

FROM: Wisconsin Public Service Corp. Green Bay, Wisconsin 54305 E. W. James		DATE OF DOCUMENT: 1-27-71		DATE RECEIVED: 1-30-71		NO.: 421	
TO:		LTR.: x notarized 1-27-71		MEMO: _____		ORT: _____	
Dr. Peter A. Morris		ORIG.: _____		CC: _____		OTHER: _____	
CLASSIF: U		POST OFFICE		ACTION NECESSARY <input type="checkbox"/>		CONCURRENCE <input type="checkbox"/>	
REG. NO:		FILE CODE: 50-305 (INPUT)		NO ACTION NECESSARY <input type="checkbox"/>		COMMENT <input type="checkbox"/>	
DESCRIPTION: (Must Be Unclassified) Ltr(In Binder, Vol 1) notarized 1-27-71 trans the following:		REFERRED TO		DATE		RECEIVED BY	
		Long W/4 cys for ACTION		1-30-71		DO NOT REMOVE	
		DISTRIBUTION:				ACKNOWLEDGED	
ENCLOSURES: <u>AMDT # 7</u> consisting of:		Regulatory File				Rosen	
FSAR - Volumes 1-2-3-4-5 (88 cys)		AEC PDR				Collins(2)	
*ENVIRO RPT (150 cys)		Compliance (2)				D. Thompson	
		OGC(Rm P 506A)				DTIE(Laughlin)	
		Dube/Wilson				NSIC(Buchanan)	
		Boyd				AEC HQ LIB J 004	
		DeYoung				ASLB(Yore "H" St)	
		Klecker				Warren Nyer (2)	
REMARKS: *NOTE: Enviro Rpt not included w/this distribution; See Mail Control # 241-A for Dist of Enviro Rpt. Holding (16) cys for ACRS		Howe				Chief Water Reactor, RDT	
		Maccary				Dr. Totter	
		Dromerick (3)				John F. Pearson	
		Minogue				Case(Reading File)od	
		Moore					

U.S. ATOMIC ENERGY COMMISSION

MAIL CONTROL FORM FORM AEC-3265  
(6-60)

WISCONSIN PUBLIC SERVICE CORPORATION



P.O. Box 700, Green Bay, Wisconsin 54305

January 27, 1971

Regulatory

File Cy.

Dr. Peter A. Morris, Director  
Division of Reactor Licensing  
U. S. Atomic Energy Commission  
Washington, D. C. 20545

Subject: Amendment No. 7 (Final Safety Analysis  
Report) to the Application for Construction  
Permit and Operating License for Kewaunee  
Nuclear Power Plant  
AEC Docket 50-305

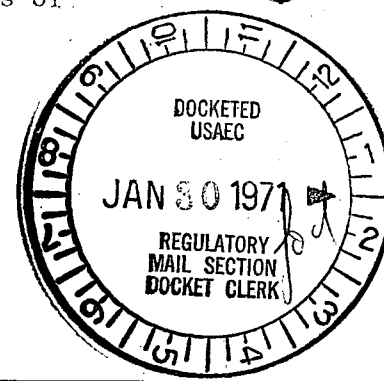
Dear Dr. Morris:

The application for construction permit and operating license for the Kewaunee Nuclear Power Plant filed August 18, 1967 in AEC Docket 50-305 is hereby amended by submission of Amendment 7 for Kewaunee Nuclear Power Plant pursuant to Section 50.34 of 10 CFR 50. Said amendment consists of the Final Safety Analysis Report excluding the proposed technical specifications. These technical specifications will be submitted within one month of the date of this application.

The Final Safety Analysis Report analyzes the design of the unit for operation at the power level currently contemplated, 1650 MWt or the equivalent net electrical output of 540 MWe. Accordingly, it is requested that the design of the unit be reviewed for operation and power levels up to 1650 MWt and that the facility license be issued authorizing operation of the unit at such power level subject to the condition of such tests and demonstrations and the submission of such further reports as the Commission may deem appropriate.

In addition to three signed originals, 85 copies of this amendment are also submitted.

*good neighbors since 1883*



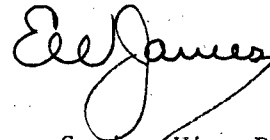
January 27, 1971

A certificate confirming service of Amendment No. 7 upon Mr. Donald L. Quistorff, Chairman Kewaunee County Board and Mr. Arden Koehler, Chairman, Town of Carlton, Kewaunee County, in which the Kewaunee Nuclear Power Plant is located, will be filed herein when an acknowledgement of receipt thereof has been obtained.

Wisconsin Public Service Corporation's "Environmental Report - Operating License Stage" for the Kewaunee Plant accompanies this amendment in accordance with Appendix D to 10 CFR 50 (Statement of General Procedure: Implementation of the National Environmental Policy Act of 1969 (Public Law 91-150)). One hundred-fifty (150) copies of this report are included.

Very truly yours,

WISCONSIN PUBLIC SERVICE CORPORATION



E. W. James, Senior Vice President  
Power Generation & Engineering

SUBSCRIBED and SWORN to  
before me this 27th day  
of January, 1971.

  
Notary Public

My Commission Expires May 9, 1971

FROM: Wisconsin Public Service Corp  
Green Bay, Wisconsin 54305  
E. W. James

DATE OF DOCUMENT

1-15-70

DATE RECEIVED

1-22-70

NO.:

202

LTR.

MEMO:

REPORT

OTHER:

x notarized 1-15-70

TO:

Dr Peter A. Morris

ORIG.:

CC:

OTHER:

6 signed & 40 conf'd

ACTION NECESSARY ☐

CONCURRENCE ☐

DATE ANSWERED

NO ACTION NECESSARY ☐

COMMENT ☐

BY:

CLASSIF:

POST OFFICE

U

REG. NO.:

FILE CODE:

50-305

DESCRIPTION: (Must Be Unclassified)

Ltr containing suppl info & answers  
in response to our 2-24-69 ltr &  
meeting of 9-18-69 and trans:

REFERRED TO

DATE

RECEIVED BY

DATE

Knuth  
w/4 cys for action

1-22-70

DISTRIBUTION:

Regulatory file  
AEC PDR

Compliance (2)  
OGC ( Rm P 506 A)

H. Price & Staff  
Dube/Levine

T. R. Wilson  
Boyd

D. Thompson  
Rosen

Moore  
Howe

Dromerick (2)

Case (Reading File)

DTIE (Laughlin)

NSIC (Buchanan)

AEC Hq Lib J-004

DO NOT REMOVE

ACKNOWLEDGED

ENCLOSURES:

New & Revised Pages to the PSAR

(40 cys rec'd)

REMARKS:

Holding (18) cys for ACRS

202

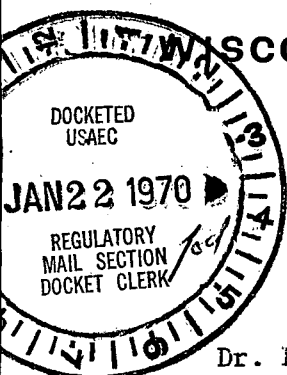
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U.S. ATOMIC ENERGY COMMISSION

MAIL CONTROL FORM

FORM AEC-3268  
(8-60)

# WISCONSIN PUBLIC SERVICE CORPORATION



P.O. Box 700, Green Bay, Wisconsin 54305

January 15, 1970

Regulatory

File Cy.

Dr. Peter A. Morris, Director  
Division of Reactor Licensing  
UNITED STATES ATOMIC ENERGY COMMISSION  
Washington, D. C. 20545

Dear Dr. Morris

Subject: Kewaunee Nuclear Power Plant  
AEC Docket Number 50-305



As requested in your letter, dated February 24, 1969, and further meeting with your staff on September 18, 1969, we are transmitting the following supplementary information to the Facility Description and Safety Analysis Report:

1. Containment Design Features: The final design of the following features was highlighted for review (reference your letter dated, February 24, 1969):
  - a. Piping penetrations, using the guard pipe.
  - b. Seismic design of large penetrations, such as the steam-pipe and personnel airlock.
  - c. Limitations on deformation of the Shield Building and connected system.
  - d. Vacuum Relief Valves.

Answers to the above items are attached.

2. Tornado criteria for spent fuel: (reference your letter dated, February 24, 1969):

Answer to this question was reviewed with your staff at the September 18, 1969, meeting and the writeup is attached with this transmittal.

3. In addition to the above items requested in your letter dated February 24, 1969, we are transmitting the following supplementary information:

- a. Reclassification of Structures and Equipment:

Appendix-B, of the Facility Description and Safety Analysis Report, outlined special design procedures for the different classification of structures and components of the plant.

In the Appendix-B, the Structures and Components were divided into two classes, Class I and Class II.

Class I was defined as (reference Appendix-B) those items which are essential to the safe shutdown and isolation of the reactor, or whose failure may cause, or increase the severity of a loss-of-coolant accident, or may result in an uncontrolled release of radioactivity.

Class II was defined as those items which are not essential to the safe shutdown and isolation of the reactor, or whose failure will not cause, or increase severity of a loss-of-coolant accident, or will not result in uncontrolled release of radioactivity.

Applicable Design Codes, Loads, Design Criteria, Structural Design basis etc. were all set forth in the Appendix-B.

The earthquake loads for Class II items were specified to be in accordance with the requirements of the uniform building code. This code specifies the location of the plant site to be in a "Zero" earthquake area. However, for Class II structures, which bear a relationship to plant safety, earthquake loads applicable to Zone I areas will be used in the analysis.

In essence Class II items as listed in the PSAR, consisted of two categories. The main difference being those that would receive seismic consideration (be designed for Zone I earthquake) and a large group of conventional equipment where earthquake design is not a consideration.

However, in the original PSAR Appendix-B, the two categories within Class II were not identified. In order to clearly establish our existing design basis, we have now expanded our classification to three classes and all items are clearly identified.

Appropriate revised pages and a reclassified list of structures and components in a tabular form, are attached to be inserted in the PSAR.

b. Electrical Changes:

Two basic changes have been made to increase the reliability of power to the safeguards equipment. These changes are in two areas: the number of sources supplying power to the safeguards buses and the manner of supplying diesel power to the safeguards buses.

Initially, two normal sources of power supply were provided to the safeguards buses with the diesel generators as the emergency source. We have since added a source of power from the 345/138 Kv substation stepdown transformer 13.8 Kv tertiary. The Tertiary Auxiliary Transformer, served by this tie to the tertiary, is rated to serve the total load on both safeguards buses.

January 15, 1970

Thus, we have three normal operating sources of power plus the diesel generator emergency source for each bus.

Appropriate revised sheets are attached to be inserted in the PSAR.

Yours very truly,



E. W. James, Vice President  
Power Generation & Engineering

Subscribed and Sworn to, before me,

this 15 day of January, 1970



My Commission Expires May 9, 1971

*done 2-13-70*

INSTRUCTIONS FOR MAKING ADDITIONS

OF SUPPLEMENTARY INFORMATION OF

DECEMBER 31, 1969

This supplementary information of December 31, 1969 has been printed on cherry color sheets in order to identify it as new/revised information after insertion into the PSAR.

The following listing of material furnished as Supplementary Information of December 31, 1969, will serve as a check list for entering the attached pages.

Enter the added pages or sections as listed. No pages are to be removed from the PSAR.

File this instruction sheet along with the cover letter in the front of Volume I as a record of additions.

1. Containment Design Features

Insert the following new pages in Volume IV, Section 5:

5.47-1 through 5.47-10. (12-31-69) ✓

2. Tornado Criteria for Spent Fuel

Insert the following new pages in Volume IV, Section 5:

5.25a-1 through 5.25a-3 (12-31-69)



<u>Enter</u>	<u>In Front of</u>
8.3.1-5/8.3.1-6 (12-31-69) ✓	8.3.1-5/Blank (1-2-68) ✓
8.3.2-1 (12-31-69)/8.3.2-2 (1/2/68) ✓	8.3.2-1/8.3.2-2 (1-2-68) ✓
8.4.1-1/Blank (12-31-69) ✓	8.4.1-1/8.4.1-2 (1-2-68) ✓
8.4.2-1/8.4.2-2 (12-31-69) ✓	8.4.2-1/8.4.2-1a (1-2-68) ✓
8.4.3-1/8.4.3-2 (12-31-69) ✓	8.4.3-1/Blank (1-2-68) ✓
Figure 8-1 (12-31-68) ✓	Figure 8-1 (5-3-68) ✓
Figure 8-2 (12-31-68) ✓	Figure 8-2 (5-3-68) ✓
Figure 8-3 (12-31-69) ✓	Figure 8-3 (5-3-68) ✓

Insert the following page in Volume IV, Section 8:

8.2-1 (12-31-69)

8.2.-1 (1-2-68)

### 3. Other Items:

#### (a) Reclassification of Structures and Equipment (Appendix - B):

Enter the revised pages, listed in Column 1 below  
ahead of the superseded pages listed in Column 2.

<u>Enter</u>	<u>In Front of</u>
B-1 (12-31-69) ✓	B-1 (3-15-68) ✓
B-1/B-2 (12-31-69) ✓	B-1/B-2 (1-2-68) ✓
B-3/B-4 (12-31-69) ✓	B-3/B-4 (1-2-68) ✓
B-5/B-6 (12-31-69) ✓	B-5/B-6 (1-2-68) ✓
B-7/B-8 (12-31-69) ✓	B-7/B-8 (5-3-68) ✓
B-9 (Blank - (12-31-69)/B-10 (1-2-68) ✓	B-9/B-10 (1-2-68) ✓
B-11 (1-2-68)/B-12 (12-31-69) ✓	B-11/B-12 (1-2-68) ✓
B-13a/B-13b (12-31-69) ✓	None
B-13c/B-13d (12-31-69) ✓	None
B-13e/B-13f (12-31-69) ✓	None
B-13g/Blank (12-31-69) ✓	None
B-31/B-32 (12-31-69) ✓	None

#### (b) Electrical changes (Section 8):

<u>Enter</u>	<u>In Front of</u>
8-1 (12-31-69) ✓	8-1 (1-2-68) ✓
8.1-1 (12-31-69)/Blank ✓	8.1-1 (1-2-68)/Blank ✓
8.2.1-1/8.2.1-2 (12-31-69) ✓	8.2.1-1/8.2.1-2 (1-2-68) ✓
8.2.2-1/8.2.2-2 (12-31-69) ✓	8.2.2-1/8.2.2-2 (1-2-68) ✓
8.3.1-1/8.3.1-2 (12-31-69) ✓	8.3.1-1/8.3.1-2 (1-2-68) ✓
8.3.1-3/8.3.1-4 (12-31-69) ✓	8.3.1-3/8.3.1-4 (1-2-68) ✓