

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

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SUBJECT: Informs of status & future plans for SPDS safety assessment sys installed in response to NUREG-0737, Item I.D.2. Sys will depart from generic sys by indicating statistical validity parameters in alarmed state.

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WISCONSIN PUBLIC SERVICE CORPORATION

P.O. Box 1200, Green Bay, WI 54305



April 26, 1985

Director of Nuclear Reactor Regulation
 Attention: Mr. S. A. Varga, Chief
 Operating Reactors Branch No. 1
 Division of Licensing
 U.S. Nuclear Regulatory Commission
 Washington, D.C. 20555

Gentlemen:

Docket 50-305
 Operating License DPR-43
 Kewaunee Nuclear Power Plant
Status of Safety Parameter Display System

References: 1) Letter from C. W. Giesler (WPSC) to D. G. Eisenhut (NRC)
 dated April 15, 1983
 2) Letter from C. W. Giesler (WPSC) to D. G. Eisenhut (NRC)
 dated September 2, 1983
 3) Letter from D. C. Hintz (WPSC) to D. G. Eisenhut (NRC)
 dated August 1, 1984

In response to NUREG-0737 item I.D.2, Plant Safety Parameter Display Console, WPSC installed the Safety Assessment System (SAS). The plant-specific SAS installed at the Kewaunee Nuclear Power Plant was described in references 1 through 3. The SAS has been implemented and operators trained prior to completion of the 1985 refueling outage this April. We would like to take this opportunity to inform you of our future plans for the SAS.

Reference 2 provided a safety analysis report (TAC number M51250) which described departures from the generic SAS. An additional departure is planned for our plant-specific SAS. SAS has the capability of showing the status of a parameter beyond the simple alarmed or not alarmed state. Also, the SAS validation described in reference 3 has the capability to indicate to the operator when a parameter's value is statistically invalid. However, generic SAS is unable to indicate statistical validity when the parameter is in the alarmed state. WPSC plans to depart from the generic SAS design in this area and indicate statistical validity in the alarmed state. This change would be made in

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conjunction with another change planned for the validation process. The plant process computer supplies the necessary inputs to the SAS. However, in some cases the plant process computer supplies a calculated average to SAS as opposed to the actual input values. To make parameter validation in the SAS consistent, WPSC plans to have individual inputs supplied directly to the SAS. We are in the process of initiating the changes necessary to provide indication of statistical validity in the alarmed state and to provide consistent parameter validation. WPSC has retained a consultant to assist in software upgrades of the plant process computer. The scope of the consultant's work includes an assessment of the changes described above. A preliminary schedule for the completion of the changes would be available upon the completion of this assessment.

Reference 3 provided information to you concerning the basis for parameter selection. However, four parameters discussed were not presently connected to the computer. This letter stated, "Future WPS action will either connect these points or remove reference to them based upon operational or licensing reviews." Action has been initiated to connect the points in question to the computer. These parameters are Reactor Vessel Level, Auxiliary Feedwater Flow, Wide Range Containment Pressure, and Containment Sump Level. Connection of these points will be dependent on other schedules such as installation of inadequate core cooling instrumentation. These additional connections will be coordinated with related projects' schedules.

WPSC recognizes the capabilities that the SAS brings into the control room. The current system is an effective tool in assisting the rapid and reliable determination of the safety status of the plant. The changes discussed above will bring consistency to the system and will increase the operator's confidence in the SAS. Therefore, the changes will proceed as efficiently as possible taking into consideration our unique system, volatile software schedules, and related project plans. We hope this information will be of assistance to you in any ongoing reviews of our Safety Assessment System.

Very truly yours,



D. C. Hintz
Manager - Nuclear Power

KAH/js

cc - Mr. Robert Nelson, US NRC