Section

9009270053 900917 PDR ADOCK 05000305 P PDC

---- - ----

Title

Section	Title	Page TS
3. 3. 3. 3. 3.4 St 3.5 In 3.6 Co	gineered Safety Features and Auxiliary Systems 3.a Accumulators 3.b Safety Injection and Residual Heat Removal Systems 3.c Containment Cooling Systems 3.d Component Cooling System 3.e Service Water System eam and Power Conversion System strumentation System ntainment System	3.3-1 3.3-1 3.3-2 3.3-4 3.3-6 3.3-6 3.4-1 3.5-1 3.6-1
3.7 Au	xiliary Electrical Systems fueling	3.7-1 3.8-1
	leted	5.0-1
3.10 Co 3. 3. 3. 3. 3. 3. 3. 3. 3. 3. 3. 3. 3.	ntrol Rod and Power Distribution Limits 10.a Shutdown Reactivity 10.b Power Distribution Limits 10.c Quadrant Power Tilt Limits 10.d Rod Insertion Limits 10.e Rod Misalignment Limitations 10.f Inoperable Rod Position Indicator Channels 10.g Inoperable Rod Limitations 10.h Rod Drop Time 10.i Rod Position Deviation Monitor 10.j Quadrant Power Tilt Monitor 10.k Inlet Temperature 10.1 Operating Pressure 10.m Coolant Flow Rate	3.10-1 3.10-1 3.10-5 3.10-5 3.10-6 3.10-6 3.10-6 3.10-6 3.10-7 3.10-7 3.10-7 3.10-7 3.10-7 3.10-7 3.10-7a 3.10-7a
	re Surveillance Instrumentation	3.11-1
	ntrol Room Postaccident Recirculation System ock Suppressors (Snubbers)	3.12-1
3.15 Del		3.14-1 3.15-1
3.1 3.1 3.1 3.1 3.1	5.a Deleted 5.b Deleted 5.c Deleted 5.d Deleted 5.e Deleted 5.f Deleted	3.15-1 3.15-1 3.15-2 3.15-2 3.15-3 3.15-3
	veillance Requirements	4.1-1
4.1 Ope 4.2 ASM 4.2	erational Safety Review ME Code Class In-service Inspection and Testing 2.a ASME Code Class 1, 2, and 3 Components and Supports 2.b Steam Generator Tubes 4.2.b.1 Steam Generator Sample Selection and Inspec 4.2.b.2 Steam Generator Tube Sample Selection and	4.1-1 4.2-1 4.2-1 4.2-3
	Inspection	4.2-4

TS ii

Section

•

on	Title	Page TS
	4.2.b.3 Inspection Frequencies	4.2-5
	4.2.b.4 Plugging Limit Criteria 4.2.b.5 Reports	4.2-6
4.3		4.2-7 4.3-1
4.4		4.3-1
•••	4.4.a Integrated Leak Rate Tests (Type A)	4.4-1
	4.4.b Local Leak Rate Tests (Type B and C)	4.4-3
	4.4.c Shield Building Ventilation System	4.4-6
	4.4.d Auxiliary Building Special Ventilation System	4.4-7
	4.4.e Containment Vacuum Breaker System	4.4-7
4.5		
	System Tests	4.5-1
	4.5.a System Tests	4.5-1
	4.5.a.1 Safety Injection System	4.5-1
	4.5.a.2 Containment Vessel Internal Spray System	4.5-2
	4.5.a.3 Containment Fan Coil Units 4.5.b Component Tests	4.5-2
	4.5.b Component Tests 4.5.b.1 Pumps	4.5-2
	4.5.b.2 Valves	4.5-2 4.5-3
4.6		4.5-3 4.6-1
	4.6.a Diesel Generators	4.6-1
	4.6.b Station Batteries	4.6-2
4.7		4.7-1
4.8		4.8-1
4.9		4.9-1
	Deleted	
	Deleted	
	Spent Fuel Pool Sweep System	4.12-1
	Radioactive Materials Sources	4.13-1
4.14	Testing and Surveillance of Shock Suppressors (Snubbers)	4.14-1
4.15	Deleted	4.15-1
	4.15.a Deleted 4.15.b Deleted	4.15-1
	4.15.c Deleted	4.15-1
	4.15.d Deleted	4.15-2 4.15-2
	4.15.e Deleted	4.15-2
	4.15.f Deleted	4.15-3
4.16	Reactor Coolant Vent System Tests	4.16-1
	Control Room Postaccident Recirculation System	4.17-1
	•	

<u>۱</u>

LIST OF TABLES

	•	
	<u>Tables - TS</u>	Title
	3.1-1	WPS (136) Reactor Vessel Toughness Data
	3.1-2	Reactor Coolant System Pressure Isolation Valves
	3.5-1	Engineered Safety Features Initiation Instrument Setting Limits
	3.5-2	Instrument Operation Conditions for Reactor Trip
	3.5-3	Emergency Cooling
	3.5-4	Instrument Operating Conditions for Isolation Functions
	3.5-5	Instrument Operation Conditions for Safeguards Bus Power Supply Functions
	3.5-6	Instrumentation Operating Conditions for Indication
	3.14-1	Deleted
•	3.15-1	Deleted
	3.15-2	Deleted
	4.1-1	Minimum Frequencies for Checks, Calibrations and Test of Instrument Channels
	4.1-2	Minimum Frequencies for Sampling Tests
	4.1-3	Minimum Frequencies for Equipment Tests
	4.2-1	Deleted
	4.2-2	Steam Generator Tube Inspection
	4.10-1	Deleted
	4.10-2	Deleted
	4.11-1	Deleted
	4.11-2	Deleted
	6.4-1	Deleted
		•

Proposed Amendment No. 96 09/17/90

DELETED

TS 3.15-1

· · ·

DELETED

Proposed Amendment No. 96 09/17/90

DELETED

DELETED

Tab

DELETED

Table TS 3.15-1

Table TS 3.15-2

Proposed Amendment No. 96 09/17/90

DELETED

6. Safeguard Bus Undervoltage and Safeguard Bus Second Level Undervoltage relays shall be calibrated at least once per operating cycle (not to exceed 18 months).

b. Station Batteries

- The voltage of each cell shall be measured to the nearest hundredth volt each month. An equalizing charge shall be applied if the lowest cell in the battery falls below 2.13 volts. The temperature and specific gravity of a pilot cell in each battery shall be measured.
- 2. The following additional measurements shall be made every three months: the specific gravity and height of electrolyte in every cell and the temperature of every fifth cell.
- 3. All measurements shall be recorded and compared with previous data to detect signs of deterioration.
- 4. The batteries shall be subjected to a load test during the first refueling and once every five years thereafter. Battery voltage shall be monitored as a function of time to establish that the battery performs as expected during heavy discharge and that all electrical connections are tight.

BASIS TECHNICAL SPECIFICATION 4.6, PERIODIC TESTING OF EMERGENCY POWER SYSTEMS

Each diesel generator can start and be ready to accept full load within 10 seconds, and will sequentially start and supply the power requirements for one complete set of engineered safety features equipment in approximately one minute.(1)

The specified test frequencies provide reasonable assurance that any mechanical or electrical deficiency will be detected and corrected before it can result in failure of one emergency power supply to respond when called upon to function. Its possible failure to respond is, of course, anticipated by providing two diesel generators, each supplying through an independent bus, a complete and adequate set of engineered safety features equipment. Further, both diesel generators are provided as backup to multiple sources of external power, and this multiplicity of sources should be considered with regard to adequacy of test frequency.

TS 4.6-2

DELETED

TS 4.15-1

•

· · ·

TS 4.15-2

Proposed Amendment No. 96 09/17/90

DELETED

· · ·

TS 4.15-3

Proposed Amendment No. 96 09/17/90

DELETED

• •

DELETED

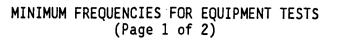
·

Proposed Amendment No. 96 09/17/90

TS 4.15-4

TABLE 4.1-3

5



Equ	ipment Tests***	Test	Frequency	Time Between Test (Days)
1.	Control Rods	Rod drop times of all full length rods	Each refueling outage	. N.A.
		Partial movement of all rods	Every 2 weeks	17
1a.	Reactor Trip Breakers	Independent Test(1) Shunt & Undervoltage Trip Attachments	Monthly	37
1b.	Reactor Coolant Pump Breakers-Open- Reactor Trip	Operability	Each refueling outage	N.A.
1c.	Manual Reactor Trip	Open Trip Reactor(2) Trip & Bypass Bkr	Each refueling outage	N.A.
2.	Deleted			
3.	Deleted			
4.	Containment Isolation Trip	Operability	Each refueling outage	N.A.
5.	Refueling System Interlocks	Operability	Prior to fuel movement each refueling outage	N.A.
6.	Deleted			
7.	Deleted			· ·
8.	RCS Leak Detection	Operability	Weekly	8
9.	Diesel Fuel Supply	*Fuel Inventory	Weekly	8
10.	Turbine Stop and Governor Valves	Operability	Annually	365
11.	Fuel Assemblies	Visual Inspection	Each refueling outage	N.A.
12.	Guard Pipes	Visual Inspection	Each refueling outage	N.A.

Table TS 4.1-3 (Page 1 of 2)

Proposed Amendment No. 96 09/17/90

Maximum

- d. At least one licensed operator shall be in the control room when fuel is in the reactor.
- e. Two licensed operators, one of which shall be an SRO, shall be present in the control room when the unit is in an operational mode other than cold shutdown or refueling.
- f. Refueling operations shall be directed by a licensed Senior Reactor Operator assigned to the refueling operation who has no other concurrent responsibilities during the refueling operation.
- g. Deleted
- h. When the reactor is above the cold shutdown condition, a qualified Shift Technical Advisor shall be within 10 minutes of the control room.

ORGANIZATIONAL CHANGES

- 6.2.3 Changes not affecting safety may be made to the offsite and facility staff organizations. Such changes shall be reported to the Commission in the form of an application for license amendment within 60 days of the implementation of the change.
- 6.3 PLANT STAFF QUALIFICATIONS
 - 6.3.1 Qualification of each member of the Plant Staff shall meet or exceed the minimum acceptable levels of ANSI N18.1-1971 for comparable positions, except for the Superintendent-Plant Radiation Protection who shall meet or exceed the recommendation of Regulatory Guide 1.8, Revision 1-R, September 1975, or their equivalent as further clarified in Attachment 1 to the Safety Evaluation Report enclosed with Amendment No. 46 to Facility Operating License DPR-43.
 - 6.3.2 The Shift Technical Advisor shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in the design of the Kewaunee Plant and plant transient and accident analysis.

TS 6-2

6.4 TRAINING

6.4.1 A retraining and replacement training program for the Plant Staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI-N18.1-1971 and Appendix A of 10 CFR Part 55.

6.5 REVIEW AND AUDIT

6.5.1 PLANT OPERATIONS REVIEW COMMITTEE (PORC)

FUNCTION

6.5.1.1 The PORC shall function to advise the Plant Manager on matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PORC shall be composed of, but not necessarily limited to:

Chairman:

Plant Manager

Required Members:

Assistant Manager-Plant Maintenance Assistant Manager-Plant Operations Assistant Manager-Plant Services Superintendent-Plant Operations Plant Reactor Supervisor Superintendent-Plant Quality Control Superintendent-Plant Technical

ALTERNATES

6.5.1.3 Alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in PORC meetings at any one time.

TS 6-2a

RESPONSIBILITIES

6.5.1.6 The PORC shall be responsible for:

- Review of operating, maintenance and other procedures including emergency operating procedures which affect nuclear safety as determined by the plant manager. Changes to those procedures are made in accordance with the provisions of TS 6.8.1.
- Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Technical Specifications.
- Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Review of all proposed changes to the Security Plan, Emergency Plan, Fire Plan, and their respective implementing procedures.
- f. Review all reports covering the investigation of all violations of the Technical
 Specifications and the recommendations to prevent recurrence.
- g. Review plant operations to detect potential safety hazards.
- h. Performance of special reviews and investigations and prepare reports thereon as requested by the Chairman of the Nuclear Safety Review and Audit Committee.

TS 6-4