



June 28, 2011
NND-11-0249

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

ATTN: Document Control Desk

Subject: Virgil C. Summer Nuclear Station (VCSNS) Units 2 and 3 Combined License Application (COLA) - Docket Numbers 52-027 and 52-028 Submittal of Roadmap of Changes for the June 28, 2011 Submittal of the COLA.

Reference: Letter from Ronald B. Clary (SCE&G) to Document Control Desk (NRC), June 28, 2011 Submittal of the Combined License Application for V.C. Summer Nuclear Station Units 2 and 3, Including Revision 5 of the Final Safety Analysis Report (FSAR), NND-11-0244.

By letter dated March 27, 2008, South Carolina Electric & Gas Company (SCE&G) submitted a combined license application (COLA) for two Westinghouse AP1000 units, designated V.C. Summer Nuclear Station (VCSNS) Units 2 and 3, to be located at the existing VCSNS site in Fairfield County, South Carolina. By letter dated June 28, 2011, SCE&G submitted an update to the VCSNS Units 2 and 3 COLA. Enclosed is a 'roadmap' of changes included in the June 28, 2011 update, which is being provided as a reviewer aid. Also provided is an explanation of the roadmap column headings.

Should you have any questions, please contact me by telephone at (803) 345-4191, or by email at apaglia@scana.com.

Very truly yours,

Alfred M. Paglia
Manager, Nuclear Licensing
New Nuclear Deployment

JIG/AMP/jg

D083
NRO

Enclosure – COLA June 28, 2011 Submittal Roadmap

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NND-11-0249

Enclosure

VCSNS Units 2 and 3 COL Application

June 28, 2011 Submittal Roadmap

(13 Pages Including Coversheet and Column Index Page)

**VCSNS Units 2 & 3 COL Application June 28, 2011 Submittal Roadmap Format
Explanation (by columns)**

Column Label	Explanation
Change ID #	Unique internal identifier for tracking purposes
COLA REP	Identifies the change as STD (standard) or VCSNS (VCS) specific
COLA Part REP	Part 1 (PT01) through Part 18 (PT18)
Chapter REP	FSAR Chapter
Section / Page REP	Identifies specific location of the change in the COLA. Page numbers, if identified, are specific to the document that was revised, i.e., Revision 5.
Complete Change Description	Short description of the change
Basis for Change	The source of the change

NuStart's COLA Tracking Management (CTM) : COLA Changes | Summer Rev 5

JUN-28-2011 10:36 AM

Summer Rev 5

Due Date is '04-01-2011' AND ...

QB Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
PT01 - (empty)						5 COLA Changes
9232	VCS	PT01		COLA Part 1 Section 1.0	COLA Part 1, Section 1.0, last sentence in the second paragraph will be revised to read (note, update to Revision 19 of the DCD): "This application presents descriptions and analyses of the final station design and incorporates by reference, Appendix D to 10 CFR Part 52 as required by Section III.B of that Appendix as amended by Revision 19."	DCD Revision 19 conforming change. Updated Revision 18 to Revision 19.
9991	VCS	PT01		COLA Part 1 Section 1.0	COLA Part 1, Section 1.0, list of parts of the application shall be revised to add the following: Part 17 - COLA Enclosure 7 – New Fuel Shipping Plan Part 18 - COLA Enclosure 8 – Supplemental Information in Support of 10 CFR Part 70 Special Nuclear Material License Application	Conformance with Letter NND-11-0174 and NND-11-0212 which added Part 17 and Part 18 to the COLA.
9453	VCS	PT01		COLA Part 1 Section 1.1	COLA Part 1, Section 1.1, Purpose of a Combined License Application, paragraphs 4 and 6 will be revised as follows: It is requested that the term of the license be for a period of 40 years, and contain provisions, as needed, to possess and use at any time such quantities of source, byproduct, and special nuclear material as needed to construct and operate the utilization facility. Special nuclear material in the form of reactor fuel and spent fuel shall be in accordance with limitations for storage and amounts required for reactor operation, as described in Part 2 of this application. Byproduct, source, and special nuclear material shall be in the form of sealed neutron sources for reactor startup and sealed sources for reactor instrumentation, radiation monitoring equipment, calibration, and fission detectors in amounts as required. In preparation for the initial fuel loading, limitations on byproduct material and Part 40 specifically licensed source material will be as described in this application. Following the 52.103(g) finding, byproduct, source, and special nuclear material in amounts as required, without restriction to chemical or physical form, shall be for sample analysis, instrument and equipment calibration, or associated with radioactive apparatus or components.	RAI Number VEGP 12.02-VR1 SNC Ltr ND-11-0435, (SCE&G Endorsement Letter NND-11-0062)
9454	VCS	PT01		COLA Part 1 Section 1.1	Revise Change ID# 9453 to remove comma after "spent fuel" in the 2nd sentence.	Editorial Correction to SCE&G Endorsement Letter NND-11-0062
9455	VCS	PT01		COLA Part 1 Table 1	Revise position title for Stephen Byrne from Executive Vice President Nuclear Operations and Chief Nuclear Officer, SCE&G to Executive Vice President, Generation and Chief Nuclear Officer, SCE&G	Consistency with position titles in other parts of the COLA.
PT02 - FSAR01						14 COLA Changes
9234	VCS	PT02	FSAR01	01.01	COLA Part 2, FSAR Chapter 1, Section 1.1, Introduction, will be revised from: Throughout this FSAR, the "referenced DCD" is the AP1000 DCD submitted by Westinghouse as Revision 18 including any supplemental material as identified in Table 1.6-201. To read: Throughout this FSAR, the "referenced DCD" is the AP1000 DCD submitted by Westinghouse as Revision 19 including any supplemental material as identified in Table 1.6-201.	DCD Rev 19
9300	STD,VCS	PT02	FSAR01	01.01	1. COLA Part 2 (Revision 4), FSAR Chapter 1, Section 1.1, Introduction, will be revised from: Unless otherwise specified, reference to the DCD refers to Tier 2 information and includes the sensitive unclassified non-safeguards information (including proprietary information), and safeguards information referenced in the AP1000 DCD. Such DCD information is included in this combined license application in the same manner as it is included in the AP1000 DCD, i.e., references in the DCD are included as references in the FSAR, and material incorporated by reference into the DCD is incorporated by reference into the FSAR. Appropriate agreements are in	VEGP letter ND-11-0254 - (SCE&G Endorsement Letter NND-11-0060)

QB Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					place to provide access to the withheld sensitive unclassified non-safeguards information (including proprietary information), and safeguards information referenced in the AP1000 DCD. To read: Unless otherwise specified, reference to the DCD refers to Tier 2 information, including references to the sensitive unclassified non-safeguards information (including proprietary information) and safeguards information, contained in the AP1000 DCD. Such DCD information is included in this combined license application in the same manner as it is included in the AP1000 DCD, i.e., references in the DCD are included as references in the FSAR, and material incorporated by reference into the DCD is incorporated by reference into the FSAR. Appropriate agreements are in place to provide for the licensee's rights to possession (including constructive possession) and use of the withheld sensitive unclassified non-safeguards information (including proprietary information) and safeguards information referenced in the AP1000 DCD for the life of the project.	
9444	STD,VCS	PT02	FSAR01	01.01	COLA Part 2, FSAR Chapter 1, Section 1.1, Introduction, will be revised to include the following second paragraph under LMA STD SUP 1.1-1: Appendix D to 10 CFR Part 52 is hereby incorporated by reference into the COL application.	DCD Rev 19 and final design certification
9233	VCS	PT02	FSAR01	01.06.T / T1.6-201	COLA Part 2, FSAR Chapter 1, Section 1.6, Table 1.6-201, will be revised from: Westinghouse/APP-GW-GL-700 AP1000 Design Control Document 18 All December 2010 ML103480572 To read: Westinghouse/APP-GW-GL-700 AP1000 Design Control Document 19 All June 2011 ML11171A500	DCD Rev 19
9493	STD,VCS	PT02	FSAR01	01.06.T / T1.6-201	COLA Part 2, FSAR Chapter 1, Section 1.6, Table 1.6-201, will be revised to add a new line item under STD SUP 1.6-1: 10CFR Part 52 Design Certification Rule -- 1.1 -- -- Appendix D for the AP1000 Design	DCD Rev 19 and final design certification
9235	VCS	PT02	FSAR01	01.06.T/1.6-201	In Table 1.6-201, change the ML Number of the VCSNS Emergency Plan to 'ML110410260'	Update of ML Number with revised Part 5 update for COLA Rev 4 submittal
10022	VCS	PT02	FSAR01	01.06.T/1.6-201	In Table 1.6-201, change the Revision and Document Transmittal Date of the Cyber Security Plan to "1" and "June 2011", respectively. Also, change font of Cyber Security Plan to match rest of the table.	Update of Table to reflect latest revision and transmittal date of Cyber Security Plan. Date of Cyber Plan Revision 1 is June 2011.
9262	STD,VCS	PT02	FSAR01	01.09.01	1. COLA Part 2, FSAR Chapter 1, Section 1.9.1, third paragraph will be revised from (retain the left margin annotation (LMA) of STD COL 1.9-1): Division 5 of the regulatory guides applies to the Physical Security Plan and the topics are addressed in the Physical Security Plan. Three Division 5 Regulatory Guides are addressed in Appendix 1AA. To read: Division 5 of the regulatory guides applies to materials and plant protection. As appropriate, the Division 5 regulatory guide topics are addressed in the DCD and plant-specific security plans (i.e., Physical Security Plan, Training and Qualification Plan, Safeguards Contingency Plan, and Cyber Security Plan).	VEGP-VOL-CH01 Div 5 RGs in response to VEGP 01 VR2 item 1 SNC Ltr ND-11-0261 (SCE&G Endorsement Letter NND-11-0049)

QB Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
9265	STD,VCS	PT02	FSAR01	01.09.01.03	<p>4. COLA Part 2, FSAR Chapter 1, Section 1.9.1.3, Division 5 Regulatory Guides - Materials and Plant Protection, will be revised from (retain the LMA of STD COL 1.9-1):</p> <p>Division 5 of the regulatory guides applies to the Physical Security Plan and the topics addressed in the Physical Security Plan. Appendix 1AA provides an evaluation of the degree of conformance with Division 5 regulatory guides as applicable to the content of this FSAR, or to the site-specific design, construction and/or operational aspects. The revisions of the regulatory guides against which the plant is evaluated are indicated. Any exceptions or alternatives to the provisions of the regulatory guides are identified and justification is provided. One such general alternative is the use of previous revisions of the Regulatory Guide for design aspects as stated in the DCD in order to preserve the finality of the certified design (see Notes at the end of Appendix 1AA). The cross-referenced sections contain descriptive information applicable to the regulatory guide positions found in Appendix 1AA.</p> <p>To read:</p> <p>Division 5 of the regulatory guides applies to materials and plant protection. Appendix 1AA provides an evaluation of the degree of conformance with Division 5 regulatory guides as applicable to the content of the AP1000 DCD and the plant-specific Cyber Security Plan. The plant-specific physical security plans (i.e., Physical Security Plan, Training and Qualification Plan, and Safeguards Contingency Plan) were developed using the template in NEI 03-12, Revision 6, "Template for the Security Plan, Training and Qualification Plan, Safeguards Contingency Plan [and Independent Spent Fuel Storage Installation Security Program]," which was endorsed for use by NRC letter dated April 9, 2009. The plant-specific physical security plans include no substantive deviations from the NRC-endorsed template in NEI 03-12, Revision 6. Therefore, the degree of conformance with Division 5 regulatory guides for the plant-specific physical security plans is consistent with the degree of conformance of NEI 03-12, Revision 6.</p>	VEGP-VOL-CH01 Div 5 RGs in response to VEGP 01 VR2 item 4 SNC Ltr ND-11-0261 (SCE&G Endorsement Letter NND-11-0049)
9263	STD,VCS	PT02	FSAR01	01.09.01.T / T1.9-201	<p>2. COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Regulatory Guide/FSAR Section Cross-References, will be revised (retain the LMA of STD COL 1.9-1) to omit specific entries for RG 5.9, 5.12, and 5.65, and replace them with [where # is the next appropriate Note number]:</p> <p>Division 5 Regulatory Guides Note #</p>	VEGP-VOL-CH01 Div 5 RGs in response to VEGP 01 VR2 item 2 SNC Ltr ND-11-0261 (SCE&G Endorsement Letter NND-11-0049)
9264	STD,VCS	PT02	FSAR01	01.09.01.T / T1.9-201	<p>3. COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Regulatory Guide/FSAR Section Cross-References, will be revised by adding Note # at the end of the table (where # is the next sequential note identifier), with an LMA of STD COL 1.9-1, as follows:</p> <p>#. Division 5 of the regulatory guides applies to materials and plant protection. As appropriate, the Division 5 regulatory guide topics are addressed in the DCD and plant specific security plans (i.e., Physical Security Plan, Training and Qualification Plan, Safeguards Contingency Plan, and Cyber Security Plan).</p>	VEGP-VOL-CH01 Div 5 RGs in response to VEGP 01 VR2 item 3 SNC Ltr ND-11-0261 (SCE&G Endorsement Letter NND-11-0049)
9960	STD,VCS	PT02	FSAR01	01.09.T / T1.9-201 1.197	COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, the date for RG 1.197, will be revised from February 2003 to May 2003.	DCD Rev 17
9244	STD,VCS	PT02	FSAR01	01AA 1.082	<p>COLA Part 2, FSAR Chapter 1, Appendix 1AA, Regulatory Guide 1.82 is revised from:</p> <p>Conformance of the design aspects with Revision 2 of this Regulatory Guide is as stated in the DCD. Conformance with Revision 3 of this Regulatory Guide for programmatic and/or operational aspects is documented below.</p> <p>To read:</p> <p>Conformance with the design aspects is as stated in the DCD. Conformance with programmatic and/or operational aspects is documented below.</p>	DCD Rev 19
9266	STD,VCS	PT02	FSAR01	APP 1AA	<p>5. COLA Part 2, FSAR Appendix 1AA, Conformance with Regulatory Guides, will be revised (retain the LMA of STD COL 1.9-1) to include the following new introductory information and an appropriately located compliance statement for RG 5.71.</p> <p>The plant-specific physical security plans include no substantive deviations from the NRC-endorsed template in NEI 03-12, Rev. 6. Therefore, the degree of conformance with Division 5 regulatory guides for the Physical Security Plan, Training and Qualification Plan, and Safeguards Contingency Plan is consistent with the degree of conformance of NEI 03-12, Rev. 6.</p> <p>-----</p> <p>Regulatory Guide 5.71, Rev. 0, 1/10 - Cyber Security Programs for Nuclear Facilities</p>	VEGP-VOL-CH01 Div 5 RGs in response to VEGP 01 VR2 item 5 SNC Ltr ND-11-0261 (SCE&G Endorsement Letter NND-11-0049)

QB Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					Conformance with regulatory positions C.1 through C.5 of Regulatory Guide 5.71, Rev. 0, is as stated in the Cyber Security Plan (CSP), with exceptions to the guidance as noted in Attachment A of the CSP.	
PT02 - FSAR03					4 COLA Changes	
9315	STD,VCS	PT02	FSAR03	03.05.01.03	COLA Part 2, FSAR Chapter 3, Subsection 3.5.1.3 will be revised from: In addition, the reinforced concrete shield building and auxiliary building walls, roofs, and floors, provide further conservative, inherent protection of the safety-related SSCs from a turbine missile. To read: In addition, the shield building and auxiliary building walls, roofs, and floors, provide further conservative, inherent protection of the safety-related SSCs from a turbine missile.	DCD R18
9535	STD,VCS	PT02	FSAR03	03.09.06.02.02	Revise the last sentence of the third bulleted paragraph following the paragraph in FSAR Section 3.9.6.2.2 containing subheading "Other Power-Operated Valve Operability Tests" from: • Periodic static testing is performed, at a minimum on high risk (high safety significance) valves, to identify potential degradation, unless those valves are periodically cycled during normal plant operation, under conditions that meet or exceed the worst case operating conditions within the licensing basis of the plant for the valve, which would provide adequate periodic demonstration of AOV capability. If required, based on valve qualification or operating experience, periodic dynamic testing is performed to re-verify the capability of the valve to perform its required functions. To read: • Periodic static testing is performed, at a minimum on high risk (high safety significance) valves, to identify potential degradation, unless those valves are periodically cycled during normal plant operation, under conditions that meet or exceed the worst case operating conditions within the licensing basis of the plant for the valve, which would provide adequate periodic demonstration of AOV capability. If required based on valve qualification or operating experience, periodic dynamic testing is performed to re-verify the capability of the valve to perform its required functions.	COL-SER-OI-Ch03 response to OI 03.09-04 (SNC Ltr ND-09-2015) and updated in SNC Item 03.09-04 VR (SNC Ltr ND-10-0393 dated Mar 1, 2010) endorsed in NND-10-0317. Editorial correction identified in NND-11-0154.
8746	VCS	PT02	FSAR03	3.09.06.02.02	Revise the 4th insertion statement to remove the entire inserted statement "Power-Operated Valve Operability Tests - The safety-related, power-operated valves (POVs) are required by the procurement specifications to have the capabilities to perform diagnostic testing to verify the capability of the valves to perform their design basis safety functions."	Removal of insertion of existing DCD text from inserted FSAR text
9980	VCS	PT02	FSAR03	3.09.06.03	Revise the last sentence of the 4th paragraph from: "OMN-1 Revision 1 incorporates the guidance on risk-informed testing of MOVs from OMN-1, "Risk-Informed Testing of Motor-Operated Valves," and provides additional guidance on design basis verification testing and functional margin, which eliminates the need for the figures on functional margin and test intervals in Code Case OMN-1." To Read: "OMN-1 Revision 1 incorporates the guidance on risk-informed testing of MOVs from OMN-11, "Risk-Informed Testing of Motor-Operated Valves," and provides additional guidance on design basis verification testing and functional margin, which eliminates the need for the figures on functional margin and test intervals in Code Case OMN-1."	Errata from Revision 3.
PT02 - FSAR04					1 COLA Change	
9494	STD,VCS	PT02	FSAR04	04.04.07	COLA Part 2, FSAR Chapter 4, Subsection 4.4.7, is revised from: Replace the second paragraph in DCD Subsection 4.4.7 with the following: To read: Replace the paragraph in DCD Subsection 4.4.7.2 with the following:	DCD Rev 17
PT02 - FSAR06					2 COLA Changes	

QB Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change									
9516	STD,VCS	PT02	FSAR06	06.01.02.01.06	<p>Section 6.1.2.1.6 2nd Paragraph:</p> <ul style="list-style-type: none"> 1st sentence, comma should be included after "phase" 2nd sentence, "coating" should read "coatings" <p>Section 6.1.2.1.6 3rd Paragraph:</p> <ul style="list-style-type: none"> 2nd sentence, word "inspection" is missing after "application," 3rd sentence, "consolidated plant coating program" should be "consolidated plant coatings program" 3rd sentence, word "inspection" should be removed after "application," <p>Section 6.1.2.1.6 4th Paragraph:</p> <ul style="list-style-type: none"> 1st sentence, word "Coating" is missing from document title (Reference 202) <p>Section 6.1.2.1.6 8th Paragraph:</p> <ul style="list-style-type: none"> 1st sentence, word and comma "application," is missing 	Editorial changes to be consistent with SNC Ltr ND-10-1264 as endorsed in SCE&G Ltr NND-10-0318. Changes identified in new letter NND-11-0154.									
9378	STD,VCS	PT02	FSAR06	06.04.04	COLA Part 2, FSAR Chapter 6, Subsection 6.4.4, will be revised to remove LMA of STD SUP 6.4-# such that LMAs match those associated with Table 6.4-201. [For VCS, this is removal of STD SUP 6.4-2.]	Consistency with removal of SUP LMA from Table 6.4-201									
PT02 - FSAR12						1 COLA Change									
9316	STD,VCS	PT02	FSAR12	12.02.01.01.10	<p>2. COLA Part 2, FSAR, Chapter 12, Section 12.2.1.1.10, Miscellaneous Sources, seventh paragraph, Item b, will be revised to read:</p> <p>b) No 10 CFR Part 40 specifically licensed source material, including natural uranium, depleted uranium and uranium hexafluoride will be received, possessed, or used during this period.</p> <p>The following radioactive sources will be used for the Radiation Monitoring System and laboratory/portable monitoring instrumentation: 1</p> <table border="1"> <thead> <tr> <th>Radioactive Licensee Material (Element and Mass Number)¹</th> <th>Chemical and/or Physical Form¹</th> <th>Maximum Quantity That Licensee May Possess at Any One Time¹</th> </tr> </thead> <tbody> <tr> <td>• Any byproduct material with atomic numbers 1 through 93 inclusive</td> <td>Sealed Sources 2</td> <td>No single source to exceed 100 millicuries 5 Curies total</td> </tr> <tr> <td>• Americium-241</td> <td>Sealed Sources 2</td> <td>No single source to exceed 300 millicuries 500 millicuries total</td> </tr> </tbody> </table> <p>Notes: 1. This information remains in effect between the issuance of the COL and the Commission's 52.103(g) finding for each unit, and will be designated historical information after that time. 2. Includes calibration and reference sources.</p>	Radioactive Licensee Material (Element and Mass Number) ¹	Chemical and/or Physical Form ¹	Maximum Quantity That Licensee May Possess at Any One Time ¹	• Any byproduct material with atomic numbers 1 through 93 inclusive	Sealed Sources 2	No single source to exceed 100 millicuries 5 Curies total	• Americium-241	Sealed Sources 2	No single source to exceed 300 millicuries 500 millicuries total	RAI Number VEGP 12.02-VR1 SNC Ltr ND-11-0435, (SCE&G Endorsement Letter NND-11-0062)
Radioactive Licensee Material (Element and Mass Number) ¹	Chemical and/or Physical Form ¹	Maximum Quantity That Licensee May Possess at Any One Time ¹													
• Any byproduct material with atomic numbers 1 through 93 inclusive	Sealed Sources 2	No single source to exceed 100 millicuries 5 Curies total													
• Americium-241	Sealed Sources 2	No single source to exceed 300 millicuries 500 millicuries total													
PT02 - FSAR13						4 COLA Changes									
9317	STD,VCS	PT02	FSAR13	13.04.T / T13.04-201 15	<p>Program, will be revised by moving the entry for portions applicable to radioactive material above the line for "Physical Security Program," and revising the portions applicable to radioactive material entry from: (retain left margin annotation (LMA) of STD 13.4-1): Refer to letter for changes to Table 13.4-201 for "from" text</p> <p>To read: Program Title: Physical Protection Program (applicable to protection of special nuclear material prior to the protected area being declared operational) Program Source (Required by): 10 CFR 73.1, 10 CFR 73.67 FSAR Section: 13.5.2.2.8, 13.6 Implementation Milestone: Prior to initial receipt of special nuclear material Implementation Requirement: 10 CFR 73.1(a), 10 CFR 73.67</p>	Letter VEGP-P02-VOL-SNMPPP-20110225 response to VEGP 13.06 VR2, SNC Letter ND-11-0313 (SCE&G Endorsement Letter NND-11-0061)									

QB Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
10030	VCS	PT02	FSAR13	13.04.T / T13.04-201 22	Table 13.4-201, Item 22, Program Source shall be changed from: "10 CFR 74, Subpart B (§§ 74.11 – 74.19, excl. § 74.17)" To read: "10 CFR 74, Subpart B (§§ 74.11 – 74.19, excl. § 74.17)"	Editorial correction to properly incorporate VEGP-RAI-LTR-064 response to RAI 01.05-003 item 2 SNC Ltr ND-10-2257 (SCE&G Endorsement NND-10-0448)
9318	STD,VCS	PT02	FSAR13	13.05.02.02.08	2. COLA Part 2, FSAR, Section 13.5.2.2.8, Security Procedures, will be revised by adding the following text after the current text in this section (retain LMA of STD COL 13.5-1): The Special Nuclear Material (SNM) Physical Protection Program describes the 10 CFR Part 70 required protection program in effect for the period of time during which new fuel as SNM is received and stored in a controlled access area (CAA), in accordance with the requirements of 10 CFR 73.67.	Letter VEGP-P02-VOL-SNMPPP-20110225 response to VEGP 13.06 VR2, SNC Letter ND-11-0313, (SCE&G Endorsement Letter NND-11-0061)
9961	STD,VCS	PT02	FSAR13	13.05.02.02.08	1. COLA Part 2, FSAR, Section 13.5.2.2.8, Security Procedures, will be revised by inserting the following paragraph at the end of this subsection, following the new paragraph that was provided regarding protection of Special Nuclear Material (SNM) prior to 10 CFR 73.55 implementation (retain left margin annotation (LMA) of STD COL 13.5-1): The New Fuel Shipping Plan addresses the applicable 10 CFR 73.67 requirements in the event that unirradiated new fuel assemblies or components are returned to the supplying fuel manufacturer(s) facility.	VEGP-RAI-LTR-065 S1 in response to RAI 13.06-37-VR1 item 1 SNC Ltr ND-11-0894 (Endorsed in SCE&G Ltr NND-11-0174)
PT02 - FSAR15						2 COLA Changes
9255	STD,VCS	PT02	FSAR15	15.00.03.02	COLA Part 2 (Rev. 4), FSAR Chapter 15, Subsection 15.0.3.2, will be revised from: The plant operating instrumentation selected for feedwater flow measurement is a Caldon [Cameron] LEFM CheckPlus™ System (Reference 201). This selected plant operating instrumentation has documented instrumentation uncertainties to calculate a power calorimetric uncertainty that confirms the 1% uncertainty assumed for the initial reactor power in the safety analysis bounds the calculated calorimetric power measurement uncertainty values. This calculated calorimetric is done in accordance with a previously accepted Westinghouse methodology (Reference 202). Administrative controls implement maintenance and contingency activities related to the power calorimetric instrumentation. To read: The plant operating instrumentation selected for feedwater flow measurement is a Caldon [Cameron] LEFM CheckPlus System (Reference 201), which will be calibrated (in a certified laboratory using a piping configuration representative of the plant piping design) prior to installation and will be tested after installation in the plant in accordance the LEFM CheckPlus commissioning procedure. This selected plant operating instrumentation has documented instrumentation uncertainties to calculate a power calorimetric uncertainty that confirms the 1% uncertainty assumed for the initial reactor power in the safety analysis bounds the calculated calorimetric power uncertainty values. The calculated calorimetric is done in accordance with a previously accepted Westinghouse methodology (Reference 202). Administrative controls implement maintenance and contingency activities related to the power calorimetric instrumentation.	VEGP-VOL-CH 15 ACRS response to OI 15.00-001 SNC Ltr ND-11-0253 (SCE&G Endorsement Letter NND-11-0048)
9304	STD,VCS	PT02	FSAR15	15.00.03.02	COLA Part 2 (Rev. 4), FSAR Chapter 15, Subsection 15.0.3.2, will be revised from: The plant operating instrumentation selected for feedwater flow measurement is a Caldon [Cameron] LEFM CheckPlus System (Reference 201), which will be calibrated (in a certified laboratory using a piping configuration representative of the plant piping design) prior to installation and will be tested after installation in the plant in accordance the LEFM CheckPlus commissioning procedure. To read: The plant operating instrumentation selected for feedwater flow measurement is a Caldon [Cameron] LEFM CheckPlus System (Reference 201), which will be calibrated (in a certified laboratory using a piping configuration representative of the plant piping design) prior to installation and will be tested after installation in the plant in accordance with the LEFM CheckPlus commissioning procedure.	Editorial revision to VEGP-VOL-CH 15 ACRS response to OI 15.00-001 SNC Ltr ND-11-0253 (SCE&G Endorsement Letter NND-11-0048)

QB Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
PT04 - (empty)						7 COLA Changes
9248	STD,VCS	PT04		A.2-03.03.01	COLA Part 4, Section A, will be revised to delete the item for GTS 3.3.1.	DCD Rev 18
9247	STD,VCS	PT04		A.2-03.03.02	COLA Part 4, Section A, will be revised to delete the item for GTS 3.3.2.	DCD Rev 18
9246	STD,VCS	PT04		A.2-03.06.04	COLA Part 4, Section A, will be revised to delete the item for GTS 3.6.4.	DCD Rev 18
9251	STD,VCS	PT04		A.2-B03.03.01	COLA Part 4, Section A, will be revised to add a new item for GTS Bases 3.3.1 to read: GTS B3.3.1 The Bases for Specification 3.3.1 (Background) contain a Reviewer Note which addresses future confirmation of chosen setpoints. Remove the reviewer note in the PSTS. There is no replacement language. Justification: The reviewer's note information for this specification is deleted because it is not intended to be a part of technical specifications.	DCD Rev 18
9250	STD,VCS	PT04		A.2-B03.03.02	COLA Part 4, Section A, will be revised to add a new item for GTS Bases 3.3.2 to read: GTS B3.3.2 The Bases for Specification 3.3.2 (Background) contain a Reviewer Note which addresses future confirmation of chosen setpoints. Remove the reviewer note in the PSTS. There is no replacement language. Justification: The reviewer's note information for this specification is deleted because it is not intended to be a part of technical specifications.	DCD Rev 18
9249	STD,VCS	PT04		A.2-B03.06.08	COLA Part 4, Section A, will be revised to add a new item for GTS Bases 3.6.8 to read: GTS B3.6.8 The Bases for Specification 3.6.8 contain a bracketed Figure which addresses the anticipated time prior to coolant inventory boiling under various conditions. Remove the reviewer note in the PSTS. There is no replacement language. Justification: The information is applicable as provided since no major departures from the certified design have been taken.	DCD Rev 15
9245	STD,VCS	PT04		B, 00 All	COLA Part 4, Technical Specifications, will be revised to incorporate the AP1000 GTS changes identified by WEC and incorporated into DCD Revision 19.	DCD Rev 19
PT10 - (empty)						11 COLA Changes
10033	VCS	PT10		App B	Revise sentence underneath "Piping Design ITAAC" from: "The ITAAC for Piping Design are included in attached 3.8-3." To read: "The ITAAC for Piping Design are included in attached Table 3.8-3."	Editorial change to ensure consistency with other sentences in section.
10032	VCS	PT10		App B, Table 3.8-1	Change ITA and acceptance criteria in EP ITAAC 1.1 from "control rooms" to "control room." Change EP program elements and acceptance criteria in EP ITAAC 3.1 from "control rooms" to "control room." Change EP program elements and acceptance criteria in EP ITAAC 3.2 from "control rooms" to "control room." and change ITA from "...using ENS from each operating Control Room..." to read "...using ENS from the Control Room..." Change 5.1.1 ITA from "OSCs" to "OSC" Change EP ITAAC 5.1.4 acceptance criteria from "control rooms" to "control room." Change 5.1.6 Acceptance Criteria from "There is an OSC located inside each Unit. It is separate from the Control Room and within the Protected Area." To read "There is an OSC located inside the Unit that is separate from the Control Room and within the Protected Area." Change EP ITAAC 5.2.3 acceptance criteria from "control rooms" to "control room."	Change to ensure that ITAAC read as single unit criteria. If plural is retained, it may imply that both control rooms, etc need to have ITAAC completed prior to one being declared complete. Identified in Letter NND-11-0246.
10104	VCS	PT10		App B, Table 3.8-1	Change Acceptance Criteria in ITAAC 5.1.1 from: "See DCD Table 3.1-1 Item 1"	Change identified in Letter NND-11-0246 to add

QB Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					To read: "The TSC has at least 3000 ft ² of floor space." (Note that ft ² should contain a superscript "2")	consistency in accordance with EP and NRC SER for floor space for 40 people.
10241	VCS	PT10		App B, Table 3.8-1	Add new acceptance criteria 5.1.8 which reads: "5.1.8 A reliable and backup electrical supply is available for the TSC."	Discussions with NRC and identified in NND-11-0246.
9301	STD,VCS	PT10		LC#01	2. COLA Part 10 (Revision 4), Proposed License Condition #1, ITAAC, introductory statements will be revised from: There are several ITAAC identified in the COLA. Once incorporated into the COL, the regulations identify the requirements that must be met. The incorporation below includes the sensitive unclassified non-safeguards information (including proprietary information), and safeguards information referenced in the AP1000 DCD. Such DCD information is included in this combined license application in the same manner as it is included in the AP1000 DCD, i.e., references in the DCD are included as references in the FSAR, and material incorporated by reference into the DCD is incorporated by reference into the FSAR. Appropriate agreements are in place to provide access to the withheld sensitive unclassified non-safeguards information (including proprietary information), and safeguards information referenced in the AP1000 DCD. To read: There are several ITAAC identified in the COLA. Once incorporated into the COL, the regulations identify the requirements that must be met. The incorporation below includes references to the sensitive unclassified non-safeguards information (including proprietary information) and safeguards information, contained in the AP1000 DCD. Such DCD information is included in this combined license application in the same manner as it is included in the AP1000 DCD, i.e., references in the DCD are included as references in the FSAR, and material incorporated by reference into the DCD is incorporated by reference into the FSAR. Appropriate agreements are in place to provide for the licensee's rights to possession (including constructive possession) and use of the withheld sensitive unclassified non-safeguards information (including proprietary information) and safeguards information referenced in the AP1000 DCD for the life of the project.	SNC Letter ND-11-0254 (SCE&G Endorsement Letter NND-11-0060)
9319	STD,VCS	PT10		LC#03 C.5	4. COLA Part 10, Proposed License Conditions (Including ITAAC), LC #3.C, Operational Program Implementation, Receipt of Materials, Item C.5, will be revised, from: C.5 - Security Program (applicable portions) To read: C.5 - Deleted	Letter VEGP-P02-VOL-SNMPPP-20110225 response to VEGP 13.06 VR2, SNC Letter ND-11-0313 (SCE&G Endorsement Letter NND-11-0061)
9320	STD,VCS	PT10		LC#03 D.3	5. COLA Part 10, Proposed License Conditions (Including ITAAC), LC #3.D, Operational Program Implementation, Fuel Receipt, Item D.3, will be revised, from: D.3 - Security Program (applicable portions) To read: D.3 - Special Nuclear Material Physical Protection Program	Letter VEGP-P02-VOL-SNMPPP-20110225 response to VEGP 13.06 VR2, SNC Letter ND-11-0313 (SCE&G Endorsement Letter NND-11-0061)
9443	STD,VCS	PT10		LC#03 D.4	COLA Part 10, proposed License Condition 3, Operational Program Implementation, item D, Fuel Receipt, will be revised from: D.4 - Emergency Planning (applicable portions) To read: D.4 - Deleted	For consistency with changes included in VEGP-RAI-LTR-062 response to RAI 01.05-001 item 3 SNC Ltr ND-10-2002, (SCE&G Endorsement Letter NND-10-0440)
9963	STD,VCS	PT10		LC#03 E.2	COLA Part 10, Proposed License Conditions; item related to construction and inspection procedures will be revised to read (where # is the next appropriate letter): #. The implementation of construction and inspection procedures for steel concrete composite (SC) construction activities for seismic Category I nuclear island modules (including shield building SC modules) before and after concrete placement, and inspection of such construction before and after concrete placement.	VEGP-VOL-Ch03 Const response to STD COL 03.08-006 Supp SNC Ltr ND-11-0895 (SCE&G Ltr NND-11-0184)
10031	VCS	PT10		LC#03 H	COLA Part 10, Proposed License Conditions, Section 3.H should be revised from:	Change to conform with

QB Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					"The licensee shall implement each operational program identified below prior to initial fuel load." To read: "The licensee shall implement each operational program identified below prior to initial startup testing"	VEGP Proposed License Conditions.
9321	STD,VCS	PT10		LC#05	<p>3. COLA Part 10, Proposed License Conditions (Including ITAAC), proposed License Condition (LC) #5, will be revised by renumbering and renaming the current proposed LC and adding a new License Condition #5B. This revision will change proposed LC#5 from:</p> <p>5. SECURITY PROGRAM REVISIONS: An implementation license condition approved in the SRM regarding SECY-05-0197 applies to the security program. PROPOSED LICENSE CONDITION: The licensee shall maintain in effect the provisions of the physical security plan, security personnel training and qualification plan, safeguards contingency plan, and cyber security plan, and all amendments made pursuant to the authority of 10 CFR 50.90, 50.54(p), 52.97, and Section VIII of Appendix D to Part 52 when nuclear fuel is onsite (protected area), and continuing until all nuclear fuel is permanently removed from the site. To read: 5. SECURITY PROGRAM: A. SECURITY PROGRAM IMPLEMENTATION An implementation license condition approved in the SRM regarding SECY-05-0197 applies to the security program. PROPOSED LICENSE CONDITION: The licensee shall maintain in effect the provisions of the physical security plan, security personnel training and qualification plan, safeguards contingency plan, and cyber security plan, and all amendments made pursuant to the authority of 10 CFR 50.90, 50.54(p), 52.97, and Section VIII of Appendix D to Part 52 when nuclear fuel is onsite (protected area), and continuing until all nuclear fuel is permanently removed from the site. B. SPECIAL NUCLEAR MATERIAL PHYSICAL PROTECTION A license condition is proposed to address when the boundary for physical protection of new fuel as SNM is required to be extended from the controlled access area (CAA) in accordance with the requirements of 10 CFR 73.67 to the operational protected area (PA) in accordance with 10 CFR 73.55. PROPOSED LICENSE CONDITION The licensee shall receive and store new fuel as SNM in a controlled access area (CAA) in accordance with the requirements of 10 CFR 73.67, until such time as an operational protected area (PA) that satisfies the requirements of 10 CFR 73.55(e)(8) is established. If new fuel is already stored in a CAA that is within the boundary of the proposed PA, then upon declaration of an operational PA, the remaining requirements of 10 CFR 73.55 shall be implemented. The PA shall be established and declared operational prior to initial fuel load.</p>	Letter VEGP-P02-VOL-SNMPPP-20110225 response to VEGP 13.06 VR2, SNC Letter ND-11-0313 (SCE&G Endorsement Letter NND-11-0061)
Supp - (empty)						6 COLA Changes
10004	STD,VCS	Supp		COLA Part 15 - CSP 01	<p>1. COLA Part 15, Cyber Security Plan, Section 1, will be revised by adding a new paragraph after the first paragraph, as follows:</p> <p>Within the scope of the NRC's cyber security rule at 10 CFR 73.54, systems or equipment that perform important to safety functions include structures, systems, and components (SSCs) in the balance of plant (BOP) that could directly or indirectly affect reactivity at a nuclear power plant and could result in an unplanned reactor shutdown or transient. Additionally, these SSCs are under the licensee's control and include electrical distribution equipment out to the first inter-tie with the offsite distribution system.</p>	VEGP-VOL-CSP BOP SSCs response to VEGP 13.06 VR item 1 SNC Ltr ND-11-0207 (Endorsed in SCE&G Ltr NND-11-0033)
10026	VCS	Supp		COLA Part 15 - CSP Cover	Change the cover page of the VCSNS Cyber Security Plan to identify it as Revision 1, dated June 2011.	Update the Cyber Security Plan (CSP) cover page to reflect latest revision and transmittal date of CSP. Date of Cyber Security Plan Rev. 1 is June 2011.
10005	STD,VCS	Supp		COLA Part 15-CSP Att A	<p>2. COLA Part 15, Cyber Security Plan, Attachment A, "Virgil C. Summer Nuclear Station, Units 2 and 3 Cyber Security Plan (CSP) Deviations from Regulatory Guide (RG) 5.71, Rev. 0," will be revised by adding a new deviation # (where # is the next sequential number), as follows:</p> <p>Deviation No. 18: Include Balance of Plant structures, systems, and components to the scope defined by important to safety - A deviation is taken to the guidance to clarify that systems or equipment that perform important to safety functions include structures, systems, and components (SSCs) in the balance of plant (BOP) that could directly or indirectly affect reactivity and could result in an unplanned reactor shutdown or transient.</p>	VEGP-VOL-CSP BOP SSCs response to VEGP 13.06 VR item 2 SNC Ltr ND-11-0207 (Endorsed in SCE&G Ltr NND-11-0033)

QB Change ID#	COLA REP	COLA Part REP	Chapter REP	Section/Page REP	Complete Change Description	Basis for Change
					Basis: This change is consistent with Commission's position provided by the Director of the Division of Security Policy in the NRC's Office of Nuclear Security and Incident Response to the Nuclear Energy Institute Director of Security by letter dated January 5, 2011.	
10006	STD,VCS	Supp		COLA Part 15-CSP Att A	<p>COLA Part 11C, Cyber Security Plan, Attachment A, "Vogtle Electric Generating Plant, Units 3 and 4 Cyber Security Plan (CSP) Deviations from Regulatory Guide (RG) 5.71, Rev. 0," table titled "VEGP Units 3 and 4 Cyber Security Plan Deviations from RG 5.71, Rev. 0 Black Text," will be revised by adding a deviation after the current deviation to RG 5.71, Appendix A, Heading (Page A-1), as follows:</p> <p>Reference: RG 5.71, Section A.1, 1st paragraph (Page A-1)</p> <p>RG 5.71 Rev. 0 Text: "... up to and including the design-basis threat (DBT) described in 10 CFR 73.1, "Purpose and Scope": * safety-related and important-to-safety functions, * security functions, * emergency preparedness functions, including offsite communications, and * support systems and equipment which, if compromised, would adversely impact safety, security, or emergency preparedness functions."</p> <p>VEGP Units 3 and 4 CSP Text: "... up to and including the design-basis threat (DBT) described in 10 CFR 73.1, "Purpose and Scope": * safety-related and important-to-safety functions, * security functions, * emergency preparedness functions, including offsite communications, and * support systems and equipment which, if compromised, would adversely impact safety, security, or emergency preparedness functions."</p> <p>Within the scope of the NRC's cyber security rule at 10 CFR 73.54, systems or equipment that perform important to safety functions include structures, systems, and components (SSCs) in the balance of plant (BOP) that could directly or indirectly affect reactivity at a nuclear power plant and could result in an unplanned reactor shutdown or transient. Additionally, these SSCs are under the licensee's control and include electrical distribution equipment out to the first inter-tie with the offsite distribution system.</p> <p>Comments: Deviation No. 18: Include Balance of Plant structures, systems, and components to the scope defined by important to safety.</p> <p>Addition of this paragraph clarifies the intent of important-to-safety function, consistent with direction provided in a January 5, 2011, letter from Richard P. Correia (NRC) to Chris Earls (NEI) on this subject.</p>	VEGP-VOL-CSP BOP SSCs response to VEGP 13.06 VR item 3 SNC Ltr ND-11-0207. (Endorsed in SCE&G Ltr NND-11-0033)
9962	STD,VCS	Supp		COLA Part 17	The COLA will be revised by the addition of a new Part 17, New Fuel Shipping Plan. Part 17 is provided as Enclosure 2 to SNC Ltr ND-11-0894 (Endorsed by NND-11-0174).	VEGP-RAI-LTR-065 S1 in response to RAI 13.06-37-VR1 item 2 SNC Ltr ND-11-0894 (Endorsed by NND-11-0174).
10007	STD,VCS	Supp		COLA Part 18	The COLA will be revised by the addition of a new Part 18, Supplemental Information in Support of 10 CFR Part 70 Special Nuclear Material License Application. Part 18 is provided as Enclosure 2 to SNC Ltr ND-11-1211 (Endorsed by NND-11-0212).	SNC Ltr ND-11-1211 (Endorsed by NND-11-0212).

SUMMARY

COLA Part REP	Chapter REP	Number of COLA Changes
PT01	<i>(empty)</i>	5
PT02	FSAR01	14
PT02	FSAR03	4
PT02	FSAR04	1
PT02	FSAR06	2
PT02	FSAR12	1
PT02	FSAR13	4
PT02	FSAR15	2
PT04	<i>(empty)</i>	7
PT10	<i>(empty)</i>	11
Supp	<i>(empty)</i>	6
TOTALS (11 groups)		57