· ·													
•		Kewaune	PUBLIC SER Pe Nuclear RATING PRO	Power P)N		NO. TITLE	Los	-10 s of	Reactor		ant
REVIEWE	D BY _	N	Juth	iam				DATE OC	14	1984	PAGE DUTUU	l of	
1.	.0 <u>IN</u>	TRODUCI	LION										
	Th	e purpo	ose of this	s emergei	ncy pro	cedure is	to:						
						erm coolin		vent cl.	addin	n dam	300		
	1.2	2 Main sump	tain long water.	term shu	utdown	and coolin	g by rec	irculat	ion o	f the	conta;	nment	
2.	0 <u>SYM</u>	IFTOMS											
	The Saf	The following symptoms MAY be present in addition to those utilized in E-O-O7, Safety Injection Actuation.											
2	2.1	High	containme	nt press	ure, hu	umidity and	d activit	v (R-2	and R	-7)			
T	2.2	Decre	asing pre-	ssurizer	DECCO	re and pre steam spac	•	_			loss of as pre	f essure	
	2.3	Steam	pressure	constant	t in bo	th steam g	enerator	s.					
			level in t										
3.0			ACTION										
	3.1	Autom	atic										
		3.1.1	Safety I occur.	njection	is act	tuated and	the auto	omatic a	iction	is of	E-0-07		
	3.2	Operat	tor										
		3.2.1	Perform (Reactor)	the Immed Trip.	diate ()perator Ac	ctions of	E-0-04	, Tur	bine	and		
		3.2.2				pump disc ual flow i tion indic		ow from n OR pur	at la np di	east schar	one ge		

3.2.3 Observe the SI and CI active status panels. Follow up any automatic action which did not occur.

3.2.4 If at any time RCS pressure DECREASES to 1500 psig or less AND safety injection flow is verified, TRIP both Reactor Coolant Pumps. PDR ADDCK 05000483 Q PDR

	WISCONSIN PUBLIC SERVICE CORPORATION	NO. E-0-10	:
(Kewaunee Nuclear Power Plant	TITLE Loss of Reac	tor Coolant
	OPERATING PROCEDURE	DATE OCT 2 1984 PA	GE 2 of 8

4.0 SUBSEQUENT ACTION

- NOTE: THROUGHOUT THE SUBSEQUENT ACTIONS, THE EMERGENCY PLAN IMPLEMENTING PROCEDURES SHOULD BE REVIEWED TO EVALUATE IF THE EMERGENCY RESPONSE ORGANIZATION SHOULD BE ACTIVATED.
- 4.1 Maintain narrow range indication on both steam generators. If level increases in an unexplained manner go to E-0-09 and investigate for a possible steam generator tube rupture.
- 4.2 Maintain seal injection flow to both reactor coolant pumps (verify adequate flow using labyrinth differential pressure).
- 4.3 CLOSE both pressurizer PORV's.
- 4.4 If the RWST level is decreasing rapidly such that the low level alarm (37%) appears imminent, GO directly to step 4.10.
- 4.5 Evaluate the following plant conditions:
 - a. RCS pressure > 2000 psig and stable or increasing
 - b. At least one steam generator indicates narrow range level
 - c. Pressurizer level is > 20%
 - d. RCS subcooling is > 50°F
- 4.6 If <u>ANY</u> of the above conditions are <u>NOT</u> established, GO directly to step 4.8.
- 4.7 If <u>ALL</u> of the above conditions are established, SI can be terminated; continue with the following steps.
 - 4.7.1 RESET safety injection
 - 4.7.2 RESET containment isolation
 - 4.7.3 STOP both safety injection pumps
 - 4.7.4 If pressurizer level decreases below 20%, or RCS subcooling is less than 50°F, then manually re-initiate SI and GO to step 4.8.
 - 4.7.5 Open instrument air to containment, valve IA-101
 - **4.7.6** Open RCP seal water return motor valves, CVC211 and CVC212
 - 4.7.7 Re-establish normal PZR level control. Adjust reactor makeup concentration as necessary.

Kewaunee	LIC SERVICE CORPORATION Nuclear Power Plant TING PROCEDURE		NO. TITLE	E-0-10 Loss of Reactor Coolar		
 			DATE OCT	2 1 984	PAGE	3 of 8
4.7.8	Re-establish normal PZR pr	essure contro	0]			
		rate to cold shutdown and perform a controlled cooldown				

- 4.8 Maitain the safety injection pumps in operation, unless operation was terminated in step 4.7.
- NOTE: If a blackout occurs after SI is reset, manually restart safety injection components, as needed.
- 4.9 If the RCS pressure stabilizes above the RHR pumps shutoff head: RESET SI and STOP both RHR pumps.
- 4.10 As the RWST level decreases, check for Containment Sump B level increase; if increase is NOT evident, return to E-0-07 Safety Injection Actuation, to re-evaluate the symptoms.
- 4.11 If RCS pressure is decreasing slowly or is stabilizing above 100 psig, commence RCS cooldown at \leq 50°/hr using steam dump or steam generator PORV's.
- 4.12 If containment pressure is below 4 psig and the containment spray pumps are operating, perform the following:
 - 4.12.1 RESET containment spray
 - 4.12.2 STOP both containment spray pumps
 - 4.12.3 CLOSE the caustic additive valves
- 4.13 If any safeguards equipment failed to function, attempt control room, or local, manual action as required.
- 4.14 Monitor incore thermocouples (I1100G Hottest, I1101B Average) and wide range RCS hot leg recorder, for indication of core temperature.
 - 4.14.1 Maintain \geq 50°F subcooling using steam dumps.
 - 4.14.2 For loss of adequate heat removal refer to Appendix

4.15 Unlock and place in ON the MCC breakers for the following motor valves:

VALVE	MCC	MOTOR NO.
SI-9A	1-52B	1-381
SI-11A	1-52B	1-386
SI-11B	1-62B	1-384

·	WISCONSIN PUBLIC SERVICE CORPORATION	NO.	E-0-10		
•	Kewaunee Nuclear Power Plant OPERATING PROCEDURE	TITLE	Loss of	Reactor	Coolant
(DATE	2 1984	PAGE	4 of 8

- 4.16 OPEN the component cooling to the RHR heat exchangers, CC-400A and CC-400B.
- 4.17 At the RWST low level alarm (37%), align ONE train for recirculation as follows:
- NOTE: If a component failure makes one train inoperable for recirculation proceed directly to Step 4.19.
 - 4.17.1 RESET safety injection.
 - 4.17.2 AS required by RCS and CNTMT pressure START, or verify running, the SI, RHR, and ICS pumps of one train then STOP the SI, RHR, and ICS pumps in the OTHER train.
 - 4.17.3 CLOSE SI-208 and SI-209, Recirc. to RWST.
 - NOTE: Valve SI-208 OR SI 209 must close to open SI-350 and SI-351, Sump B to RHR pump suction.

In the shutdown train:

4.17.4 CLOSE SI-300A(B), RWST to RHR pump suction. If the valve fails to close continue with these steps but take manual action to close the valve.

4.17.5 OPEN SI-350A(B) and SI 351A(B), Sump B to RHR pump suction.

4.18 At the RWST low-low level alarm (4%), complete the transfer to the recirculation mode as follows:

4.18.1 Shutdown the remaining RHR, SI, and ICS pumps.

- 4.18.2 START the RHR pump aligned for recirculation.
- 4.18.3 Verify recirculation flow with the RHR system. If no flow exists:
 - a. CLOSE SI-5A (B), SI pump suction.
 - b. When SI-5A (B) is closed, OPEN RHR-300A (B), RHR heat exchanger to SI pump.
 - c. START SI pump 1A(1B) and verify recirculation flow.
- **4.18.4** If high containment pressure exists, open RHR-400A (B), RHR heat exchanger to ICS pump, and start the corresponding ICS pump.

4.18.5 Line up the remaining train for recirculation operation.

•	WISCONSIN FUBLIC SERVICE CORFORATION	NO. E-0-10
	Kewaunee Nuclear Power Plant OPERATING PROCEDURE	TITLE Loss of Reactor Coolant
Ĺ	OFERALING PROCEDUKE	DATE OCT 2 1984 PAGE 5 of 8

- 4.19 If one train of RHR, SI, or ICS has failed that will prevent recirculation flow to the core or prevent containment spray if it is required, complete the following steps at the RWST low-low level (4%) FOR THE RUNNING TRAIN:
 - 4.19.1 CLOSE SI-208 and SI-209, recirc. to RWST.
 - NOTE: Valve SI-208 OR SI-209 must close to open SI-350 and SI-351, Sump B to RHR pump suction.
 - 4.19.2 OPEN SI-350A (B) and SI-351A (B), Sump B to RHR pump suction.
 - 4.19.3 CLOSE SI-300A (B), RWST to RHR pump suction. If the valve fails to close continue with these steps but take manual action to close the valve.
 - 4.19.4 If required, START RHR pump 1A (B).
 - 4.19.5 Verify recirculation flow with the RHR system. If no flow exists:
 - a. Shutdown the running SI pump.
 - b. CLOSE SI-5A (B), SI pump suction. If this valve fails to close take immediate corrective action.
 - c. OPEN RHR-300A (B), RHR heat exchanger to SI pump.
 - d. Restart the SI pump.
 - e. Verify recirculation flow.
 - 4.19.6 If high containment pressure exists, OPEN RHR-400A (B), RHR heat exchanger to ICS pump, and start the corresponding ICS pump.

WISCONSIN PUBLIC SERVICE CORPORATION

NO. E-O-10 TITLE Loss of Reactor Coola.

Kewaunee Nuclear Power Plant OPERATING PROCEDURE

DATE OCT 2 1984 PAGE

۰.

6 of

(

Ő

•	WISCONSIN PUBLIC SERVICE CORPORATION	NO. E-0-10
	Kewaunee Nuclear Power Plant OPERATING PROCEDURE	TITLE Loss of Reactor Coolan
		DATE OCT 2 1984 PAGE 7 of {

APPENDIX

i.

INSTRUCTIONS TO RESTORE CORE COOLING DURING A SMALL LOCA

1.0 INTRODUCTION

This Appendix describes operator actions in the event adequate core cooling is lost during a small break LOCA.

2.0 SYMPTOMS

2.1 Incore thermocouple temperatures are rapidly increasing.

3.0 IMMEDIATE ACTIONS

- 3.1 Operator
 - 3.1.1 Initiate MANUAL SI and verify injection flow to the Reactor Coolant System.

4.0 SUBSEQUENT ACTIONS

- 4.1 Monitor incore thermocouples to determine effectiveness of actions taken.
- 4.2 In the event of equipment failure, attempt to take manual action.
- 4.3 Throughout this instruction, continue effort to provide:
 - a. Safety Injection to the Reactor Coolant System.
 - b. Feedwater flow to the steam generator (s).
- 4.4 Depressurize the Reactor Coolant System by:
 - a. Atmospheric or condenser steam dump
 - b. Open both pressurizer PORV's ONLY IF:
 - Reactor Coolant System depressurization cannot be accomplished with the steam generator.
 - Safety injection is capable of being delivered to the Reactor Coolant System, i.e., pumps running.

	WISCONSIN PUBLIC SERVICE CORPORATION	NO.	E-0-10	
•	Kewaunee Nuclear Power Plant OPERATING PROCEDURE	TITLE		Reactor Coolan
<u> </u>		DATE	2 1984	PAGE 8 of

4.5 If no means of depressurizing the Reactor Coolant System is available, or if the depressurization did not result in decreasing incore thermocouple temperatures, then start a reactor coolant pump if possible.

ς.

ANNUNCIATOR NUMBER 47017-15

SETPOINT 10	% ON 1/4 SENSORS
SYSTEM	CV-35
FLOW DWG	X-K100-38
LOGIC DWG.	E-2029
S.E.R. PT	NONE
COMPUTER PT.	NONE
FUNCT. DESC.	E-752
OTHER E-2	033

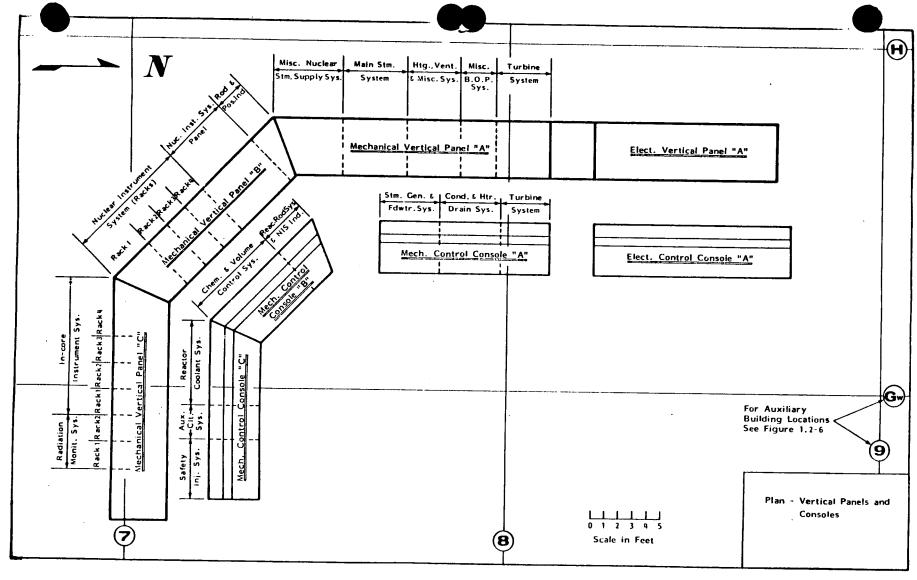
BORIC ACID	
TANK IA LEVEL	
LOW - LOW	
·	

COMMENTS: 1. LO-LO LEVEL IN THE SI SELECTED BORIC AC10 TANK CAUSES THE REFUELING WATER TO SAFETY INJECTION PUMO'S SUPPLY VALVES (SI-4A, SI-4B) TO OPEN AND THE BORIC ACTO SUPPLY VALVES (SI-2A, SI-2B) TO SAFETY INJECTION PURPS TO CLOSE.

RECOMMENDED ACTION: 1. IF SI HAS BEEN ACTIVATED VERIFY THAT AT 10% LEVEL SI 44/6 OPEN AND SI 2A/B CLOSE OR MANUALLY DO SO.

2. WHEN LEVEL GOES BELOW 10% TURN OFF BORIS ACTO TARK HEATER.

REVISION 0476 7-06-84



PLAN-VERTICAL PANELS AND CONSOLES

ATTACHMENT 9 Control Room Layout

BOY EXH FAN IA

TRIP BOT RESET

PuP in A.

PA PEC FAR IA

THIP NOT REBET

atart in Sta

SOCA IN OLT

RUS! TIST INLT

NY 5 200 11

WILLIT ON OL

(in the second sec

1

CAT IN MEN

i.

SOV EXH FAN P

THIP NOT RESET

BOPIC ACID The

OUT OF SERVICE

PA BEC FAN IB

18 1 83 - 41611

AC 111 18

DSC ... WY CL

ABST TEST INLT

VLV \$1 298 CL

Statt Hill He

WILLET CV AL

2 LADA A/6 48

(A) III OPEN

VLY BU KA OL

N SEV THE N.

BATT BOOMS EUH

FANS STOP

AIR CLU FAB IA

18-P NOT HESET

BHR PRP SUCT

VIV & YODA CL

LP A COLD LES

VL # SJ 114 CL

80 184 OPT D

SI READY

000 10

VLV 100 100 CL

NOV REG FAIL IS

TRIP NOT RENET

AIR OLE FAR IS

18-P 101 RENET

BHE PHP SUCT

VL1 51 5000 CL

EP 8 (010 .15

-

THE NAME ADDRESS

THE OCT MENT

401 EDD 748 1

*** 80* 81 87

V PMP 19 1947

AT \$ 18 Q in in dist.

KI V 00 X A 44 B 49 ----WAT TO LE POL IT YE PLUT w wh

THT VLUE ADDRESS

THE HET NEXT

NET CHI THE M.

THIP NOT NEART

PRIME CUI FILL

THIP NOT ACLET

SUPURP & BUCT

SI COLE LES

IN OLT W IS THEPHEL. IN AN THEMAN P LOOS HE VENT VIN WIT NER ME TRATE PUERLANE THIPPER -----CAT VLVB OPEN

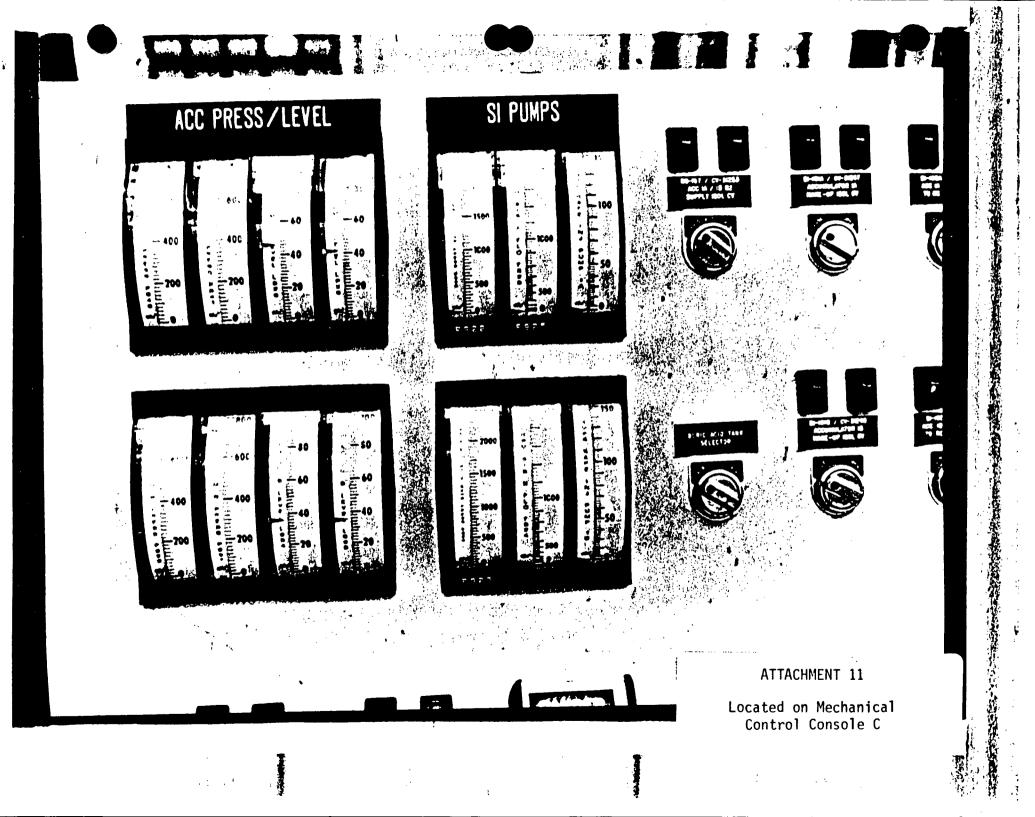
ATTACHMENT 10

Located on Mechanical Vertical Panel C

の「東京の国家

たちない

No. of Street



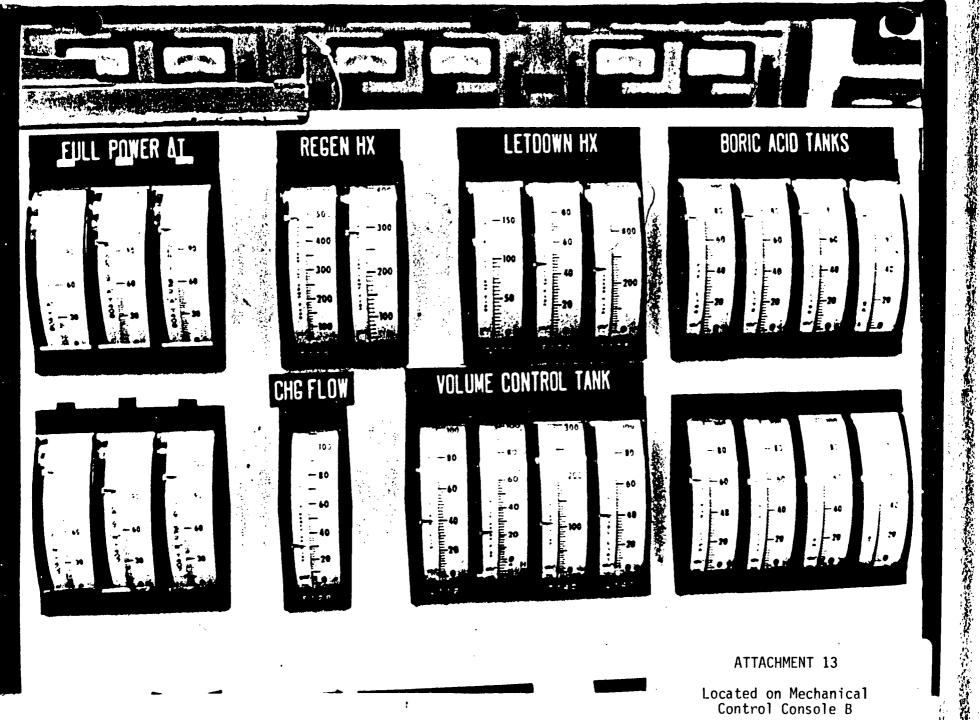
OL ACTIVE 1725-1 The state of the state **10 100 10** ----1.14.1 110.7 -----.... ALL STORES -e ia أنتثر حزب . . 140 Yo 110 -1. 1250 • , - 相信性 **ATTACHMENT 12** Located on Mechanical Vertical Panel C

Station of the state of the sta

NO POST

Party Party

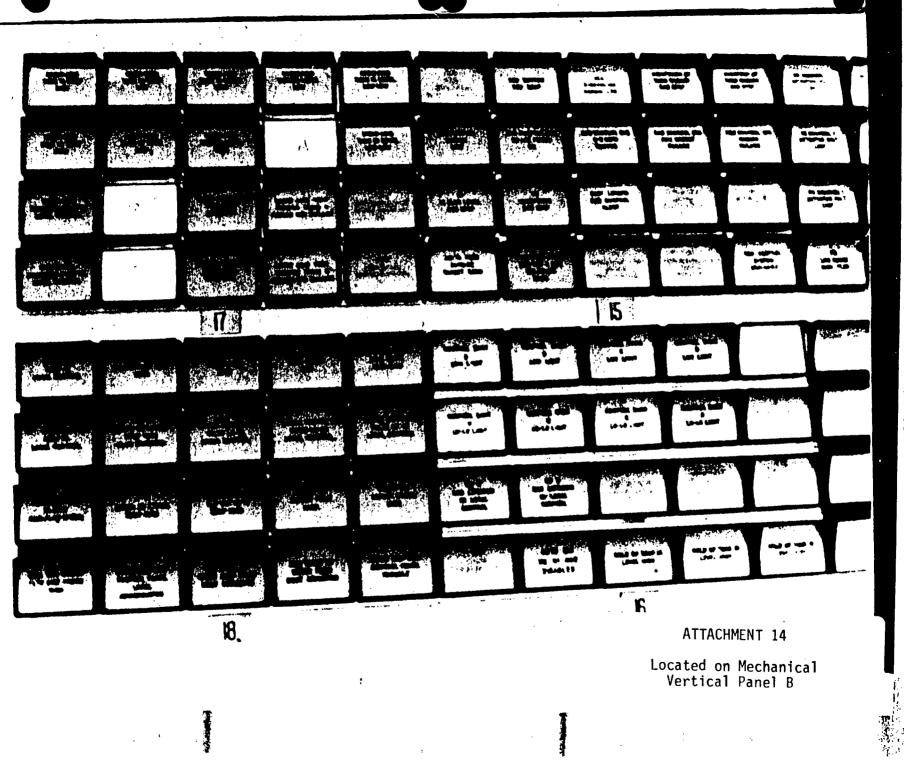
PLAN PLAN PLAN PLAN



÷.

Located on Mechanical Control Console B

÷



.

ŧ.

の方法があるのないであった。ためにないのないで、

il.

