

AUGUST 28 1978

DISTRIBUTION

Docket No. 50-305

~~ORXXXXXX~~

ORB1 Rdg

A. Schwencer

C. Parrish

D. Eisenhut

Gray Files (1)

Extra Copies (5)

Docket No. 50-305

Wisconsin Public Service Corporation

ATTN: Mr. E. W. James

Senior Vice President

Post Office Box 1200

Green Bay, Wisconsin 54305

Gentlemen:

RE: OVERPRESSURE PROTECTION TECHNICAL SPECIFICATIONS

In our earlier discussions with you regarding your overpressure protection system, we indicated that Technical Specifications would be necessary to ensure the operability of the system and to require the disabling of any equipment, such as charging pumps or safety injection pumps. Such specifications will ensure that the Appendix G limits for your reactor vessel will not be exceeded in the event of a pressure transient, assuming a single failure.

The enclosed model Technical Specifications are being provided for your guidance in preparation of your submittal for the Kewaunee Nuclear Power Plant, Unit 1 facility. You are requested to submit your proposed Technical Specifications within 45 days of receipt of this letter.

This license amendment to incorporate the proposed Technical Specifications has been determined to be a Class III Amendment. You should submit the appropriate fee.

Sincerely,

ORIGINAL SIGNED

A. Schwencer, Chief
Operating Reactors Branch # 1
Division of Operating Reactors

Enclosure:
Model Technical Specifications

cc w/enclosure:
See next page

MA 4
60

OFFICE	DOR:ORB1	DOR:ORB1	DOR:ORB1		
SURNAME	L01shan:jd	ASchwencer	Gzech		
DATE	8/25/78	8/25/78	8/25/78		



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

August 28, 1978

Docket No. 50-305

Wisconsin Public Service Corporation
ATTN: Mr. E. W. James
Senior Vice President
Post Office Box 1200
Green Bay, Wisconsin 54305

Gentlemen:

RE: OVERPRESSURE PROTECTION TECHNICAL SPECIFICATIONS

In our earlier discussions with you regarding your overpressure protection system, we indicated that Technical Specifications would be necessary to ensure the operability of the system and to require the disabling of any equipment, such as charging pumps or safety injection pumps. Such specifications will ensure that the Appendix G limits for your reactor vessel will not be exceeded in the event of a pressure transient, assuming a single failure.

The enclosed model Technical Specifications are being provided for your guidance in preparation of your submittal for the Kewaunee Nuclear Power Plant, Unit 1 facility. You are requested to submit your proposed Technical Specifications within 45 days of receipt of this letter.

This license amendment to incorporate the proposed Technical Specifications has been determined to be a Class III Amendment. You should submit the appropriate fee.

Sincerely,

A handwritten signature in cursive script, appearing to read "A. Schwencer".

A. Schwencer, Chief
Operating Reactors Branch # 1
Division of Operating Reactors

Enclosure:
Model Technical Specifications

cc w/enclosure:
See next page

Wisconsin Public Service Corporation - 2 -

cc: Steven E. Keane, Esquire
Foley, Sammond & Lardner
777 East Wisconsin Avenue
Milwaukee, Wisconsin 53202

Bruce W. Churchill, Esquire
Shaw, Pittman, Potts & Trowbridge
1800 M Street, NW.
Washington, D. C. 20036

Kewaunee Public Library
314 Milwaukee Street
Kewaunee, Wisconsin 54216

EMERGENCY CORE COOLING SYSTEMS

ECCS SUBSYSTEMS - $T_{avg} < 350^{\circ}\text{F}$

LIMITING CONDITION FOR OPERATION

3.5.3 A maximum of one centrifugal charging pump and one high pressure injection pump shall be OPERABLE whenever the temperature of one or more of the RCS cold legs is $< (275)^{\circ}\text{F}$.

SURVEILLANCE REQUIREMENTS

4.5.3 All charging pumps and safety injection pumps except those otherwise required to be OPERABLE shall be demonstrated inoperable at least once per 12 hours whenever the temperature of one or more of the RCS cold legs is $< (275)^{\circ}\text{F}$ by verifying the motor circuit breakers have been removed from their electrical power supply circuits.

EMERGENCY CORE COOLING SYSTEMS

BASES

The limitation for a maximum of one high pressure safety injection pump to be OPERABLE and the Surveillance Requirement to verify all high pressure safety injection pumps except the required OPERABLE pump to be inoperable below (275)°F provides assurance that a mass addition pressure transient can be relieved by the operation of a single PORV.

REACTIVITY CONTROL SYSTEMS

CHARGING PUMP - SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.1.2.3 One charging pump in the boron injection flow path required by Specification (3.1.2.1) shall be OPERABLE and capable of being powered from an OPERABLE emergency bus.

APPLICABILITY: MODES 5 and 6.

SURVEILLANCE REQUIREMENTS

4.1.2.3 All charging pumps, except the above required OPERABLE pump, shall be demonstrated inoperable at least once per 12 hours verifying that the motor circuit breakers have been removed from their electrical power supply circuits.

3/4.1 REACTIVITY CONTROL SYSTEMS

BASES

3/4.1.1 BORATION SYSTEMS

The limitations for a maximum of one centrifugal charging pump to be OPERABLE and the Surveillance Requirement to verify all charging pumps except the required OPERABLE pump to be inoperable below (275)°F provides assurance that a mass addition pressure transient can be relieved by the operation of a single PORV.

REACTOR COOLANT SYSTEM

OVERPRESSURE PROTECTION SYSTEMS

LIMITING CONDITION FOR OPERATION

3.4.9.3 At least one of the following overpressure protection systems shall be OPERABLE:

- a. Two power operated relief valves (PORVs) with a lift setting of \leq (450) psig, or
- b. A reactor coolant system vent of \geq () square inches.

APPLICABILITY: When the temperature of one or more of the RCS cold legs is \leq (275) $^{\circ}$ F.

ACTION:

- a. With one PORV inoperable, either restore the inoperable PORV to OPERABLE status within 7 days or depressurize and vent the RCS through a () square inch vent(s) within the next 8 hours; maintain the RCS in a vented condition until both PORVs have been restored to OPERABLE status.
- b. With both PORVs inoperable, depressurize and vent the RCS through a () square inch vent(s) within 8 hours; maintain the RCS in a vented condition until both PORVs have been restored to OPERABLE status.
- c. In the event either the PORVs or the RCS vent(s) are used to mitigate a RCS pressure transient, a Special Report shall be prepared and submitted to the Commission pursuant to Specification 6.9.2 within 30 days. The report shall describe the circumstances initiating the transient, the effect of the PORVs or vent(s) on the transient and any corrective action necessary to prevent recurrence.
- d. The provisions of Specification 3.0.4 are not applicable.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS

4.4.9.3.1 Each PORV shall be demonstrated OPERABLE by:

- a. Performance of a CHANNEL FUNCTIONAL TEST on the PORV actuation channel, but excluding valve operation, within 31 days prior to entering a condition in which the PORV is required OPERABLE and at least once per 31 days thereafter when the PORV is required OPERABLE.
- b. Performance of a CHANNEL CALIBRATION on the PORV actuation channel at least once per 18 months.
- c. Verifying the PORV isolation valve is open at least once per 72 hours when the PORV is being used for overpressure protection.
- d. Testing in accordance with the inservice test requirements for ASME Category C values as specified in Section XI of the ASME Boiler and Pressure Vessel Code 1974 Edition, and Addenda through Winter 1975.

4.4.9.3.2 The RCS vent(s) shall be verified to be open at least once per 12 hours* when the vent(s) is being used for overpressure protection.

*Except when the vent pathway is provided with a valve which is locked, sealed, or otherwise secured in the open position, then verify these valves open at least once per 31 days.

3/4.4 REACTOR COOLANT SYSTEM

3/4.4.1 REACTOR COOLANT LOOPS

NORMAL OPERATION

LIMITING CONDITION FOR OPERATION

A reactor coolant pump shall not be started with one or more of the RCS cold leg temperature $\leq (275)^{\circ}\text{F}$ unless 1) the pressurizer water volume is less than _____ cubic feet or 2) the secondary water temperature of each steam generator is less than _____ $^{\circ}\text{F}$ above each of the RCS cold leg temperatures.

REACTOR COOLANT SYSTEM

BASES

The OPERABILITY of two PORVs or an RCS vent opening of greater than () square inches ensures that the RCS will be protected from pressure transients which could exceed the limits of Appendix G to 10 CFR Part 50 when one or more of the RCS cold legs are $\leq (275)^{\circ}\text{F}$. Either PORV has adequate relieving capability to protect the RCS from overpressurization when the transient is limited to either (1) the start of an idle RCS with the secondary water temperature of the steam generator $\leq ()^{\circ}\text{F}$ above the RCS cold leg temperature or (2) the start of a HPSI pump and its injection into a water solid RCS.