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SUBJECT: Forwards notarized proposed Amend 101A to License DPR-43, inadvertently omitted from 910625 transmittal.

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Gentlemen:

Docket 50-305
Operating License DPR-43
Kewaunee Nuclear Power Plant
Correction to Proposed Amendment 101A
To the Kewaunee Nuclear Power Plant Technical Specifications

Reference: 1) Letter from C. R. Steinhardt (WPSC) to US NRC Document Control Desk, dated June 25, 1991

In accordance with the requirements of 10 CFR 50.30(b) proposed amendments to Technical Specifications shall be signed and notarized. However, the referenced submittal was inadvertently transmitted without being notarized. Attachment 1 contains a corrected copy of the referenced document which supersedes the previous transmittal.

WPSC apologizes for any inconvenience this may have caused.

Sincerely,

RA Kneeque for

Clark R. Steinhardt
Senior Vice President - Nuclear Power

BJD/jms
Attach.
cc - US NRC - Region III
Mr. Patrick Castleman, US NRC
Mr. R. S. Cullen, PSCW

LIC\NRC\N492

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ATTACHMENT 1

to

Letter from C. R. Steinhardt (WPSC) to Document Control Desk (NRC)

dated

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June 25, 1991 10 CFR 50.90

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Gentlemen:

Docket 50-305
Operating License DPR-43
Kewaunee Nuclear Power Plant
Proposed Amendment 101A to the Kewaunee
Nuclear Power Plant Technical Specifications

Reference: 1) Letter from C. R. Steinhardt (WPSC) to Document Control Desk (NRC) dated May 9, 1991

Proposed Amendment 101 (Reference 1) to the Kewaunee Nuclear Power Plant (KNPP) Technical Specifications was submitted to satisfy the commitments WPSC made in response to the recommendations set forth by the NRC in Generic Letter 90-06.

A subsequent review of this submittal by WPSC revealed that 1) the Table of Contents was inadvertently omitted, and 2) the second footnote listed on page TS 3.1.17 incorrectly refers to footnote (13) and should refer to footnote (14).

Attached please find the revised TS Table of Contents and the corrected page TS 3.1-17. WPSC apologizes for any inconvenience this may have caused.

In accordance with the requirements of 10 CFR 50.30(b), this submittal has been signed and notarized. A complete copy of this submittal has been transmitted to the State of Wisconsin as required by 10 CFR 50.91(b)(1).

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Sincerely,

R A Krueger for

Clark R. Steinhardt
Senior Vice President - Nuclear Power

BJD/jms

Attach.

cc - US NRC - Region III
Mr. Patrick Castleman, US NRC
Mr. R. S. Cullen, PSCW

Subscribed and Sworn to
Before Me This 25th Day
of June 1991

Jeanne M. Stein
Notary Public, State of Wisconsin

My Commission Expires:
June 18, 1995

LIC\NRC\N489

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TECHNICAL SPECIFICATIONS
APPENDIX A

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of RHR suction are isolated or valve RHR-33-1 is inoperable, the system can still be considered operable if an alternate vent path is provided which has the same or greater effective flow cross section as the LTOP safety valve (≥ 6.4 sq. inches).

Maximum Coolant Activity (TS 3.1.c)

This specification is based on the evaluation of the consequences of a postulated rupture of a steam generator tube when the maximum activity in the reactor coolant is at the allowable limit. The potential release of activity to the atmosphere has been evaluated to insure that the public is protected.

Rupture of a steam generator tube would allow reactor coolant activity to enter the secondary system. The major portion of this activity is noble gases[14] which would be released to the atmosphere from the air ejector or a relief valve. Activity could continue to be released until the operator could reduce the Reactor Coolant System pressure below the setpoint of the secondary relief valves and could isolate the faulty steam generator. The worst credible set of circumstances is considered to be a double-ended break of a single tube, followed by isolation of the faulty steam generator by the operator within one-half hour after the event. During this period, 120,000 lbs. of reactor coolant are discharged into the steam generator.[14]

The limiting off-site dose is the whole-body dose resulting from immersion in the cloud containing the released activity. Radiation would include both gamma and beta radiation. The gamma dose is dependent on the finite size and configuration of the cloud. However, for purposes of analysis, the simple model of a semi-infinite cloud, which gives an upper limit to the potential gamma dose, has been used. The semi-infinite cloud model is applicable to the beta dose because of the short range of beta radiation in air. The effectiveness of clothing as shielding against beta radiation is neglected and therefore the analysis model also gives an upper limit to the potential beta dose.

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[14] USAR Section 14.2.4