NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL						50-305
						FILE NUMBER INCIDENT REPORT
TO: Mr J G Keppler		FROM:		consin Public Service Corp en Bay, Wis James		DATE OF DOCUMENT 3-4-77
						DATE RECEIVED 3-7-77
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						1cc
DESCRIPTION				ENCL	OSU RE .	•
Ltr trans the following		Licensee Event Report (RO# 77-7) on 2-18-77 concerning seals on door actuation shafts on outer door of dual personnel air locks leaking in excess of testing measurement capability				
,					2p	NOT REMOVE
PLANT NAME: Kewaunne	e					A TOTAL DON'T
•						
*				NOT		XPOSURE IS INVOLVED TO KREGER/J. COLLINS
				<u></u>		
				INFOR	RMATION 3-11-	77 ehf
BRANCH CHIEF: W/3 CYS FOR ACTION	Schw	encer	· 			
LIC. ASST.:	Sha	eppara	$t^{-}$			
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KREGER/J. COLLINS						
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NRC EORM 195 (2-76)

legulatory Docket File

## WISCONSIN PUBLIC SERVICE CORPORATION



P.O. Box 1200, Green Bay, Wisconsin 54305

Mr. J. G. Keppler, Regional Director Office of Inspection & Enforcement Region III U. S. Nuclear Regulatory Commission 799 Roosevelt Road

60137

Dear Mr. Keppler:

Glen Ellyn, IL

Subject: Docket 50-305

Operating License DPR-43 Reportable Occurrence March 4, 1977

MAR 7 1977

MAR 7 1977

Uz Sz NUCLEAR REGULATORY

COMMISSION

Adolf Section

Adolf Section

In accordance with the requirements of Technical Specifications, Section 6.9.2.b, the attached Licensse Event Report for reportable occurrence RO 77-7 is being submitted. Initially, this report was transmitted via telegram in 24 hours in accordance with Section 6.9.2.a of Technical Specifications. However, after reviewing the information available, this incident has been reevaluated to be a reduction in the degree of redundancy provided by this safety feature and, therefore, reportable as a 30-day reportable occurrence in accordance with Technical Specifications, Section 6.9.2.b. We have discussed this reevaluation with Mr. Hunter of your office via telephone on March 1, 1977.

Very truly yours,

E. W. James

Senior Vice President
Power Supply & Engineering

EWJ:sna Enc.

cc - Dir, Office of Inspection & Enforcement

US NRC, Washington, D. C. 20555

Dir, Office of Management Info & Program Control

US NRC, Washington, D. C. 20555



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01 7 8	LICENSEE NAME  LICENSE NUMBER  LICENSE NUMBER  LICENSE TYPE  TYPE  14 1 1 1 1 1 9 9 9  14 15 25 26 30 31 32	
01 7 8	CATEGORY TYPE SOURCE DOCKET NUMBER EVENT DATE REPORT DATE  ON'T P O L L L 0 5 0 - 0 3 0 5 0 2 1 8 7 7 0 3 0 4 7 5 57 58 59 60 61 68 69 74 75	<b>7</b> 80
02	EVENT DESCRIPTION  During the pressure test of the dual personnel air lock doors, the seals on the door	_ B0
03	actuating shafts on the outer door were found to be leaking in excess of the tester	
7 8	measuring capability. After adjusting the packing on the seals on the outer door	90
7 8 05	only, a retest showed the leakage to be minimal and verified that the inner airlock	80
7 8 06	door did not leak. Normal operating practice requires that both airlock doors (Cont.	
7 B	SYSTEM CAUSE COMPONENT CODE COMPONENT SUPPLIER MANUFACTURER VIOLATION  S A E P E N E T R A C 3 1 0 N  O 11 12 17 43 44 47 48  CAUSE DESCRIPTION	BO
08	The shaft packing on the door actuating device had become loose through normal usage.	لِ
7 8 09	The packing was retightened and the seal was tested satisfactorily.	80
7 8		90
7 8 111 7 8	FACILITY STATUS % POWER OTHER STATUS DISCOVERY DESCRIPTION  H 0 0 0 NA B Local Leak Rate Testing  9 10 12 13 44 45 46	80
12 7 8	FORM OF ACTIVITY CONTENT AMOUNT OF ACTIVITY LOCATION OF RELEASE  Z Z NA NA NA 9 10 11 44 45  PERSONNEL EXPOSURES	80
13 7 8	NUMBER TYPE DESCRIPTION  O O O Z NA	 80
14 7 8	PERSONNEL INJURIES  NUMBER DESCRIPTION  O O O NA  3 11 12	 80
15 7 8	OFFSITE CONSEQUENCES  NA	 80
16	LOSS OR DAMAGE TO FACILITY  TYPE DESCRIPTION  Z NA	لِـــ
7 8	9 10 PUBLICITY	80
17	NA NA	80
18 7 8	ADDITIONAL FACTORS  The shaft seals were tested satisfactory in 1973 and 1976, and the annual refueling	_ 
19	test should be adequate to determine seal degradation.	لِ
7 8	March 7 March 4 616/632=3311	80

## Event Description (Cont.)

are kept shut except during the brief period of time someone is entering or exiting containment, thus during an accident the one door would have been able to maintain the integrity of the containment boundary within the acceptance criteria specified in section 4.4 of Technical Specifications. (RO 77-7)