REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

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FACIL:50-305 Kewaunee Nuclear Power Plant, Wisconsin Public Servic 05000305
AUTH.NAME AUTHOR AFFILIATION
HINTZ,D.C. Wisconsin Public Service Corp.
RECIP.NAME RECIPIENT AFFILIATION
VARGA,S.A. Operating Reactors Branch 1

SUBJECT: Discusses design review of reactor inventory tracking sys which revealed need for further analysis to better define containment temp profiles during smal break LOCAs.Sys description will be transmitted after receipt of C-E rept.

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TITLE: OR Submittal:Inadequate Core Cooling (Item II.F.2) GL 82-28

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NRC-85-129

WISCONSIN PUBLIC SERVICE CORPORATION
P.O. Box 19002, Green Bay, WI 54307-9002

July 29, 1985

Director of Nuclear Reactor Regulation
Attention: Mr. S. A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Gentlemen:

Docket 50-305
Operating License DPR-43
Kewaunee Nuclear Power Plant
TAC #M45143
Inadequate Core Cooling Instrumentation

References: 1) Letter from Mr. S. A. Varga to Mr. D. C. Hintz dated May 22, 1985
2) Letter from Mr. D. C. Hintz to Mr. S. A. Varga dated March 29, 1985
3) Letter from Mr. S. A. Varga to Mr. D. C. Hintz dated November 30, 1984

Reference 3 transmitted your Safety Evaluation Report (SER) for our proposed Reactor Inventory Tracking System (RITS), and a request for additional information on this system. The Request was for information on the RITS error analysis and other points of interest to NRC.

Reference 2 transmitted our error analysis method and sample calculations indicating how the method was to be applied. The sample calculation used manufacturer's data when available and best estimates when necessary.

Subsequent to transmittal of the error analysis, a design review of the RIT System was initiated to further refine the analysis and to solidify the basis for best estimate assumptions. The design review has revealed the need for further analysis to better define containment temperature profiles during small break LOCAs; bulk RCS fluid temperatures; two phase flow conditions; output displays; and operating range.

This design review, performed in-house, developed the specification for an independent design review which is currently being performed by Combustion Engineering.

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Reference 3 transmitted a request for additional information concerning the specifics of both the error analysis and the system design. The response was requested within 30 days of receipt of the letter and was extended to July 29, 1985 by consent of our Project Manager.

Since the questions are specific to the design detail and analysis they can best be answered by a detailed system description. The system description previously transmitted to you will be expanded and forwarded subsequent to the independent design review. Each of your questions will be addressed in the system description.

We currently expect the independent design review to be completed on October 1, 1985. Assuming only minor system changes resulting from this review, we expect to transmit the completed system description to you within 60 days of receiving Combustion Engineering's final report. If difficulties arise which prevent transmittal on this schedule we will notify our Project Manager.

Very truly yours,

D. C. Hintz
Manager - Nuclear Power

DWS/js

cc - Mr. Robert Nelson, US NRC