NRC Amendment No. 48 Instruction Sheet

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TECHNICAL SPECIFICATION REVISION CONTROL

WPS Amend. No.	Date	Page Submitted	NRC Amend. No.	Date	Pages Removed	Pages Inserted
49	08/18/82 09/15/82	TS i thru TS v, TS 3.1-2a, TS 3.3-2, TS 3.5-6, TS 3.9-6, TS 3.9-7, TS 3.10-2, TS 3.10-6, TS 3.10-8, TS 3.10-21, Table TS 3.5-1, Table TS 3.15-1, Table TS 3.15-2, TS 4.15-2, Table TS 4.10-1 (Page 1 of 6), Table TS 4.10-1 (Page 5 of 6), TS 6-17, TS 6-24, ES i, Rest of ETS Section	47	11/29/82	All of those submitted.	In addition to those submitted: TS vi, TS vii, TS 3.1-8, All ETS pages deleted.
50	09/27/82	TS 6-2a, TS 6-5 thru TS 6-8, TS 6-11, TS 6-12, TS 6-17, TS 6-26 thru TS 6-28	48	01/03/83	All of those submitted.	In addition to those submitted: Figure 6.2-1, Figure 6.2-2

physical form for sample analysis or instrument calibration or associated with radioactive apperatus or components:

(5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility".

C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR, Chapter 1: Part 20, Section 30.34 of Part 30 Section 40.41 of Part 40, Section 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensees are authorized to operate the facility at steady state reactor core power levels not in excess of 1650 megawatts (thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 48, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications. | 48

(3) The licensee may proceed with and is required to complete the modifications identified in Paragraphs 3.1.1, 3.1.2 and 3.1.4 through 3.1.28 of the Fire Protection Safety Evaluation Report. These modifications shall be completed by the dates specified in Table 3.1. Dates for resolution of items are specified in Table 3.2. In the event that these dates for completion cannot be met, the licensee shall submit a report explaining the circumstances and propose a revised schedule.

(4) Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the following Commission approved documents, including amendments and changes made pursuant to the authority of 10 CFR 50.54(p). These approved documents consist of information withheld from public disclosures pursuant to 10 CFR 2.790 (d).

a) "Industrial Security Manual" dated May 25, 1977, January 9, 1978, December 18, 1978, January 30, 1979, March 7, 1979 and March 27, 1979.

b) Kewaunee Nuclear Power Plant Safeguards Contingency Plan, as originally submitted by letter of March 27, 1979, and subsequently revised and re-submitted by letter of February 20, 1981, pursuant to 10 CFR 73.40. The Safeguards Contingency Plan shall be fully implemented, in accordance with 10 CFR 73.40(b) within 30 days of this approval by the Commission.

6.4 TRAINING

48

6.4.1 A retraining and replacement training program for the Plant Staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI-N18.1-1971 and Appendix A of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Fire Marshall and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except that training sessions shall be held quarterly.

6.5 REVIEW AND AUDIT

6.5.1 PLANT OPERATIONS REVIEW COMMITTEE (PORC)
FUNCTION

6.5.1.1 The PORC shall function to advise the Plant.

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FUNCTION

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changes to the Vice-President - Nuclear Power. | 48

AUTHORITY

6.5.1.7 The PORC shall:

a. Recommend to the Plant Manager approval or disapproval of items considered under 6.5.1.6a through d above.

b. Make determinations with regard to whether or not each item considered under 6.5.1.6a through e above constitutes an unreviewed safety question.

c. Provide immediate notification in the form of draft meeting minutes to the Manager-Nuclear Power and the Chairman-Nuclear Safety Review and Audit Committee of disagreement between the PORC and the Plant Manager. The Plant Manager shall have responsibility for resolution of such disagreements. | 48

RECORDS

6.5.1.8 Minutes shall be kept of all meetings of the PORC and copies shall be sent to the Manager -Nuclear Power and the Chairman-Nuclear Safety Review and Audit Committee. | 48

6.5.2 CORPORATE NUCLEAR ENGINEERING STAFF (CNES)

FUNCTION

6.5.2.1 The CNES shall function to provide engineering,

changes to the Vice-President - Nuclear Power. | 48

AUTHORITY

6.5.1.7 The PORC shall:

a. Recommend to the Plant Manager approval or disapproval of items considered under 6.5.1.6a through d above.

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RECORDS

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6.5.2 CORPORATE NUCLEAR ENGINEERING STAFF (CNES)

FUNCTION

6.5.2.1 The CNES shall function to provide engineering,

technical and quality assurance activities in support of the Kewaunee Plant Staff.

ORGANIZATION

6.5.2.2 The CNES consists of the following groups:

a. Nuclear Licensing and Systems
b. Nuclear Services
c. Nuclear Training
d. Nuclear Design Change
e. System Planning and Engineering
f. Power Plant Design and Construction
g. Fuel and Fossil Operations
h. Administrative Staff

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ACTIVITIES

6.5.2.3

1. Review and report all violations of the Technical Specifications, codes, regulations, and statutes.
2. Review all activities associated with nuclear safety for technical adequacy and compliance with internal procedures or instructions.
3. Review and report significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
4. Review and report all events which are required by regulations or Technical Specifications to be reported to the NRC (Plant personnel will provide the initial reporting to the NRC of those events requiring 24 hour notification.)
5. Investigate any indication of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems or components.

6. Review and/or prepare safety evalutions of all plant design changes.
7. Audits as required by the Quality Assurance Program and as outlined in Section 6.5.3.8.

AUTHORITY

6.5.2.4 Members of the Fuel and Fossil Operations, Power Plant Design and Construction, and System Planning and Engineering groups, although not directly responsible to the Vice President - Nuclear Power, are available for special projects and support of the Kewaunee Plant. 48

The Nuclear Design Change, Nuclear Licensing and Systems, Nuclear Services, and Nuclear Training groups are responsible to the Manager Nuclear Power.

6.5.3 NUCLEAR SAFETY REVIEW AND AUDIT COMMITTEE (NSRAC)

FUNCTION

6.5.3.1 The NSRAC shall function to provide independent review and audit of designated activities in the areas of:

a. Nuclear Power Plant Operations
b. Nuclear Engineering
c. Chemistry and Radio-Chemistry
d. Metallurgy
e. Instrumentation
f. Radiological Safety
g. Mechanical and Electrical Engineering
h. Quality Assurance Practices
i. Other appropriate fields as determined by the Committee, to be associated with the unique characteristics of the nuclear power plant.

COMPOSITION

6.5.3.2 The NSRAC shall be composed of, but not necessarily limited to:

a. At least three technically qualified persons who are not members of the plant staff.

b. One member from the supervisory staff of the plant.

c. At least two qualified non-company affiliated technical consultants.

d. Plus in-house staff management advisors as required.

The Committee membership and its Chairman and Vice Chairman shall be appointed by the Vice-President - Nuclear Power or such person as he shall designate. Each member of the NSRAC shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of five years technical experience, of which a minimum shall be in one or more areas given in 6.5.3.1. | 48

ALTERNATES

6.5.3.3 Alternate members shall be appointed by the NSRAC Chairman, upon approval by the Vice-President - Nuclear Power, to serve on a temporary basis; however, no more than two alternates shall participate in NSRAC activities at any one time. | 48

CONSULTANTS

6.5.3.4 Consultants may be utilized as determined by the Chairman - NSRAC to provide expert advice to the NSRAC.

MEETING FREQUENCY

6.5.3.5 The NSRAC shall meet at least once every six months. | 48

g. Any other area of plant operation considered appropriate by the NSRAC or the Vice-President - Nuclear Power. | 48

AUTHORITY

6.5.3.9 The NSRAC shall report to and advise the Vice-President - Nuclear Power on those areas of responsibility specified in Section 6.5.3.7 and 6.5.3.8. | 48

RECORDS

6.5.3.10 Records of NSRAC activities shall be prepared, approved and distributed as follows:

a. Minutes of each NSRAC meeting forwarded to the Vice-President - Nuclear Power within 14 days following each meeting. | 48

b. Reports of reviews required by Section 6.5.3.7e, f, g and h above, forwarded to the Vice-President - Nuclear Power within 14 days following completion of the review. | 48

c. Reports of audits performed by NSRAC shall be forwarded to the Vice-President - Nuclear Power and to the management positions responsible for the areas audited within 30 days after completion of the audit. | 48

6.6 DELETED

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a safety limit is violated:

a. The reactor shall be shutdown and operation shall not be resumed until authorized by the Commission.

b. The Safety Limit violation shall be reported to the Commission, the Manager-Nuclear Power, and to the NSRAC-Chairman within 14 days of the violation. | 48

c. The Report shall be prepared in accordance with Section 6.9 of the Technical Specifications.

6.8 PROCEDURES

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained that meet the requirements and recommendations of Section 5.1 and 5.3 of ANSI N18.7-1972.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the Plant Manager prior to implementation and periodically as determined by the Plant Manager.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

a. The intent of the original procedure is not altered.

b. The change is approved by two members of the Plant Management Staff, at least one of which holds a Senior Reactor Operator's License, if the procedure affects nuclear safety.

c. The change is documented, reviewed by the PORC, and approved by the Plant Manager.

steady state conditions greater than or equal to 1% ΔK/K; a calculated reactivity balance indicating a shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if subcritical, an unplanned reactivity insertion of more than 50¢; or occurrence of any unplanned criticality.

(5) Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR. | 48

(6) Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.

Note: For items 6.9.2.a(5) and 6.9.2.a(6) reduced redundancy that does not result in a loss of system function need not be reported under this section but may be reportable under items 6.9.2.b(2) and 6.9.2.b(3) below.

(7) Conditions arising from natural or man-made events that, as a direct result of the event require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.

(8) Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications.

6.14 ENVIRONMENTAL QUALIFICATION

6.14.1 By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of: Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification Class IE Electrical Equipment in Operating Reactors" (DOR Guidelines); or, NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment," December 1979. Copies of these documents are attached to Order for Modification of License No. DPR-43 dated October 24, 1980. | 48

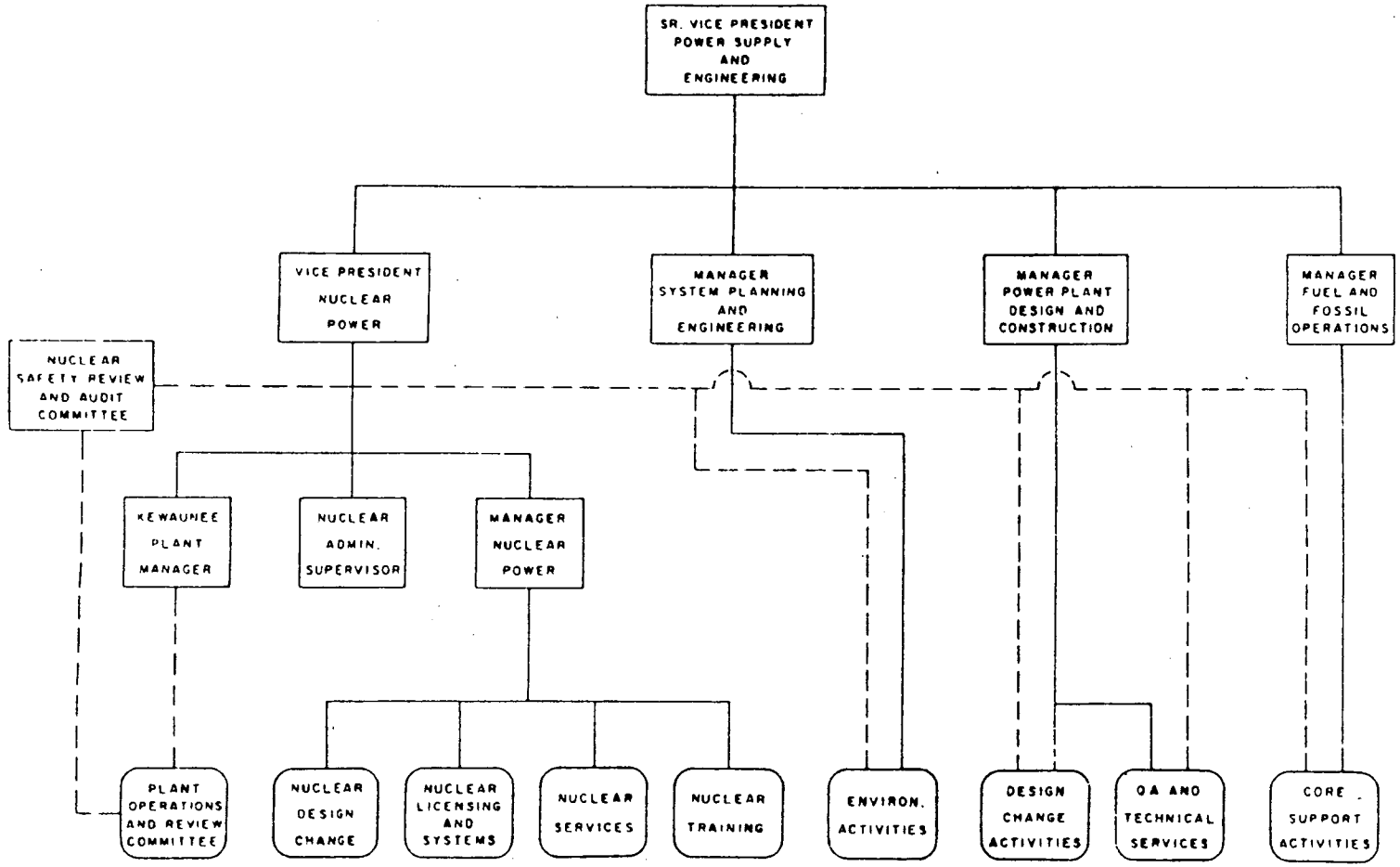
6.14.2 By no later than December 1, 1980, complete and auditible records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified.

NOTE: 10CFR50.49 suspends the June 30, 1982, requirement.

PAGE TS 6-27 DELETED

Page TS 6-26 replaces page TS 6-28

Page TS 6-28 deleted.



LEGEND

_____ Direct Responsibility

------ Lines of Communication

Figure TS 6.2-1
Effective Date 06/01/82

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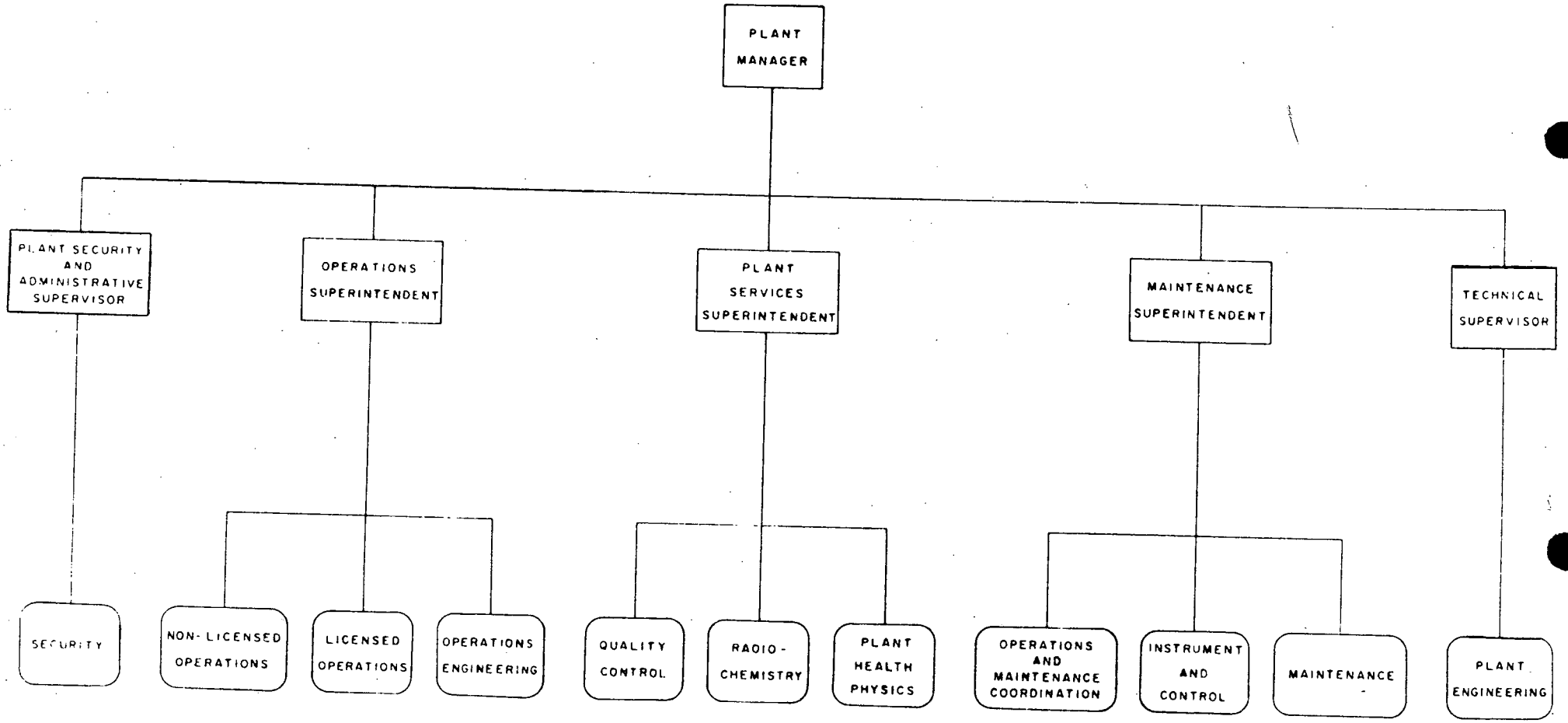
01/03/83



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01/03/83
Figure TS 6.2-2
Effective Date 06/01/82