CENTRAL FILES

UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
799 ROOSEVELT ROAD
GLEN ELLYN, ILLINOIS 60137

MAY 09 1978

Docket No. 50-305/78-04

Wisconsin Public Service
Corporation
ATTN: Mr. E. W. James
Senior Vice President
Power Generation and
Engineering
P.O. Box 1200
Green Bay, WI 54305

Gentlemen:

This refers to the inspection conducted by Mr. N. C. Choules of this office on March 30 and April 11-14, 1978, of activities at Kewaunee Nuclear Power Plant authorized by NRC Operating License No. DPR-43 and to the discussion of our findings with Mr. C. Luoma and others of your staff at the conclusion of the inspection.

The enclosed copy of our inspection report identifies areas examined during the inspection. Within these areas, the inspection consisted of a selective examination of procedures and representative records, observations, and interviews with personnel.

No items of noncompliance with NRC requirements were identified during the course of this inspection.

In accordance with Section 2.790 of the NRC's "Rules of Practice," Part 2, Title 10, Code of Federal Regulations, a copy of this letter and the enclosed inspection report will be placed in the NRC's Public Document Room, except as follows. If this report contains information that you or your contractors believe to be proprietary, you must apply in writing to this office, within twenty days of your receipt of this letter, to withhold such information from public disclosure. The application must include a full statement of the reasons for which the information is considered proprietary, and should be prepared so that proprietary information identified in the application is contained in an enclosure to the application.

GD

We will gladly discuss any questions you have concerning this inspection.

Sincerely,

Gaston Fiorelli, Chief
Reactor Operations and
Nuclear Support Branch

Enclosure: IE Inspection Rpt
No. 50-305/78-04

cc w/encl:
Mr. C. Luoma, Plant
Superintendent
Central Files
Reproduction Unit NRC 20b
PDR
Local PDR
NSIC
TIC

OFFICE▶	RIII MCC	RIII RFW	RIII
SURNAME▶	Choules/dec	Warnick	Fiorelli
DATE▶	5/6/78		

U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT

REGION III

Report No. 50-305/78-04

Docket No. 50-305 License No. DPR-43

Licensee: Wisconsin Public Service Corporation
P. O. Box 1200
Green Bay, WI 54305

Facility Name: Kewaunee Nuclear Power Plant

Inspection At: Kewaunee Site, Kewaunee, WI

Inspection Conducted: March 30, April 11-14, 1978

Inspector: N. C. Choules N C Choules 5/9/78

Approved by: R. F. Warnick, Chief RF Warnick 5/8/78
Reactor Projects Section 2

Inspection Summary

Inspection on March 30, April 11-14, 1978 (Report No. 50-305/78-04)
Areas Inspected: Routine, announced inspection of calibrations, IE Bulletin followup, nonroutine event followup, procedures, implementation of procedures and controls for 10 CFR Part 21, and independent inspection. The inspection involved 27 inspector-hours onsite by one NRC inspector. The inspector was accompanied by Mr. Young S. Hahn of the Korean Atomic Energy Bureau.
Results: No items of noncompliance or deviations were identified.

DETAILS

1. Persons Contacted

Plant Site

*C. R. Luoma, Plant Superintendnet
C. R. Steinhardt, Assistant Superintendent
*R. W. Lange, Assistant Superintendent, Maintenance
J. S. Richmond, Technical Supervisor
W. S. Truttman, Operations Supervisor
*M. L. Marchi, Nuclear Systems Engineer
D. M. McSwain, Instrument and Control Supervisor

Corporate Office

D. Hintze, Nuclear Services Supervisor

*Denotes those present at exit interview.

2. Calibration Required by Technical Specifications

a. Primary Calibration Standards

(1) Mansfield and Green Dead Weight Tester, Serial No. 6399.

(2) Hewlett-Packard AC/DC Ohms Converter, Serial No. 951-00373.

(3) Hewlett-Packard Digital Voltmeter, Serial No. 977-01104.

(4) Hewlett-Packard High Resistance Meter, Serial No. 948-0662.

All the above instruments have been calibrated annually as prescribed by the licensee. Documentation was available which indicated that the calibrations are traceable to the National Bureau of Standards. Storage of the above instruments appears to be adequate.

b. Component Instrumentation

The inspector reviewed the previous year's calibration records for the following component instrumentation and determined that instruments were being calibrated and tested using procedures as required by the licensee and the Technical Specifications.

Surveillance Procedure Number (SP)	Procedure Title
010	Reactor Coolant Temperature Instrument Channel Test
011	Reactor Coolant Temperature Instrument Calibration
017	Pressurizer Level Instrument Calibration
030	Steam Generator Pressure Instrument Channel Test
031	Steam Generator Pressure Instrument Calibration
040	Refueling Water Storage Tank Level Instrument Calibration
059	Turbine First Stage Pressure Instrument Calibration
133	Seismic Monitoring System Check and Calibration
147	Boric Acid Tank Level Instrument Test

Review of the calibration procedures indicated that satisfactory calibration of related components will result. Trip points, where applicable, comply with Technical Specifications requirements.

The inspector witnessed the performance of calibration checks per SPs 031 and 147. No discrepancies were noted in the performance of these checks.

c. I&C Technicians

The inspector reviewed the qualifications of two I&C repairmen and verified they were in accordance with ANSI Standard 18.1.

3. Calibrations Not Required by Technical Specifications

The inspector reviewed the licensee's program for the calibration of equipment associated with reactor safety, but not

identified in Technical Specifications as requiring specific calibrations. The licensee has identified the instruments in the above category and has prepared calibration procedures. The inspector reviewed past calibration records and procedures for the following systems and equipment.

a. Safety Injection - Instrument Control Procedure (ICP) 33

(1) Flow Indicator, FI 925

(2) Pressure Indicator, PI 929

b. Spent Fuel Cooling - ICP 21

(1) Temperature Indicator, TI 12007

(2) Temperature Indicator, TI 12012

c. Component Cooling - ICP 31

(1) Temperature Indicator, TI 604

(2) Pressure Indicator, PI 617A

d. Chemical and Volume Control - ICP 35

(1) Flow Transmitter, FT 134

(2) Flow Indication Alarm, FIA 134

e. Auxiliary Feedwater - ICP 5

Flow Indicator, FI 18201

f. Service Water - ICP 2

(1) Pressure Indicator, PI 11332

(2) Differential Pressure Indicator, DPI 11086

(3) Differential Pressure Switch, DPS 16422

No items of concern were identified in the review of procedures and calibration records.

4. IE Bulletin Followup

IEB 78-04 - The inspector verified by discussion and review of records and procedures that the licensee had reviewed the bulletin and had taken the action as indicated in his reply.[1/]

5. Reportable Occurrences

The following reportable occurrences were reviewed by examination of logs, records and through discussions with plant personnel. Occurrences were reviewed for completion of reporting requirements, investigation and determination of cause, proposed corrective measures, and completion of corrective action.

a. RO 50-305/77-02[2/] - Head correction for pressurizer pressure transmitters not applied.

b. RO 50-305/77-04[3/] and 77-05[4/] - Component cooling water pump failed to start.

c. RO 50-305/77-06[5/] - One of two blowdown line containment isolation valves would not shut completely.

As indicated in the licensee's report, the valve failed to close completely due to dry hard packing. As a result of this, the licensee is considering periodic replacement of packing for valves of this type.

d. RO 50-305/78-07[6/] - One of three pressurizer level channels SI trip setpoint below that required by Technical Specifications.

e. RO 50-305/78-08[7/] - Containment isolation valve on the excess heat exchanger return line failed to shut completely.

1/ Ltr, WPS to RIII, dtd 3/23/78.
2/ LER 50-305/78-02, WPS to RIII, dtd 2/20/78.
3/ LER 50-305/78-04, WPS to RIII, dtd 1/15/78.
4/ LER 50-305/78-05, WPS to RIII, dtd 1/18/78.
5/ LER 50-305/78-06, WPS to RIII, dtd 3/3/78.
6/ LER 50-305/78-07, WPS to RIII, dtd 3/8/78.
7/ LER 50-305/78-08, WPS to RIII, dtd 3/30/78.

In this event, the torque switch contacts were oxidized and prevented the valve from closing. The inspector suggested that the torque switch contacts be periodically inspected and cleaned if required. The licensee's representative stated that the valves are periodically inspected, and if the contacts are not included in this inspection, the procedure for inspection will be revised to include the contacts.

f. RO 50-305/78-09[8/] - Train "B" containment sump isolation valve would not fully open.

In reviewing this occurrence with the licensee, the licensee's representative indicated this valve had, since the licensee's report was issued, again failed to open on the first try. The licensee believes that the valve packing is causing the problem and plans to replace the packind during the refueling outage starting April 22, 1978. The valve is located inside a housing which is part of containment and is not accessible during normal operation.

Review of these occurrences indicated the licensee's corrective actions or proposed corrective actions appear to be adequate.

5. Procedures

a. The following procedures were reviewed by the inspector.

(1) Operating Procedures

N-EDC-38, Rev. B - DC Supply and Distribution System
N-EDC-38B, Rev. A - Operations of Static Inverters

(2) Maintenance and Preventive Maintenance Procedures

PMP 1-3, Rev. E - Station Air Compressor 1A, 1B and 1C Maintenance

PMP 2-5, Rev. D - Service Water Pump Maintenance

PMP 5B-1, Rev. D - Auxiliary Feedwater Pump Maintenance

PMP 25-5, Rev. C - Damper Maintenance

PMP 33-1, Rev. D - Safety Injection Pump Inspection and Lubrication

8/ LER 50-305/78-09, WPS to RIII, dtd 3/30/78.

PMP 36-2, Rev. 2 - Reactor Coolant Pump Maintenance

PMP 38-1, Rev. E - Battery and Battery Charger Maintenance

CMP 1-1 - Instrument Air Filter Cleaning

CMP 5B-2 - Auxiliary Feedwater Pump and Auxiliary Lube Oil Pump Overhaul

CMP 33-1 - Safety Injection Pump Overhaul

CMP 36-2 - Reactor Coolant Pump Controlled Leakage Seals

CMP 38-1, Rev. A - Station Battery 1B Ground Search

(3) Administrative Procedures

ACD 4.5 - Shift Operation and Turnover
ACD 4.7 - Shift Supervisors Log
ACD 4.8 - Reactor and Control Room Log

b. The procedures above were reviewed to verify that the procedure review and approval and procedure changes were accomplished in accordance with the Technical Specifications.

Procedure N-EDC-38 and 38B were walked through with an operator. Certain steps in these procedures were not completely clear. The licensee's representative noted these and will revise these steps. No other items of concern were identified.

6. 10 CFR Part 21

The inspector reviewed the licensee's program to assure compliance with 10 CFR Part 21 at the corporate office and the plant.

a. Corporate Office

Administrative Procedure FCD 11.4, Evaluation of Potential Safety Hazards, Deviations or Defects, is the licensee's procedure to implement Part 21 requirements at the corporate office. In this procedure, the inspector noted that the following were not addressed.

(1) Posting requirements of 10 CFR 21.6.

(2) Controls to assure each procurement document for a basic component, when applicable, specifies the provisions of 10 CFR 21.31.

(3) Controls to assure maintenance and disposition of records in accordance with 10 CFR 21.51.

The licensee's representative stated that the above would be addressed in FCD 11.4 or other procedures.

The inspector verified that the licensee has met the posting requirement of Part 21. The inspector also verified that recent purchase orders contain an appropriate statement to comply with 10 CFR 21.31.

b. Plant

Administrative Procedure ACD 4.14 is the licensee's procedure which contains instructions to implement Part 21 requirements at the plant. In this procedure, the inspector noted that the posting requirements of 10 CFR 21.6 were not addressed. The licensee stated posting requirements would be specified.

The inspector verified that the licensee has met the posting requirements of Part 21. At the time of the inspection, the plant had not identified any potential Part 21 reports.

7. Other Inspection Items

In a previous inspection,[9/] the licensee indicated the Surveillance Procedure SP 102 would be revised to establish that all station battery electrical connections are tight. In place of including the above in SP 102, the licensee has included tightness checks in SP 38-182. The tightness check will be performed annually.

8. Exit Interview

The inspector met with licensee representatives (denoted in Paragraph 1) at the conclusion of the inspection on April 14, 1978.

The inspector summarized the scope and findings of the inspection.

9/ IE Inspection Rpt No. 50-305/77-22.