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AUTH.NAME AUTHOR AFFILIATION
DREESEN,. Wisconsin Public Service Corp.
MARCHI,M. Wisconsin Public Service Corp.
RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 96-012-00:on 961217,RHR pump flow instrument not checked IAW TSs.Cause being investigated.SP 87-149 revised.W/970116 ltr.

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NRC-97-4

January 16, 1997

10 CFR 50.73

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Ladies/Gentlemen:

Docket 50-305
Operating License DPR-43
Kewaunee Nuclear Power Plant
Reportable Occurrence 96-012-00

In accordance with the requirements of 10 CFR 50.73, "Licensee Event Report System," the attached Lieensee Event Report (LER) for reportable occurrence 96-012-00 is being submitted.

Sincerely,

M L Marchi

M. L. Marchi
Manager - Nuclear Business Group

JDD/jmf

Attach.

cc - INPO Records Center
US NRC Senior Resident Inspector
US NRC, Region III

IE22 1/1

NRC FORM 366 (4-95) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104
EXPIRES 04/30/98

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Kewuanee Nuclear Power Plant	05000305	1 OF 3

TITLE (4)

RHR Pump Flow Instrument Not Checked in Accordance with Technical Specifications

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
12	17	96	96	-- 012	-- 00	1	16	97	N/A	05000
									FACILITY NAME	DOCKET NUMBER
										05000

OPERATING MODE (9)	N
POWER LEVEL (10)	000

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)

20.2201(b)	20.2203(a)(2)(v)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)
20.2203(a)(1)	20.2203(a)(3)(i)	50.73(a)(2)(ii)	50.73(a)(2)(x)
20.2203(a)(2)(i)	20.2203(a)(3)(ii)	50.73(a)(2)(iii)	73.71
20.2203(a)(2)(ii)	20.2203(a)(4)	50.73(a)(2)(iv)	OTHER
20.2203(a)(2)(iii)	50.36(c)(1)	50.73(a)(2)(v)	Specify in Abstract below or in NRC Form 366A
20.2203(a)(2)(iv)	50.36(c)(2)	50.73(a)(2)(vii)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Include Area Code)
Jay Dressen	(414)388-2560 x2233

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE).	X	NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On December 17, 1996, with the plant in refueling shutdown, it was determined that the requirement of Technical Specification (TS) Table 4.1-1 item 14 was not being met. Item 14 of TS Table 4.1-1 requires the Residual Heat Removal (RHR) Pump flow instrument to be checked each shift when RHR is in operation. When the plant is in Cold Shutdown and Refueling Shutdown modes, Surveillance Procedure (SP) 87-149, "Shift Channel Checks - Shutdown," is performed. SP 87-149 did not have the Nuclear Control Operator (NCO) document a check of the RHR pump flow instrument.

The root cause of this event could not be definitively determined. A review of the SP 87-149 historical file was performed and it was determined that RHR pump flow was never included in the surveillance procedure. It is assumed that personnel developing and auditing SP 87-149 thought that the requirement of TS Table 4.1-1 item 14 was being met by SP 87-125, "Shift Instrument Channel Checks - Operating," which requires documentation of RHR pump flow when RHR is in operation during the Hot Shutdown, Hot Standby, and Operating Modes. SP 87-149 has been revised to document RHR pump flow when RHR is in operation. The NCOs documented RHR pump flow, in the control room log, from December 18, 1996 until SP 87-149 was revised.

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		YEAR	SEQUENTIAL	REVISION	
Kewaunee Nuclear Power Plant	05000305	96	-- 012	-- 00	2 OF 3

TEXT *(If more space is required, use additional copies of NRC Form 366A)* (17)

DESCRIPTION OF EVENT

On December 17, 1996, with the plant in Refueling shutdown mode, Operations personnel questioned if Technical Specification (TS) Table 4.1-1 item 14 was being performed. Item 14 of TS Table 4.1-1 requires Residual Heat Removal (RHR) pump flow instrument channel to be checked each shift when RHR is in operation.

Surveillance Procedure (SP) 87-125, "Shift Instrument Channel Checks - Operating," has the Nuclear Control Operator (NCO) document RHR pump [P] flow when RHR is in operation. This SP is required during all modes of plant operation except Cold Shutdown and Refueling Shutdown. When the plant is in the Cold Shutdown and Refueling Shutdown modes, SP 87-149, "Shift Channel Checks - Shutdown," is performed. SP 87-149 did not have the NCO document RHR pump flow channel check. SP 87-149 does however, have the NCO document Reactor Coolant System (RCS) hot leg temperature for both loops and RHR pump suction header temperature. With the NCOs monitoring hot leg temperatures and RHR pump suction header temperature degraded RHR pump flow would be noted and corrective actions would be taken if needed.

CAUSE OF EVENT

The root cause of this event could not be definitively determined. A review of the SP 87-149 historical file was performed and it was determined that RHR pump flow was never included in the surveillance procedure. It is assumed that personnel developing and auditing SP 87-149 thought that the requirement of TS Table 4.1-1 item 14 was being met by SP 87-125, which requires documentation of RHR pump flow when RHR is in operation.

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TEXT *(If more space is required, use additional copies of NRC Form 366A)* (17)

ANALYSIS OF EVENT

This event is being reported in accordance with 10 CFR50.73(a)(2)(I)(B) as an event or condition prohibited by the plant's Technical Specifications.

SP 87-149 instructs the NCO to document RCS hot leg and RHR pump suction header temperatures. With the NCO monitoring/trending RCS temperature the NCO is indirectly monitoring RHR pump flow. The NCO would notice if RCS temperature is increasing and check the operation of the RHR pumps. The NCOs also perform control board walkdowns and monitor computer logs which would prompt the operator if RHR pump flow was degrading or lost. Therefore, there was no increased risk to the health and safety of the public during the time RHR pump flow was not documented in SP 87-149.

CORRECTIVE ACTIONS

1. SP 87-149 has been revised to document RHR pump flow when RHR is in operation.

2. The NCOs documented RHR pump flow, in the control room log, from December 18, 1996 until SP 87-149 was revised.

ADDITIONAL INFORMATION

None.

EQUIPMENT FAILURE

None.

SIMILAR EVENTS

None.