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SUBJECT: Responds to Generic Ltr 89-21 re status of implementation of  
 USI requirements.

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November 27, 1989

10 CFR 50.54(f)

U. S. Nuclear Regulatory Commission  
 ATTN: Document Control Desk  
 Washington, D.C. 20555

Gentlemen:

Docket 50-305  
 Operating License DPR-43  
 Kewaunee Nuclear Power Plant  
Unresolved Safety Issues

Reference: 1) Generic Letter 89-21 dated October 19, 1989

This letter provides Wisconsin Public Service Corporation's (WPSC) response to Reference 1 which requested information concerning the status of implementation of unresolved safety issue (USI) requirements.

In accordance with the instructions provided in Reference 1, a "C" has been marked in the Status column for items which are complete, along with the implementation date. An "I" has been marked in the Status column for items which are not fully implemented, along with the projected implementation date. "N/A" has been marked in the Status column for items not applicable to the Kewaunee Nuclear Power Plant (KNPP). "N/C" has been marked in the Status column for items applicable to KNPP, but with no changes necessary.

References to any supporting documentation have been included in the Remarks column.

The results of our documentation search concludes that of the 27 unresolved safety issues referred to in Reference 1, WPSC has completed 10, continues to implement 2 issues, 11 are not applicable to KNPP, and finally, 4 were identified as applicable to KNPP but with no changes necessary.

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Document Control Desk  
November 27, 1989  
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If you have any questions or require additional information, please contact me or Ms. B. J. De Cleene of my staff.

Sincerely,

A handwritten signature in cursive script, appearing to read "K. H. Evers".

K. H. Evers  
Manager - Nuclear Power

BJD/jms

Enc.

cc - Mr. Patrick Castleman, US NRC  
US NRC, Region III

UNRESOLVED SAFETY ISSUES FOR WHICH A FINAL TECHNICAL RESOLUTION HAS BEEN ACHIEVED

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-1	Water Hammer	SECY 84-119 NUREG-0927, Rev. 1 NUREG-0993, Rev. 1 NUREG-0737 Item I.A.2.3 SRP revisions	A11	C 3/76	E.W. James (WPSC) to R.A. Purple (NRC) dated 3/10/76. Also ref. NRC SERs dated 9/5/79 and 9/13/79.
A-2/ MPA D-10	Asymmetric Blowdown Loads on Reactor Primary Coolant Systems	NUREG-0609 GL 84-04, GDC-4	PWR	C 2/88	NRC SER dated 2/16/88.
A-3	Westinghouse Steam Generator Tube Integrity	NUREG-0844 SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	W-PWR	C 6/85	D.C. Hintz (WPSC) to H.L. Thompson (NRC) dated 6/17/85
A-4	CE Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	CE-PWR	N/A	N/A
A-5	B&W Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	B&W-PWR	N/A	N/A
E A-6	Mark I Containment Short-Term Program	NUREG-0408	Mark I-BWR	N/A	N/A

\*C - COMPLETE  
N/C - NO CHANGES NECESSARY  
N/A - NOT APPLICABLE  
I - INCOMPLETE  
E - EVALUATING ACTIONS REQUIRED

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>		<u>REMARKS</u>
A-7/ D-01	Mark I Long-Term Program	NUREG-0661 NUREG-0661 Suppl. 1 GL 79-57	Mark I-BWR	N/A		N/A
A-8	Mark II Containment Pool Dynamic Loads	NUREG-0808 NUREG-0487, Suppl. 1/2 NUREG-0802 SRP 6.2.1.1C GDC 16	Mark II-BWR	N/A		N/A
A-9	Anticipated Transients Without Scram	NUREG-0460, Vol. 4 10 CFR 50.62	All	C	10/89	NRC SER dated 8/25/88 NRC Inspection Report 89013 dated 10/25/89.
A-10/ MPA B-25	BWR Feedwater Nozzle Cracking	NUREG-0619 Letter from DG Eisenhut dated 11/13/80 GL 81-11	BWR	N/A		N/A
A-11	Reactor Vessel Material Toughness	NUREG-0744, Rev. 1 10 CFR 50.60/ GL 82-26	All	N/C		No licensee response required. (Reviewed internally)
A-12	Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports	NUREG-0577, Rev. 1 SRP Revision 5.3.4	PWR	N/A		Only applies to plants with a new constr. permit issued after 10/83
A-17	Systems Interactions	Ltr: DeYoung to licensees - 9/72 NUREG-1174, NUREG-1229, NUREG/CR-3922, NUREG/ CR-4261, NUREG/CR-4470, GL 89-18 (No requirements)	All	N/C		No requirements. (Reviewed internally)
A-24/ MPA B-60	Qualification of Class 1E Safety-Related Equipment	NUREG-0588, Rev. 1 SRP 3.11 10 CFR 50.49 GL 82-09, GL 84-24 GL 85-15	All	C	1/85	D.C. Hintz (WPSC) to D.G. Eisenhut (NRC) dated 1/31/85. NRC SER dated 2/2/83.

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-26/ MPA B-04	Reactor Vessel Pressure Transient Protection	DOR Letters to Licensees 8/76 NUREG-0224 NUREG-0371 SRP 5.2 GL 88-11	PWR	C 11/88	C.R. Steinhardt (WPSC) to Doc. Control Desk (NRC) dated 11/1/88
A-31	Residual Heat Removal Shutdown Requirements	NUREG-0606 RG 1.113, RG 1.139 SRP 5.4.7	All OLs After 01/79.	N/A	N/A
A-36/ C-10, C-15	Control of Heavy Loads Near Spent Fuel	NUREG-0612 SRP 9.1.5 GL 81-07, GL 83-42, GL 85-11 Letter from DG Eisenhut dated 12/22/80	All	C 6/85	Phase I report dated 12/23/82. Phase II report dated 3/9/83. H.L. Thompson (NRC) to all Licensees, dated 6/28/85
A-39	Determination of SRV Pool Dynamic Loads and Pressure Transients	NUREG-0802 NUREGs-0763, 0783, 0802 NUREG-0661 SRP 6.2.1.1.C	BWR	N/A	N/A
A-40	Seismic Design Criteria	SRP Revisions, NUREG/ CR-4776, NUREG/CR-0054 NUREG/CR-3480, NUREG/ CR-1582, NUREG/CR-1161, NUREG-1233, NUREG-4776 NUREG/CR-3805 NUREG/CR-5347 NUREG/CR-3509	All	N/C	No licensee response required. (Reviewed internally)
A-42/ MPA B-05	Pipe Cracks in Boiling Water Reactors	NUREG-0313, Rev. 1 NUREG-0313, Rev. 2 GL 81-03, GL 88-01	BWR	N/A	N/A

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-43	Containment Emergency Sump Performance	NUREG-0510, NUREG-0869, Rev. 1 NUREG-0897, R.G.1.82 (Rev. 0), SRP 6.2.2 GL 85-22 No Requirements	All	N/C	No requirements. (Reviewed internally)
A-44	Station Blackout	RG 1.155 NUREG-1032 NUREG-1109 10 CFR 50.63	All	I	C.R. Steinhardt (WPSC) to NRC dated 4/17/89. (Awaiting NRC review of submittal)
A-45	Shutdown Decay Heat Removal Requirements	SECY 88-260 NUREG-1289 NUREG/CR-5230 SECY 88-260 (No requirements)	All	N/C	No requirements. This will be factored into IPE portion of severe accident program.
A-46	Seismic Qualification of Equipment in Operating Plants	NUREG-1030 NUREG-1211/ GL 87-02, GL 87-03	All	C 12/87	D.C. Hintz (WPSC) to Doc. Control Desk (NRC) dated 12/1/87. Implementation will be addressed through SQUG.
A-47	Safety Implication of Control Systems	NUREG-1217, NUREG- 1218 GL 89-19	All	I 3/90	WPSC response due 3/90
A-48	Hydrogen Control Measures and Effects of Hydrogen Burns of Safety Equipment	10 CFR 50.44 SECY 89-122	All, except PWRs with large dry containments	N/A	N/A
A-49	Pressurized Thermal Shock	RGs 1.154, 1.99 SECY 82-465 SECY 83-288 SECY 81-687 10 CFR 50.61/ GL 88-11	PWR	C 5/89	NRC SER dated 5/26/89.