WISCONSIN PUBLIC SERVICE CORPORATION



P.O. Box 1200, Green Bay, Wisconsin 54305

January 24, 1979

Mr. J. G. Keppler, Regional Director Office of Inspection & Enforcement Region III U. S. Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, IL 60137

Dear Mr. Keppler:

Docket 50-305 Operating License DPR-43 Reportable Occurrences LER 78-037/03L-0 and LER 79-001/01T-0

In accordance with the requirements of Technical Specifications, Section 6.9, the attached Licensee Event Reports for reportable occurrences LER 78-037/03L-0 and 79-001/01T-0 are being submitted.

Very truly yours,

E. W. James

Senior Vice President Power Supply & Engineering

 snf

Attach.

7901300103

cc - Dir, Office of Inspection & Enforcement US NRC, Washington, D. C. 20555 Dir, Office of Mgt Info & Program Control US NRC, Washington, D. C. 20555

JAN 26 1979

(7.77) LICENSEE EVENT REPORT (PLEASE PRINT OF TYP L REQUIRED INFORMATION) CONTROL BLOCK: |(1)0 0 0 0 0 0 - 0 0 3 4 1 1 1 1 UCENSE NUMBER 1 2 0 0 IKNP LICENSE NUMBER LICENSEE CODE CON'T 5 0 0 0 3 0 5 7 1 2 2 6 7 8 8 0 1REPORT L 6 0 0 1 SOURCE 60 EVENTDATE DOCKET NUMBER EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) During steady full power operation surveillance testing showed that one of the two 0 2 reactor trip breakers would not trip. This placed the plant under LCO T.S.3.5.b. 03 The other trip breaker was verified to be operable and the faulty breaker was re-0 4 paired and returned to service within two hours of discovery. There was no effect on 0 5 plant operation or public safety. 0 6 0 7 8 80 COMP. VALVE SYSTEM CAUSE CAUSE COMPONENT CODE SUBCODE SUBCODE SUBCODE CODE CODE Z (16) Т B R K (14 A (15 (12)B (13) C E K IAI (11 9 0 13 18 REVISION OCCUBRENCE REPORT SEQUENTIAL CODE TYPE NO. LER/RO EVENT YEAR REPORT NO. 10 3 01 0 3 L 7 (17) REPORT 8 32 NUMBER 30 28 PRIME COMP. SUPPLIER COMPONENT NPRD-4 ATTACHMENT ACTION FUTURE EFFECT ON PLANT SHUTDOWN METHOD HOURS (22) MANUFACTURER SUBMITTED FORM SUB. ĮΫ́ N 23 (25) W 11 12 18 <u>G</u> υ](24) | N Z (20) | Z |(21) 0 0 0 34 33 35 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) | The shaft of the trip breaker under voltage relay was sticking which prevented a 1 0 The UV relay shaft was cleaned and lubricated and the breaker was |breaker trip. 111 This breaker is being tested on a weekly basis for tested and returned to service. 1 2 The existing PM program is being revised to include one month to verify operability. 1 3 routine inspection of reactor trip breakers for cleanliness and proper lubrication. 4 80 9 8 METHOD OF DISCOVERY OTHER STATUS FACILITY STATUS DISCOVERY DESCRIPTION (32) % POWER Surveillance Testing | B |(31) |1 |0 |0 (29) NA E (28) 5 នព 45 44 46 CONTENT ACTIVITY LOCATION OF RELEASE (36) AMOUNT OF ACTIVITY (35) RELEASED_OF RELEASE Z 33 Z 34 NA 'NA 6 80 10 11 PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER TYPE 01 0 (37) Z (38) NA 7 80 11 12 PERSONNEL INJURIES DESCRIPTION (41) 7901300106 NUMBER 0 0 (40) NA 8 80 11 12 LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION Z (42) NA 80 10 PUBLICITY NRC USE ONLY DESCRIPTION (45) ISSUED N (44) NA 0 68 80 5 69 10 000 (414)433-1329 G. H. Ruiter PHONE: NAME OF PREPARER.

U. S. NUCLEAR REGULATORY COMMISSION NRC FORM 366 (7.77) • LICENSEE EVENT REPORT (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK: $\begin{array}{|c|c|c|c|c|c|} \hline 1 \\ \hline 14 \\ \hline 15 \\ \hline 16 \\ \hline 16$ CON'T L 6 0 5 0 0 3 0 5 7 0 1 1 0 7 9 8 0 1 2 4 7 9 9 0 0 0 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80 REPORT 0 1 SOURCE L EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) During steady full power operation D/G 1A was started to verify operability prior to 0 2 placing D/G 1B out of service for maintenance. Approximately 80 minutes later, with 0 3 D/G 1B 00S, a D/G 1A Start-Run-Failure alarm was received indicating loss of control 0.4 With both D/G's OOS, the facility was in a condition less conservative than power. 0 5 permitted by LCO T.S.3.7.b.2. Unit backdown to hot standby was commenced. D/G 1A06 was returned to service in about 1/2 hour and the unit was returned to full power 0 7 operation. 8 80 8 q COMP VALVE CAUSE SYSTEM CAUSE COMPONENT CODE SUBCODE SUBCODE SUBCODE CODE Z [(15 X | X | X | X | (14) ^Z| (16) F | (13) XI E |E |(12) | X E (11) 18 REVISION OCCURRENCE REPORT SEQUENTIAL REPORT NO CODE NO. EVENT YEAR 0 1 LER BO 0 REPORT NUMBER 28 ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT SHUTDOWN METHOD ACTION FUTURE TAKEN ACTION EFFECT ON PLANT HOURS (22) MANUFACTURER 9 9 9 01 N 23 Y (24) 0 0 0 A (25) (26) Z (21) Z (20) C (18) CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) A blown fuse in D/G 1A control cabinet caused loss of control power. No circuitry 1 0 problems were found and a new fuse was installed without incident; D/G 1A was tested 1] and returned to service. Since no electrical system problems were found, this occur-1 2 rence is attributed to normal fuse performance and no further actions are planned. 3 NRC, Region III was notified via telegram within 24 hours per TS 6.9.2.a.2. 4 80 9 8 METHOD OF OTHER STATUS FACILITY DISCOVERY DESCRIPTION (32) DISCOVERY % POWER A (31) Control Room Annunciation 0 0 (29) NA 80 46 9 10 ACTIVITY CONTENT LOCATION OF RELEASE (36) AMOUNT OF ACTIVITY (35) RELEASED_OF RELEASE NA NA 6 80 10 11 PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER TYPE 0 0 0 37 Z 38NA 63 11 12 PERSONNEL INJURIES 7901300109 DESCRIPTION (41) NUMBER 0 0 0 (40) NA 8 80 11 12 LOSS OF OR DAMAGE TO FACILITY (43) DESCRIPTION TYPE Z (42) 9 NA 80 10 NRC USE ONLY PUBLICITY DESCRIPTION (45) N (A) NA 0 80 6 68 69 (414) 433 - 1329000 G. H. Ruiter PHONE:-NAME OF PREPARER -