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UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
799 ROOSEVELT ROAD
GLEN ELLYN, ILLINOIS 60137

JUL 8 1975

Wisconsin Public Service Corporation
ATTN: Mr. E. W. James, Senior Vice
President
Power Generation and Engineering
P. O. Box 1200
Green Bay, Wisconsin 54305

Docket No. 50-305

Gentlemen:

This refers to the inspection conducted by Mr. D. C. Boyd of this office on June 2, 4, 11, 17 and 19, 1975, of activities at the Kewaunee Nuclear Facility authorized by NRC License No. DPR-43 and to the discussion of our findings with Mr. Luoma and others of your staff at the conclusion of the inspection.

A copy of our report of this inspection is enclosed and identifies the areas examined during the inspection. Within these areas, the inspection consisted of a selective examination of procedures and representative records, interviews with plant personnel, and observations by the inspector.

During this inspection, it was found that certain of your activities appear to be in noncompliance with NRC requirements. The items and reference to the pertinent requirements are listed under Enforcement Action in the Summary of Findings Section of the enclosed inspection report. Prior to the conclusion of the inspection, the inspector determined that corrective action had been taken with respect to these items of noncompliance and that measures have been taken to assure that a similar, future noncompliance will be avoided. Consequently, no reply to this letter is required, and we have no further questions regarding these matters at this time.

In accordance with Section 2.790 of the NRC's "Rules of Practice," Part 2, Title 10, Code of Federal Regulations, a copy of this letter and the enclosed inspection report will be placed in the NRC's Public Document Room. If this report contains any information that you or your contractors believe to be proprietary, it is necessary that you make a written



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JUL 8 1975

application to this office, within twenty days of your receipt of this letter, to withhold such information from public disclosure. Any such application must include a full statement of the reasons for which it is claimed that the information is proprietary, and should be prepared so the proprietary information identified in the application is contained in a separate part of the document. Unless we receive an application to withhold information or are otherwise contacted within the specified time period, the written material identified in this paragraph will be placed in the Public Document Room.

No reply to this letter is necessary; however, should you have any questions concerning this inspection, we will be glad to discuss them with you.

Sincerely yours,

Gaston Fiorelli, Chief
Reactor Operations Branch

Enclosure:
IE Inspection Rpt No. 050-305/75-10

bcc: PDR
Local PDR
NSIC
TIC
OGC, Beth, P-506A

U. S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT

REGION III

Report of Operations Inspection

IE Inspection Report No. 050-305/75-10

Licensee: Wisconsin Public Service Corporation
P. O. Box 1200
Green Bay, Wisconsin 54305

Kewaunee Nuclear Power Plant
Kewaunee, Wisconsin

License No. DPR-43
Category: C

Type of Licensee: PWR W 1650 MWt

Type of Inspection: Resident Inspector Program

Dates of Inspection: June 2, 4, 11, 17, and 19, 1975

Principal Inspector: *E. L. Jordan*
D. C. Boyd

7/8/75
(Date)

Accompanying Inspectors: None

Other Accompanying Personnel: None

Reviewed By: *E. L. Jordan*
E. L. Jordan
Senior Inspector
Projects Unit 2
Reactor Operations Branch

7/8/75
(Date)

SUMMARY OF FINDINGS

Inspection Summary

Inspection on June 2, 4, 11, 17, and 19, (75-10): Observations in control room; verification of adherence to the following Technical Specifications; 2.3 reactor trip settings; 3.0, primary coolant activity and quality 3.3; engineered safety features limiting conditions; table 3.5.1, engineered safety system instrument settings; 6.2, abnormal occurrences; 6.6, review committee functioning. One item of noncompliance was identified regarding abnormal occurrence reporting.

Enforcement Items

Deficiency

Contrary to Technical Specification 6.2, records do not exist to verify that in all cases "Abnormal Occurrences" are immediately reported to the superintendent - Nuclear Power - or that the notification to the NRC of the abnormal occurrence is made by the "Superintendent - Steam Plants". (Paragraph 4)

Licensee Action on Previously Identified Enforcement Items

None.

Other Significant Items

A. Systems and Components

Heater drain pump to be returned to service during outage ending June 22, 1975. (Paragraph 8)

B. Facility Items (Plans and Procedures)

Scheduled maintenance outage from June 19, 1975 to June 22, 1975, for miscellaneous small repairs.

C. Managerial Items

None.

D. Noncompliance Identified and Corrected by Licensee

May 21, 1975, four containment sump isolation valves were not tested in accordance with Technical Specifications. (Paragraph 7)

E. Deviations

None.

F. Status of Previously Reported Unresolved Items

None.

Management Interview

A management interview was conducted on June 19, 1975, with Messrs. Luoma, Plant Superintendent, and Lange, Assistant to the Plant Superintendent. The items discussed included the following:

- A. Resident Inspection Program Activities. (Paragraph 3)
- B. Enforcement Items. (Paragraphs 4 and 7)
- C. Abnormal Occurrences. (Paragraph 5)
- D. Followup Inspection Items. (Paragraphs 6.a, b, and c)
- E. System and Component Problems. (Paragraph 8)

REPORT DETAILS

1. Persons Contacted

Persons contacted during this inspection include:

Wisconsin Public Service Corporation (WPS)

C. Luoma, Plant Superintendent
R. Lange, Assistant Plant Superintendent
C. Steinhardt, Reactor Supervisor
W. Truttman, Operations Supervisor
D. Ristau, Training Supervisor
J. Richmond, Technical Supervisor
D. McSwain, Instrument and Control Supervisor
R. Hirst, Maintenance Supervisor

2. General

Reports issued under the resident inspection program address areas of inspection completed in the implementation of the program, and will not report in detail in each area unless the findings warrant further discussion.

3. Inspection Activities

- a. Observation in control room.
- b. Observation of an emergency plan drill, and participation in critique of the drill.
- c. Verification of adherence to the following Technical Specifications:
 - (1) 2.3 - Reactor Trip Settings.
 - (2) 3.1.c - Max. coolant activity.
 - (3) 3.1.e - Primary coolant O₂, Cl, F.
 - (4) 3.1.f - Min. conditions for criticality.
 - (5) 3.2 - CVCS.
 - (6) 3.3 - Engineered safety features and auxiliary systems.

- (7) 3.4 - Steam and power conversion.
- (8) Table 3.5.1 - Engineered safety system trip settings.
- (9) 6.2 - Actions in event of AO.
- (10) 6.1.c.1 - Plant Operations Review Committee (PORC).
- (11) 6.1.c.2 - Nuclear Safety Review and Audit Committee (NSRAC).
- d. Review of plant AO files, discussion of AO resolutions.
- e. Review of plant files on PORC and NSRAC meeting minutes.
- f. Review of plant reactor operator and senior reactor operator requalification tests.
- g. Followup items:
 - (1) ASCO valves.
 - (2) Procedure for setting Limitorque switches and for sizing thermal overload devices.
 - (3) Training - retraining of nonlicensed personnel.

4. Enforcement Items

In a review of the management actions taken in the event of an abnormal occurrence the inspector noted certain inconsistencies with the applicable technical specification. First, Section 6.2, Action to be Taken in the Event of an Abnormal Occurrence in Plant Operation, states: "Any abnormal occurrences shall be reported immediately to ----- the Superintendent - Nuclear Power -----". Records reviewed establish that plant management has, in each case, been notified immediately, however, these records do not substantiate that the Superintendent - Nuclear Power (Corporate Office) is notified immediately as required. The licensee stated that in some instances, such as instrument drift or a sticking solenoid in a redundant channel, the Superintendent - Nuclear Power might not be notified immediately, but that in all cases the Superintendent - Nuclear Power receives a complete written report of each abnormal occurrence within a few days of the occurrence.

Secondly, Section 6.2 states in part: "The Superintendent - Steam Plants shall notify the NRC within 24 hours -- of the circumstances of any abnormal occurrence." Records reviewed

establish that the NRC has been notified of each abnormal occurrence within 24 hours as required, however, these notifications are usually made by the Plant Superintendent rather than by the Superintendent -- Steam Plant as required. The licensee stated that they believe that their reporting practices meet the intent of the Technical Specifications, that being prompt and accurate notification to the Nuclear Regulatory Commission of all abnormal occurrences. The licensee believes that this reporting function is best accomplished by plant management, who are directly involved in the day by day operation of the plant, rather than by members of the corporate office management staff.

Proposed changes to the Technical Specifications were submitted in January of 1975 for both of the above items.

5. Abnormal Occurrences

Failure of Containment Isolation Valves on Process Sample Lines to Close

On June 12, 1975, while performing special functional tests on containment isolation valves pilot valves (ASCO solenoid valves) two containment isolation valves on process sample lines failed to close. In both instances the ASCO pilot valve functioned properly to cause the process sample valves to close, however, in both cases the process sample valve failed to close due to a deposit buildup on the stem of the valves. Both were completely disassembled and cleaned, after which both valves functioned properly.

A review of the records establishes that immediately upon determining that these containment isolation valves were inoperable the licensee closed and tagged redundant isolation valves to assure that containment integrity remained intact. Both valves were subsequently corrected and returned to normal service on June 13, 1975.

6. Followup Items

- a. Previous reports 1/2/ have identified a possible problem with a specific series of model of an ASCO solenoid valve. In response to this possible problem the licensee

1/ IE Inspection Rpt. No. 050-305/75-06.

2/ IE Inspection Rpt. No. 050-305/75-04.

has identified all such valves used in the plant and has initiated a program to determine the degree, if any, that the problem of incorrect gap and/or incorrect spring strength exists at this plant. To date approximately 25 percent of these valves have been checked and thus far none have been found to be incorrect. This program is continuing and will remain an open item until completion.

- b. In a previous report 3/ the licensee agreed to formalize their procedure for testing and adjusting Limitorque valve operators following maintenance or repair of the valve or operator. The licensee reports that this procedure has been drafted and is being revised following review and comment. This item remains open.
- c. A previous report 4/ indicated a need to provide a more formalized training - retraining program and training records for nonlicensed plant personnel.

The inspectors review of records and discussions with the plant training supervisor indicate that this has been accomplished. This item is closed.

7. Surveillance Testing

A plant management review of the surveillance program identified four containment sump isolation valves that were not included in the periodic surveillance testing program. These valves SI 350 A & B, and SI 351 A & B, were tested and verified to perform properly. The plant surveillance program is being revised to include the operability checks for these valves. The Nuclear Regulatory Commission was notified of this event.5/

8. Heater Drain Pump Coupling Failure

A heater drain pump, which has been out of service for several weeks due to the failure of the variable speed magnetic coupling, has been repaired and is scheduled to be returned to service during the June 19, 1975, outage. The loss of this pump has resulted in a power level penalty of approximately eleven percent.

- 3/ IE Inspection Rpt. No. 050-305/75-C4
- 4/ IE Inspection Rpt. No. 050-305/75-08
- 5/ Ltr, E. James to Bernard Rusche, 6/18/75.