54.37(b), with the licensee's license renewal staff. The licensee personnel indicated that no new components have been identified to date that should have been within the scope of the license renewal program.

(2) Review of Revised Commitments

As part of reviewing the Aging Management Programs (AMPs) associated with the commitments, the inspectors reviewed a sample of commitment revisions related to license renewal. The inspectors reviewed licensee's revision to Commitment No. 10, as documented in Appendix A of NUREG 1796, NRC Safety Evaluation of the Dresden Nuclear Power Station, Units 2 and 3 and Quad Cities Nuclear Power Station, Units 1 and 2. Exelon Generation Company, LLC (EGC) made the following commitment:

"An aging management program will be implemented for thermal aging and neutron irradiation embrittlement of Cast Austenitic Stainless Steel (CASS) reactor internal components within the scope of license renewal. A component specific evaluation for the loss of fracture toughness will be included. For those components where the loss of fracture toughness may affect the function of the component, an inspection will be performed as part of the ISI program."

These component specific evaluations require the material composition be obtained from Certified Material Test Reports (CMTRs). In letter dated December 9, 2010, the licensee stated the CMTRs could not be located for components within the scope of this commitment. Therefore, in the December 9, 2010, letter, the licensee revised the aforementioned commitment by stating that "EGC will conservatively assume all applicable [without CMTRs] components are susceptible and establish an inspection program for these components, the same as would be required by the original commitment in an evaluation had concluded there was a loss of fracture toughness potentially affecting component function." In addition, the licensee included in their December 2010 letter a revised commitment stating the following:

"An aging management program will be implemented for thermal aging and neutron irradiation embrittlement of Cast Austenitic Stainless Steel (CASS) reactor internal components within the scope of license renewal. A component

specific evaluation for the loss of fracture toughness will be included. If material composition cannot be determined, a loss of fracture toughness may [emphasis added] be assumed as an alternative to a specific evaluation. For those components where it is assumed or the evaluation has determined a loss of fracture toughness may [emphasis added] affect the function of the component, an inspection will be performed as part of the ISI program and Reactor Internals Programs.”

The inspectors were concerned with the discrepancies between the licensee’s original commitment and the newly revised commitment as the later allowed of the licensee an option of not inspecting some of these components. The inspectors discussed the revised commitment with Nuclear Reactor Regulation (NRR) staff and concluded that it is NRC’s expectations that loss of fracture must be assumed and component inspection performed for every component without CMTRs since a component specific evaluation cannot be performed. After discussion with licensee staff on this commitment revision, the licensee explained that their expectations were aligned with NRC’s expectations. Given the above discussion, the inspectors find the licensee’s commitment revision acceptable.

The inspectors also reviewed the licensee’s commitment tracking program to evaluate its effectiveness.

(3) Review of Commitments

The inspectors reviewed supporting documents including completed surveillance records, conducted interviews, observed non-destructive examination (NDE) activities, performed visual inspection of structures and components, including those not accessible during power operation, and observed the activities described below to verify the licensee completed the necessary actions to comply with the license conditions that are a part of the renewed operating license. The inspectors verified the licensee implemented the “outage related” aging management programs included in NUREG 1796, “NRC Safety Evaluation of the Dresden Nuclear Power Station, Units 2 and 3 and Quad Cities Nuclear Power Station, Units 1 and 2,” in accordance with 10 CFR Part 54, “Requirements for the Renewal of Operating Licenses for Nuclear Power Plants.” The inspectors also verified a selected sample of corrective actions taken as a result of the license renewal inspection.

b. Findings and Observations

Results of Detailed Reviews

The inspectors reviewed portions of the commitments below, which are referenced to Appendix A of the SER. Activities observed related to these commitments are also listed.

(1) (B.1.1) ASME Code, Section XI Inservice Inspection, Subsections IWB, IWC, and IWD (Commitment 1)

The ASME Section XI Inservice Inspection, Subsections IWB, IWC, and IWD aging management program (AMP) is part of the inservice inspection (ISI) program (an existing program) and provides for condition monitoring of reactor coolant pressure retaining piping and components within the scope of license renewal, except for the

reactor pressure vessel. It also provides for condition monitoring of reactor internal components within the scope of license renewal. The program is implemented through procedures that require examinations consistent with ASME Section XI and approved relief request PR-02.

A Risk Informed Inservice Inspection (RISI) is implemented in lieu of ASME Section XI requirements for portions of Class 1 and Class 2 systems. The RISI is implemented as an alternative to the 1995 Edition through 1996 Addenda of ASME Section XI requirements for portions of Class 1 and Class 2 systems.

With enhancement, the ASME Section XI Inservice Inspection, Subsections IWB, IWC, and IWD, the AMP is consistent with the ten elements of Aging Management Program XI.M1, "ASME Section XI Inservice Inspection, Subsections IWB, IWC, and IWD," specified in NUREG-1801.

The inspectors conducted interviews and reviewed documentation. The inspectors also observed a number of NDE examinations as part of the RISI Program. These included ultrasonic testing (UT) of recirculation system, 02AD-S2, pipe-to-tee welds, and residual heat removal (RHR) 10AD-S3, elbow-to-pipe seam welds, performed under work order (WO) 001283272. The inspectors had no concerns with the observed activities.

(2) (B.1.11) Flow Accelerated Corrosion (FAC) (Commitment No. 11)

The Flow Accelerated Corrosion (FAC) AMP is an existing program to be enhanced, which will be enhanced to include portions of the main steam and the reactor vessel head vent systems that are within the scope of license renewal. The program's activities predict, detect, and monitor wall thinning in piping and valves due to flow-accelerated corrosion. It also provides for prediction of the amount of wall thinning in pipes and fittings through analytical evaluations and periodic examinations of locations most susceptible to FAC induced loss of material. Specifically, the program includes analyses to determine critical locations, baseline inspections to determine the extent of thinning at these locations, and follow-up inspections to confirm the predictions. Repairs and replacements are performed as necessary.

The inspectors observed a surveillance credited for the FAC AMP. This surveillance, documented in report 1B2-V09 BR, performed an ultrasonic thickness measurement of portions of the feedwater heater vents to detect any material loss as a result of FAC.

(3) (B.1.20) Above-Ground Carbon Steel Tanks (Commitment No. 20)

The Above-Ground Carbon Steel Tanks AMP is an existing program, which applies to the carbon steel nitrogen storage tanks used in the Drywell Nitrogen Inerting System. In addition to the carbon steel tanks, this AMP also includes aluminum above ground tanks, which is the Contaminated Condensate Storage Tank (CCST) at Quad Cities Nuclear Power Station.

With enhancements, the Above-Ground Carbon Steel Tanks AMP is consistent with the ten elements of aging management program XI.M29, "Above-ground Carbon Steel Tanks," specified in NUREG-1801. Specifically, the program will provide for initiating documentation of inspection results of periodic system engineer walkdowns of the nitrogen storage tanks. The enhancements include:

- The program will provide for initiating documentation of inspection results of periodic system engineer walkdowns of the nitrogen storage tanks. Since the carbon steel nitrogen storage tanks included in this AMP are not ground-level tanks, the discussions regarding concrete or earthen foundations and inaccessible tank bottoms do not apply to these tanks.
- The program will provide for the performance of periodic internal/external visual inspections of the outdoor aluminum tank and a one-time UT inspection of the bottom of one of the in-scope outdoor aluminum tank at Quad Cities. The aluminum tank is not painted. It is visually inspected to manage the effects of corrosion on the intended function of this tank. Inspections will be performed on the entire surface of the tank up to the surface in contact with the foundation and the sealants/coatings at the foundation interfaces. In order to detect corrosion that may occur at the bottom external surfaces that are inaccessible, internal UT inspections will be performed on the tank bottom.

The inspectors noted NUREG-1801 does not address outdoor above ground aluminum tanks, and therefore including inspections on this tank as part of the aboveground carbon steel tank program is an exception.

To verify the program's effectiveness, the inspectors performed interviews and also reviewed completed WO 01070836 related to AMP B.1.20, which credits the Above-Ground Carbon Steel Tanks Program commitment to perform a one-time UT thickness inspection of the aluminum condensate storage tank. The inspectors had no concerns with the observed activities.

(4) (B.1.23) One-Time Inspection Program (Commitment No. 23 (F, H and L))

The One-Time Inspection AMP is a new program provides one-time inspections that manage aging effects of identified components within the scope of license renewal. The purpose of the program is to determine if a specified aging effect is occurring. If the aging effect is occurring, the program provides for evaluation of the effect it will have on the ability of affected components to perform their intended functions for the period of extended operation, and appropriate corrective action. The program is implemented through station procedures.

The inspectors reviewed the licensee's activities to implement Commitment Item No. 23, of the NRC license renewal SER. This commitment documents that the licensee agreed to perform one-time inspection sampling of 12 specific inspection areas.

To verify the program's effectiveness the inspectors interviewed the program owner and reviewed completed WO 00588912, related to AMP B.1.23L, which credits the One-Time Inspection Program to perform a one-time visual inspection of the U1 torus saddle lubrite base-plates for loss of material due to galvanic corrosion, lock-up, or wear. In addition, the inspectors interviewed the program owner and reviewed completed WO 00592919, related to AMP B.1.23H, which credits the One-Time Inspection Program to perform a one-time ultrasonic testing (UT) inspection of the of the U1 emergency diesel generator day tank 1-5202/T05 for loss of material. The inspectors reviewed completed WO 00837220, related to AMP B.1.23F, which credits the One-Time Inspection Program to perform a one-time visual inspection of the reactor core isolation cooling (RCIC) turbine exhaust header vacuum breaker check valve. The inspectors had no concerns with the observed activities.

(5) (B.1.24) Selective Leaching of Materials (Commitment No. 24)

The Selective Leaching of Materials AMP is a new program. The Selective Leaching of Materials AMP consists of numerous one-time inspections to determine if selective leaching of materials is occurring. The scope of the program includes susceptible components within the scope of license renewal that are exposed to chemically treated water, de-mineralized water, raw water and ground water, and moist ventilation and gas environments. Susceptible component materials are gray cast iron, copper alloys with less than 85 percent copper, aluminum bronze, and Muntz metal.

The program is consistent with the ten elements of aging program XI.M33, "Selective Leaching of Materials," specified in NUREG-1801 with the following exceptions. NUREG-1801 indicates that the Selective Leaching of Materials AMP includes a one-time hardness measurement of a selected set of components. The Quad Cities program provides for visual examination in lieu of hardness testing.

The commitment documents that a sample of components that are made of susceptible materials will be visually inspected for evidence of selective leaching and that the sample will be expanded if failed conditions are identified.

To verify the program's effectiveness, the inspectors reviewed program documents and VT examination results performed in accordance with completed WO 00938446, of turbine strainer, 0-4130 and also three other work orders, WO 01282084, WO 00880546, and WO 00692750.

Some ambiguity and lack of content in the first completed examination report (VT3-026) lead to the inspectors questioning the adequacy of the selective leaching exams being performed. In a letter to staff dated November 20, 2003, the licensee indicated that visual inspections [for the selective leaching program] would be performed in accordance with ASME Code Section XI VT-1. The inspectors, with concurrence from NRR, understood this to mean that the examinations would be performed with VT-1 qualified personnel following the requirements of ASME Section XI for VT-1 examinations. It was the licensee's belief that the commitment was limited to the use of VT-1 qualified personnel to perform the examinations but not to the requirements of the ASME Code. This issue was capture in AR 101562.

The licensee agreed that based on the verbiage, an ASME Code Section XI VT-1 should have been completed for the selective leaching exams. As such, the licensee intends to submit a commitment revision to clarify their commitment. This commitment revision will be submitted to NRR for revision and acceptability.

(6) (B. 1.26) ASME Code Section XI, Subsection IWE (Commitment No. 26)

The ASME Section XI, Subsection IWE aging management program is an existing program and provides for primary containment inspections for loss of material. The program includes visual examination and limited surface or volumetric examination, when augmented examination is required. It is implemented through station plans and procedures and covers steel containment shells and their integral attachments; containment hatches and airlocks; seals, gaskets and moisture barriers; and pressure retaining bolting. This AMP will be enhanced as follows:

1. The program will be based on the latest edition and addenda, which is approved by the NRC 12 months prior to the end of the current 120-month inspection interval.
2. The program will be updated in accordance with 10 CFR 50.55(a).
3. Requirements will be provided in procedures to ensure sand pocket drains are clear.
4. The pressurized testing methodology will be credited for managing the aging of bellows.

The inspectors reviewed completed WO 01256579, which performed a surveillance of water leakage through the Dryer-Separator Pool, Spent Fuel Pool, and Drywell Liners to verify the integrity of these components. The inspectors had no concerns with the observed activities.

(7) (B. 1.32) Protective Coating Monitoring and Maintenance Program (Commitment No. 32)

The inspectors reviewed the licensee's activities to implement Commitment Item No. 32, of the NRC license renewal SER. This commitment documents that the existing Protective Coating Monitoring and Maintenance Program is credited and that the program will be enhanced as follows: (1) visual inspection of Service Level I coatings near sumps or screens associated with the emergency core cooling system; (2) pre-inspection review of previous reports so that trends can be identified for the program; and (3) analysis of coating failures to determine reasons for failures.

The Protective Coating Monitoring and Maintenance Program provide for aging management of Service Level I coatings inside primary containment. Service Level I coatings are used in areas where the coating failure could adversely affect the operation of post-accident fluid systems and thereby impair safe shutdown. With the enhancements noted, the program is consistent with the ten elements of Aging Management Program XI.S8, "Protective Coating Monitoring and Maintenance Program," specified in NUREG-1801.

The coatings program uses the guidance provided in Electric Power Research Institute (EPRI) Technical Report TR-109937, "Guidelines on Nuclear Safety-Related Coating," to develop a corporate procedure for performing coating inspections.

The inspectors reviewed program documents and observed coating examinations of the "centipede" per direction of procedure ER-QC-330-1000. The inspection consisted of areas behind the missile shield, the vent headers, and the down-comers in accordance with WO1255481. The inspectors had no concerns with the observed activities.

(8) (B.1.33) Electrical Cables and Connections Not Subject to 10 CFR 50.49 Environmental Qualification Requirements (Commitment No. 33)

The inspectors reviewed the licensee's activities to implement Commitment Item No. 33, of the NRC license renewal SER. This commitment documents that the licensee agreed to develop a program that is consistent with NUREG-1801 AMP X1.E1 for electrical cables and connections installed in adverse localized environments not subject to 10 CFR 50.49 environmental qualification requirements.

The AMP for electrical cables and connections not subject to 10 CFR 50.49 environmental qualification requirements is a new program and is consistent with the ten elements of aging program XI.E1, "Electrical Cables and Connections Not Subject to 10 CFR 50.49 Environmental Qualification Requirements," specified in NUREG-1801. The program manages cables and connections within the scope of license renewal that are subject to an adverse environment. It also identifies and manages cables and connections subject to an adverse localized environment.

Adverse localized environments would be the areas that have temperature, radiation, and moisture significantly higher than the maximum normal (plant design) ambient conditions, and could appreciably increase the rate of aging of the cables or connections or have an immediate adverse effect on operability.

Cables and connections subject to an adverse environment are managed by inspection of a sample of these components. Selected cables and connections from accessible areas are inspected and represent, with reasonable assurance, the cables, and connections in adverse environments. They are inspected for signs of accelerated age-related degradation. Additional inspections, repairs, or replacements are initiated as appropriate.

The Quad Cities AMP provides reasonable assurance that the intended functions of the non-EQ electrical cables and connections within the scope of 10 CFR Part 54 exposed to adverse localized environments caused by heat, radiation or moisture will be maintained through the extended period of operation.

The licensee identified a representative sample of accessible electrical cables and connections installed in adverse localized environments and visually inspected for cable and connection jacket surface anomalies, such as discoloration, swelling, cracking, or surface contamination. The inspectors reviewed program documents and accompanied the licensee to observe visual examination walkdowns of cables in several adverse environmental areas in the plant per direction of procedure MA-AA-723-500 and in accordance with WO 01416771. The inspectors had no concerns with the observed activities.

4OA6 Management Meetings

.1 Exit Meeting Summary

On June 3, 2011, the inspectors presented the inspection results to Mr. S. Darrin and other members of the licensee staff. The licensee acknowledged the issues presented.

The inspectors confirmed that none of the potential report input discussed was considered proprietary.

ATTACHMENT: SUPPLEMENTAL INFORMATION

SUPPLEMENTAL INFORMATION

KEY POINTS OF CONTACT

Licensee

S. Darrin, Engineering Director
W. Beck, Regulatory Assurance Manager
A. Lewis, Regulatory Assurance
M. Wagner, Regulatory Assurance

LIST OF ITEMS OPENED, CLOSED AND DISCUSSED

Opened, Closed, Discussed

None

LIST OF DOCUMENTS REVIEWED

The following is a list of documents reviewed during the inspection. Inclusion on this list does not imply that the NRC inspectors reviewed the documents in their entirety, but rather, that selected sections or portions of the documents were evaluated as part of the overall inspection effort. Inclusion of a document on this list does not imply NRC acceptance of the document or any part of it, unless this is stated in the body of the inspection report.

40A5 Other Activities

License Renewal Program Basis Documents

Inspections Observed

1B2-V09 BR; UT Feedwater Heater Vents; May 17, 2011

WO 01255481; Inspect Coating Condition on the Centipede; May 17, 2011

WO 01283272; Perform UT-2 Examinations of Pipe Welds for LR; May 17, 2011

WO 01416771; (LR) Perform MA-AA-723-500 Att. 3 on MSIV Rm Cables; May 16, 2011

Examination Records

Q1R21-011; UT Examination Summary Sheet for Pipe-to-Tee (3 Seam Welds) for Component 02AD-S2; May 17, 2011

Q1R21-044; UT Examination Summary Sheet for Elbow-to-Pipe (3 Seam Welds) for Component 10AD-S3; May 17, 2011

UT-025; UT Thickness Report, 'A' CCST (0-3303-A); December 20, 2007

Procedures

ER-AA-335-004; Ultrasonic Measurement of Material Thickness and Interfering Conditions; Revision 5

ER-AA-335-014; VT-1 Visual Examination; Revision 5

ER-AA-335-016; VT-3 Visual Examination of Component Supports, Attachments and Interiors of Reactor Vessels; Revision 6

ER-AA-335-017; VT-3 Visual Examinations of Pump and Valve Internals; Revision 5

ER-AA-335-047; Attachment 2; Vendor Personnel NDE Certification Review Form; Revision 0

ER-AA-700-1001; Aging Management Program Site Implementation Guideline; Revision 1

ER-QC-330-1000; Primary Containment and Coating Inspections; Revision 2

HU-AA-104-101; Procedure Use and Adherence; Revision 4

LS-AA-110; Commitment Management; Revision 7

MA-AA-723-500; Inspection of Non EQ Cables and Connections for Managing Adverse Localized Environments; Revision 4

Completed Work Orders Reviewed

WO 00588912; One-Time VT-3 Inspection of Torus Lubrite Baseplates-License Renewal; May 9, 2011

WO 00592919; One-Time Inspect U1 EDG Day Tank UT (License Renewal); June 30, 2009

WO 00692750; One-Time Inspect U2 DG Cool Water Drn Vlv (License Renewal); October 28, 2008

WO 00837220; U1 RCIC Turbine Exhaust Header Vacuum Breaker Check Valve Maintenance Inspect; May 13, 2011

WO 00880546; (LR) HPCI Lube Oil Clr EC/Inspect; March 17, 2008

WO 00938446; Strainer Flush 2nd Floor Turbine Behind U1 TBC; August 3, 2007

WO 01070836; Inspect Pipe Located Outside Power Block for Integrity; December 20, 2007

WO 01256579, Dryer Separator Pool, Spent Fuel Pool, Drywell Liner Drain; May 11, 2011

WO 01282074; Inspect Service Water System Butterfly Valve Disk for Cracks; April 12; 2011

AR's Generated

AR 01220460; License Renewal Requirements Regarding Visual Inspections; May 25, 2011

AR 01218171; LR-A CCST UT Examination; May 19, 2011

AR 01218173; Procedure not at Job Site; May 19, 2011

AR's Reviewed

AR 00656976; During One-Time Inspection General Corrosion Found; August 3, 2007

AR 00714795; Corrosion/Crud Spots Found During CCST Internal Inspection; December 21, 2007

AR 01216296; LR Q1R21 Stabilizer Exam Results at 45 and 90 Degree Locations; May 15, 2011

Miscellaneous

Drawing 1 DW 1; U-1 Drywell, Basement 579 Elv.; Revision 10

Drawing 1 Tor A; U-1 Suppression Chamber and Centipede; Revision 4

M-3103; Inservice Inspection Isometric Recirculation System; Revision K

M-3105; Inservice Inspection Isometric Residual Heat Removal System; Revision H

LIST OF ACRONYMS USED

ADAMS	Agency wide Document Access Management System
ASME	American Society of Mechanical Engineers
AMP	Aging Management Program
AR	Action Request
CAP	Corrective Action Program
CFR	Code of Federal Regulations
CMTS	Certified Material Test Reports
CASS	Cast Austenitic Stainless Steel
EC	Eddy Current
EGC	Exelon Generating Company, LLC
EQ	Environmental Qualification
GALL	Generic Aging Lessons Learned
GTE	Greater Than or Equal to
HQ	Head Quarters
IMC	Inspection Manual Chapter
IR	Inspection Report
ISI	Inservice Inspection Program
LR	License Renewal
LRA	License Renewal Application
NDE	Nondestructive Examination
NRC	U.S. Nuclear Regulatory Commission
NRR	Nuclear Reactor Regulation
NUREG	Nuclear Regulatory
PARS	Publicly Available Records
RCIC	Reactor Core Isolation Cooling
RHR	Residual Heat Removal
SDP	Significance Determination Process
SER	Safety Evaluation Report
SG	Steam Generator
SSC	Structures, Systems, Components
UT	Ultrasonic Thickness
VT	Visual Testing
WO	Work Order

Mr. Michael J. Pacilio
 Senior Vice President, Exelon Generation Company, LLC
 President and Chief Nuclear Officer (CNO), Exelon Nuclear
 4300 Winfield Road
 Warrenville, IL 60555

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Sincerely,

/RA/
 Ann Marie Stone, Chief
 Engineering Branch 2
 Division of Reactor Safety

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Letter to Mr. Michael J. Pacilio from Ms. Ann Marie Stone dated July 12, 2011.

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