Contral file

WISCONSIN PUBLIC SERVICE CORPORATION



P.O. Box 1200, Green Bay, Wisconsin 54305

September 25, 1979

Mr. J. G. Keppler, Regional DirectorOffice of Inspection & EnforcementRegion IIIU. S. Nuclear Regulatory Commission799 Roosevelt RoadGlen Ellyn, IL 60137

Dear Mr. Keppler:

Docket 50-305 Operating License DPR-43 Additional Information for IE Bulletin 79-01

We have reviewed your request for additional information concerning response items to IE Bulletin 79-01. The cover letter for this request indicates that the letter had been previously transmitted by telecopy. We have not received a telecopy, and a review of our incoming telecopy log showed that no transmittal was received.

In response to Item 1, all power, control and instrument cable used in safeguards in containment at the Kewaunee Nuclear Power Plant are qualified post LOCA. Test reports are available to substantiate this. Specifically report B904 prepared by Boston Insulated Wire & Cable Company addresses testing of cables showered with a boric acid solution of pH 4.0 and adjusted to a pH of 10 with sodium hydroxide, and concludes that the subject cable exhibits negligible changes in physical properties which indicate their suitability for use in such environments.

Report F-C2737 from the Kerite Company similarly qualifies purchases of Kerite cable in a post LOCA environment. Likewise, test report 110E from Okonite qualifies cable purchased from them.

In reference to temperature qualification, please refer to IE Inspection 50-305/78-20 which addresses and closes out this concern.

Pressure qualification of electrical cable was not specifically addressed in either IE Circular 78-08 or IE Bulletin 79-01, and by itself is not a concern for the pressures exhibited post LOCA. Nevertheless, the test information referenced above indicate all tests were performed in elevated pressure, temperature and steam environments.

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Item 2 requests a considerable quantity of information that took in excess of six months to collect and review. All the items in FSAR table 6.2-13 were thoroughly reviewed by our staff and by Mr. Choules and Mr. Jablonski of your office. It is unreasonable to expect us to provide written documentation to substantiate the review performed in seven days. Since we believe our review to be complete and verified so by your inspectors, we do not intend to duplicate our efforts in written form. However, if there is still an open item not resolved after discussions with your inspectors, please inform us of the specifics and we will attempt to provide written documentation of that qualification.

Concerning Item 3 of your request, page 7.5-15 of the Kewaunee FSAR states "The maximum dose levels within the containment would be approximately $3.8(10)^6$ Rads during one hour . . .". This dose is the dose expected either in the reactor vessel cavity or in the reactor coolant pump vaults. The actual dose received by the transmitters would be reduced significantly by the shielding in this area. Documentation provided by the Initial Startup Report, 1974 shows a reduction of 2000 in the dose which would be received by this instrumentation. Table 6.2-13 of the FSAR requires operation of this instrumentation for a period of two hours maximum. Therefore, qualification to $1(10)^5$ Rads is justifiable.

Item 4 was transmitted in a letter to Mr. Keppler dated September 14, 1979.

Very truly yours,

E.R. Mathewa

E. R. Mathews, Vice President Power Supply & Engineering

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cc - Dir, Office of Inspection & Enforcement Division of Reactor Operations Inspection U S Nuclear Regulatory Commission Washington, D C 20555