

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: S707090447 DOC. DATE: 87/07/01 NOTARIZED: NO DOCKET #
 FACIL: 50-305 Kewaunee Nuclear Power Plant, Wisconsin Public Service 05000305
 AUTH. NAME AUTHOR AFFILIATION
 HINTZ, D. C. Wisconsin Public Service Corp.
 RECIP. NAME RECIPIENT AFFILIATION
 Document Control Branch (Document Control Desk)

SUBJECT: Informs of currently pursuing addl. tech info re removal of trip signal & stopping further project design work per DC Hintz 861114 ltr to GE Lear & 10CFR60.62. Discussion to consolidate design details planned for Dec 1987 submittal.

DISTRIBUTION CODE: A055D COPIES RECEIVED: LTR 1 ENCL 0 SIZE: 1
 TITLE: OR/Licensing Submittal: Salem ATWS Events GL-83-28

NOTES:

	RECIPIENT ID CODE/NAME	COPIES		RECIPIENT ID CODE/NAME	COPIES	
		LTR	ENCL		LTR	ENCL
	PD3-3 LA	1	0	PD3-3 PD	3	3
	QUAY, T	1	0			
INTERNAL:	ARM/DAF/LFMB	1	0	NRR LASHER, D	1	1
	NRR/DEST/ICSB	1	0	NRR/DEST/PSB	1	0
	NRR/DEST/RSB	1	0	NRR/DLPQ/QAB	1	0
	NRR/DOEA/GCB	1	0	NRR/PMAS/ILRB	1	0
	NRR/PMAS/PMSB	1	0	OGC/HDS1	1	0
	<u>REG FILE</u> 01	1	1	RES/DE/EIB	1	1
EXTERNAL:	LPDR	1	1	NRC PDR	1	1
	NSIC	1	1			

TOTAL NUMBER OF COPIES REQUIRED: LTR 20 ENCL 0
 12



WISCONSIN PUBLIC SERVICE CORPORATION

600 North Adams • P.O. Box 19002 • Green Bay, WI 54307-9002

July 1, 1987

10 CFR 50.62

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Docket 50-305
Operating License DPR-43
Kewaunee Nuclear Power Plant
AMSAC

Reference 1: Letter from D. C. Hintz to G. E. Lear dated November 14, 1986

Reference 1 provided a Milestone Schedule for implementation of 10 CFR 50.62, ATWS Mitigation System at the Kewaunee Nuclear Power Plant. Reference 1 also identified the preferred design from WCAP 10858-A, as the design which generates the ATWS signal from main feedwater pump and main feedwater valve status. This design was preferred due to its complete diversity from the existing reactor protection system. However, Westinghouse has recently provided information on a modification to remove the steam flow feedwater flow mismatch coincident with low steam generator water level reactor trip. The removal of this reactor trip signal would make available existing feedwater flow transmitters since they would no longer be required for reactor protection. This in turn would change our preferred design to low feedwater flow initiation. This design is a more direct measurement of loss of normal feedwater.

We are currently pursuing additional technical information from Westinghouse on removal of the trip signal and therefore have stopped further project design work. When we receive the technical information a decision will be made on which design option to choose. We feel it would be premature to discuss plant specific design details at this time and instead will consolidate a discussion of all design details in our planned December, 1987 submittal.

Sincerely,

A handwritten signature in dark ink, appearing to read "D. C. Hintz".

8707090447 870701
PDR ADOCK 05000305
P PDR

D. C. Hintz
Vice President - Nuclear Power

DJM/jms

cc - Mr. Robert Nelson, US NRC
US NRC, Region III

Aoss
1/0