

EPEI ELECTRIC POWER RESEARCH INSTITUTE

Materials Reliability Program





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Industry/NRC Executive Meeting on Materials Program

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MRP Organization





MRP Membership





MRP Strategic Plan

- Develop fundamental understanding of degradation mechanisms and identify materials (and locations) of high susceptibility
- Conduct generic operability and safety assessments for susceptible locations
- Develop Inspection and Evaluation (I&E) guidelines for identified locations
- Evaluate available mitigation options and, if necessary develop additional options.
- Evaluate repair/replace options, where necessary
- Monitor, evaluate and feedback plant Operating Experience
- Develop technical bases for regulatory review and support licensees on plant-specific applications







MRP Current Issues/Activities

- Stress Corrosion Cracking (SCC) of nickel-based alloys
- Reactor Internals Degradation Management
- Testing for Irradiation Assisted Degradation of Reactor Internals
- I&E guidelines (e.g. Thermal Fatigue, Reactor Internals)
- SCC of stainless steel including CASS
- Low Alloy Steel corrosion (upper and bottom RPV Heads)
- Non-Destructive Examination (NDE) qualifications (e.g. CRDM, RI standards)
- Fatigue degradation management including Environmental Fatigue
- Reactor Pressure Vessel Integrity
- Extremely Low Probability of Rupture (xLPR)
- More than 50 tasks defined and funded for 2011

MRP Leveraging and Collaborations

- All Reactor Internals testing projects (Zorita, Halden, Gondole)
- Stainless steel SCC research with MAI/EDF & PWROG

- Alloy A690 international collaboration for long-term leveraging on A690/52/152 research
- xLPR (with NRC RES)
- Weld residual stresses (NRC RES)
- Chemical Mitigation (PWROG, FRP, Chemistry)
- Surface stress mitigation methods with Hitachi, Toshiba and MHI
- Low K testing with Studsvik and AREVA
- Environmental Fatigue (BWRVIP, ANT, NRC RES and others)
- Inspection issues with EPRI NDE Center
- Information exchange agreement with Naval Reactors contractors

MRP Guidelines Status

Guideline Title	ITG/ TSC	NEI 03- 08 ¹¹¹	MRP Rpt #	Curren t Rev #	G/L Report #	Last Pub. Date	Next G/L Review Date	Next G/L Rev Pub Date ^[ii]
Mitigation of Thermal Fatigue in Unisolable Piping Connected to PWR Reactor Coolant Systems	TSC	GP 📖	29	0	1001017	Dec. 2000	2012	2012
Thermal Fatigue Monitoring Guidelines	TSC	GP ^[iy]	32	0	1001016	April 2001	-	June 2011
Computer-Based NDE Training for Thermal Fatigue Cracking	TSC	GP	36	1	1016935	Nov. 2008	-	2012
Guidelines for Addressing Fatigue Environmental Effects in a License Renewal Application	TSC	GP	47	1	1012017	Sept. 2005	-	2012
Generic Guidance for Alloy 600 Management	AST	Man.	126	0	1009561	Nov 2004	-	<u>[v]</u>
Primary System Piping Butt Welds Inspection and Evaluation Guidelines	AST	• Man.	139	1	1015009	Dec. 2008	-	<u> vi]</u>
Management of Thermal Fatigue in Normally Stagnant Non-Isolable Reactor Coolant System Branch Lines	TSC	Need	146	0	1011955 <u>[×</u> <u>ii]</u>	June 2005	-	June 2011
Management of Thermal Fatigue in Normally Stagnant Non-Isolable Reactor Coolant System Branch Lines – Supplemental Guidance	TSC	Need	146S	0	1018330	January 2009	2012	2013
Integrated Fatigue Management Guideline	TSC	GP	148	0	1012018	April 2005	-	2012
Thermal Fatigue Monitoring Guideline	TSC	GP	149	0	1011957	April 2005	2012	2013
Assessment of RHR Mixing Tee Thermal Fatigue in PWR Plants	TSC	GP	192	1 <u>[viii]</u>	1018395	Dec. 2008	-	2012
Materials Reliability Program: PWR Internals Inspection and Evaluation Guidelines	AST	Man.	227	0	1016596	Dec. 2008	-	2011
Materials Reliability Program: Inspection Standard for PWR Internals	Insp	Need	228	0	1016609	July 2009	-	2012

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MRP-227 Safety Evaluation Schedule

- Draft SE Issued on 28 March
- Draft Public Comment Period from 11 April to 10 May
- Industry Comments Transmitted on 10 May
 - Included Industry-proposed changes to the SE Conditions and Action Items for Staff consideration
- Teleconference Held on 26 May to Review Comment with Staff
- Final SE to be Issued on or about 24 June
- RIS to be Issued with Final SE
 - -Anticipated on or about 28 June
 - Guidance on AMP submittal/revision expectations for plants in different classifications







MRP-227-A

MRP-227-A will Incorporate Condition in Final SE

- -Will also include changes submitted with RAI responses
- Proposed Changes to Incorporate Conditions, As Modified, Submitted with Draft SE Comments
- MRP-227-A will be Issued on or before 31 December this Year
 - Publication will start the two-year clock on the Needed requirement to implement the tables in MRP-227-A.
- Revision to MRP-228 (Reactor Internals Inspection Standard)
 will be Prepared and Issued Next Year
 - Revision for consistency with MRP-227-A and to incorporate operating experience and methodology updates



Byron 1 RV Head Inspection Summary

- CRDM inspections required by CC N-729-1
 - B1R17 Refuel Outage
 - UT & PT indications in penetrations 64, 76, 43 & 31
 - Indications just below & at the J-groove weld
 - "Low" susceptibility T_{cold} head ~557°F
- Head fabricated by B&W
- CRDM penetration tubes
 - Nominal 4" dia. x 0.625" wall
 - B&W material heats
- Previous volumetric inspection in 2005
- PWSCC assumed





US PWR RV Head Inspections

- Baseline volumetric exams
 - Completed for US PWR fleet in accordance with NRC First Revised Order EA-03-009
 - Completed in 2008
- Re-inspections for T_{cold} RV heads
 - Order replaced by CC N-729-1
 - Lesser of 2.25 RIY or eight calendar years
 - First volumetric re-inspections of T_{cold} RV heads just beginning
 - Temporal distribution of upcoming inspections presented in next slide

T_{cold} Head Reinspection Plans





MRP OE Evaluation Screening Questionnaire

- Is there anything about this indication that would suggest it appropriate to re-evaluate any program requirements, previous inspection results, or upcoming inspection plans?
 - Initial qualitative assessment indicates that Code Case N-729-1 remains acceptable.
 - Although the indications were not safety significant in their orientation, did not challenge structural integrity of the penetration, and did not leak, the timing of identification of PWSCC in a T_{cold} RV head was sooner than may have been expected.
 - MRP will review the inspection program technical basis in light of this event.



MRP T_{Cold} Upper Head Action Plan

- Complete a review of the technical basis for ASME CC N-729-1
 - Address potential impact of OE on underlying analyses and conclusions
 - Substantiate the continued validity of T_{cold} head inspection requirements
- Bottom Mounted Nozzle Implication Evaluation
 - The implications of all CRDM conclusions reached will be reviewed relative to the current BMN inspection requirements (CC N-722) and susceptibility to PWSCC

Coordinated Reactor Vessel Surveillance Program (CRVSP)



Develop a coordinated PWR capsule withdrawal/test plan to increase the high-fluence PWR surveillance data, to support development of a future ETC applicable to the U.S. PWR fleet.

Existing RVSP of each US PWR was reviewed (capsule contents, fluences, withdrawal schedules); projected gaps in high-fluence data (>3E+19 n/cm²) were identified. Changes to withdrawal schedules are recommended to obtain high fluence surveillance data for the full range of materials across the entire industry.

CRVSP will significantly add to the quantity and quality of high fluence surveillance data by the year 2025.

Utility comments on Draft #2 are being addressed; Draft #3 expected from AREVA in June 2011; submit for TSC/IIG/PMMP review & approval and publish by end of 2011.





Preliminary Results

 CRVSP increases amount of high-fluence data across broad spectrum of RPV materials by 2025

 No. of additional capsules to be tested by 2025 at or above the stated fluence

Capsule Fluence	Current Plan	Recommended Plan
3x10 ¹⁹ n/cm ²	30	40
6x10 ¹⁹ n/cm ²	6	17
8x10 ¹⁹ n/cm ²	0	8
9x10 ¹⁹ n/cm ²	0	2

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Surface Stress Mitigation Techniques – MRP R&D Status

- Technical Basis Document for PWSCC Mitigation by Surface Treatments (MRP-267) - Product ID – 1020481 (January 2010)
 - Cavitation/Water Jet Peening and Fiber Laser Peening

- Surface Stress Mitigation Technologies
- Vendor laboratory testing and data
- MRP laboratory testing and data
- Factor of Improvements
- Definition of effective application
- Plant application and experience
- Experimental Program on the Effects of Surface Condition and Mitigation of Primary Water Stress Corrosion Cracking of Alloy 182 Welds (MRP-265) -Product ID – 1019084 (December 2009)
- Addressing remaining technical questions with resolution planned for the end of 2011 with no outstanding issues



Surface Stress Mitigation Techniques – Implementation and Summary

- Robust technologies
 - Implemented routinely in LWRs in Japan
 - Implemented in routinely Safety Critical Applications in Power, Defense, Aerospace, Automotive and Medical Industries
 - CRDM repair mitigation used in the U.S. in numerous PWRs
 - Abrasive Water Jet Peening part of 1/2 nozzle repairs
- Proactive mitigation desired
- Utilities are making near-term decisions on implementation for asset protection
- Warrants inspection credit and currently Code actions have been initiated
- Seek further NRC engagement



2012 International BWR/PWR Conference

International Boiling Water Reactor and Pressurized Water Reactor Materials Reliability Conference & Exhibition 2012 July 16-19, 2012

Gaylord Resort and Convention Center, 201 Waterfront Street, National Harbor, MD 20745





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