DRAFT

ES-401

PWR Examination Outline

Form ES-401-2

Facility: ST.	LUCIE	Date of Exam: FEBRUARY 2011																
			RO K/A Category Points SRO-Only											ly Poir	nts			
lier	Group	К 1	к 2	к 3	К 4	К 5	К 6	A 1	A 2	A 3	A 4	G *	Total		42		∃*	Total
1.	1	3	3	3				3	3			3	18		3		3	6
Abnormal	2	2	1	2		N/A		2	1	N	/A	1	9		2		2	4
Plant Evolutions	Tier Totals	5	4	5				5	4		:	4	27		5		5	10
<u> </u>	1	3	3	3	2	2	2	3	2	2	3	3	28		3		2	5
2. Plant	2	1	1	1	1	1	1	1	1	1	0	1	10	0	2		1	3
Systems	Tier Totals	4	4	4	3	3	3	4	3	3	3	4	38		5		3	8
3. Generic Knowledge and Abiliti Categories			ities			1	1	2		3	4	1	10	1	2	3	4	7
						3		2	3	}	2	2		1	2	2	2	
Note: 1. 2. 3. 4. 5. 6. 7.* 8. 9.	 Categories 1 2 3 4 10 1 2 3 4 10 1 2 3 4 10 1 2 3 4 7 Categories 1 2 3 4 10 1 2 3 4 7 Insert that at least two topics from every applicable K/A category are sampled within each tier of the RO and SRO-only outlines (i.e., except for one category in Tier 3 of the SRO-only outline, the 'Tier Totals'' in each K/A category shall not be less than two). 2. The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and the SRO-only exam must total 25 points. 3. Systems/evolutions within each group are identified on the associated outline, systems or evolutions that do not apply at the facility should be deleted and justified; operationally important, site-specific systems/evolutions that are not included on the outline should be added. Refer to Section D.1.b of ES-401 for guidance regarding the elimination of inappropriate K/A statements. 4. Select topics from as many systems and evolutions as possible; sample every system or evolution in the group before selecting a second topic for any system or evolution. 5. Absent a plant-specific priority, only those K/As having an importance rating (IR) of 2.5 or higher shall be selected. Use the RO and SRO ratings for the RO and SRO-only portions, respectively. 6. Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories. 7.* The generic (G) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A catalog, but the topics mortance rating (IR) for teach opticable K/As. 8. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance rating (IR) for teach opticable K/As. <																	

ES-401, RI	EV 9	T1	G1 PWR EXAMINATION OUTLINE	FORM ES-401-2
KA	NAME / SAFETY FUNCTION:	IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
/		RO SR	80	
V007EA2.06	Reactor Trip - Stabilization - Recovery / 1	4.3 4.5		Occurrence of a reactor trip
008AK2.02	Pressurizer Vapor Space Accident / 3	2.7 2.7		Sensors and detectors
1009EG2.4.4	Small Break LOCA / 3	4.5 4.7		Ability to recognize abnormal indications for system operating parameters which are entry-level conditions for emergency and abnormal operating procedures.
015AK2.07	RCP Malfunctions / 4	2.9 2.9		RCP seals
022AA2.04	Loss of Rx Coolant Makeup / 2	2.9 3.8		How long PZR level can be maintained within limits
025AK3.01	Loss of RHR System / 4	3.1 3.4		Shift to alternate flowpath
1026AA1.02	Loss of Component Cooling Water / 8	3.2 3.3		Loads on the CCWS in the control room
1027AK2.03	Pressurizer Pressure Control System Malfunction / 3	2.6 2.8		Controllers and positioners
¥029EK1.05	ATWS / 1	2.8 3.2		definition of negative temperature coefficient as applied to large PWR coolant systems
038EG2.4.46	Steam Gen. Tube Rupture / 3	4.2 4.2		Ability to verify that the alarms are consistent with the plant conditions.
055EK3.02	Station Blackout / 6	4.3 4.6		Actions contained in EOP for loss of offsite and onsite power

ES-401, RI	EV 9		T1(G1 PWR EXAMINATION OUTLINE	FORM ES-401-2	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRC	0		
056AA1.07	Loss of Off-site Power / 6	3.2	3.2		Service water pump	
J058AA1.02	Loss of DC Power / 6	3.1	3.1		Static inverter dc input breaker, frequency meter, ac output breaker and ground fault detector	
062AA2.04	Loss of Nuclear Svc Water / 4	2.5	2.9		The normal values and upper limits for the temperatures of the components cooled by SWS	
V065AG2.1.23	Loss of Instrument Air / 8	4.3	4.4		Ability to perform specific system and integrated plant procedures during all modes of plant operation.	
077AK1.02	Generator Voltage and Electric Grid Disturbances / 6	3.3	3.4		Over-excitation	
√CE05EK3.3	Steam Line Rupture - Excessive Heat Transfer / 4	3.8	4		Manipulation of controls required to obtain desired operating results during abnormal, and emergency situations.	
CE06EK1.3	Loss of Main Feedwater / 4	3.2	3.7		Annunciators and conditions indicating signals, and remedial actions associated with the (Loss of Feedwater).	

•

.

				C. C	
ES-401, RI	EV 9		T16	32 PWR EXAMINATION OUTLINE	FORM ES-401-2
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRC)	
V001AA2.04	Continuous Rod Withdrawal / 1	4.2	4.3		Reactor power and its trend
033AA1.01	Loss of Intermediate Range NI / 7	2.9	3.1		Power-available indicators in cabinets or equipment drawers
v051AK3.01	Loss of Condenser Vacuum / 4	2.8	3.1		Loss of steam dump capability upon loss of condenser vacuum
059AG2.1.30	Accidental Liquid RadWaste Rel. / 9	4.4	4.0		Ability to locate and operate components, including local controls.
060AK1.04	Accidental Gaseous Radwaste Rel. / 9	2.5	3.7		Calculation of offsite doses due to a release from the power plant
V069AK3.01	Loss of CTMT Integrity / 5	3.8	4.2		Guidance contained in EOP for loss of containment integrity
√074EK1.04	Inad. Core Cooling / 4	3.7	4.1		Use of steam tables, including subcooled, saturated and superheated regions.
VCA11AA1.2	RCS Overcooling - PTS / 4	3.2	3.4		Operating behavior characteristics of the facility.
CA13AK2.2	Natural Circ. / 4	3.4	3.6		Facility's heat removal systems, including primary coolant, emergency coolant, the decay heat removal systems, and relations between the proper operation of these systems to the operation of the facility.

New York				New grade and the second se		
ES-401, F	REV 9		T20	G1 PWR EXAMINATION OUTLINE	FORM FS-401-2	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
1		RO	SRC	D .		
V003A4.02	Reactor Coolant Pump	2.9	2.9		RCP motor parameters	
1						
1004K3.04	Chemical and Volume Control	3.7	3.9		RCPS	
J004K5.30	Chemical and Volume Control	3.8	4.2		Relationship between temperature and pressure in CVCS components during solid plant operation	
005A4.01	Residual Heat Removal	3.6	3.4		Controls and indication for RHR pumps	
V006A1.05	Emergency Core Cooling	2.9	3.3		CCW flow (establish flow to RHR heat exchanger prior to placing in service	
007G2.4.6	Pressurizer Relief/Quench Tank	3.7	4.7		Knowledge symptom based EOP mitigation strategies.	
J008K2.02	Component Cooling Water	3.0	3.2		CCW pump, including emergency backup	
V010K6.01	Pressurizer Pressure Control	2.7	3.1		Pressure detection systems	
101201 01	Departer Protection					
/	neadior Protection	2.9	3.4		Trip setpoint adjustment	
012G2.4.2	Reactor Protection	4.5	4.6		Knowledge of system set points, interlocks and automatic actions associated with EOP entry conditions.	
013A2.01	Engineered Safety Features Actuation	4.6	4.8		LOCA	

ES-401, F	REV 9	T20	31 PWR EXAMINATION OUTLINE	FORM ES-401-2
KA	NAME / SAFETY FUNCTION:	IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO SRC)	
✓ 013A3.02	Engineered Safety Features Actuation	4.1 4.2		Operation of actuated equipment
/ 022K2.01	Containment Cooling	3.0 3.1		Containment cooling fans
/ 022K2.02	Containment Cooling	2.5 2.4		Chillers
026A1.03	Containment Spray	3.5 3.5		Containment sump level
√026K3.02	Containment Spray	4.2 4.3		Recirculation spray system
V039A2.04	Main and Reheat Steam	3.4 3.7		Malfunctioning steam dump
V059K1.05	Main Feedwater	3.1 3.2		RCS
061K5.02	Auxiliary/Emergency Feedwater	3.2 3.6		Decay heat sources and magnitude
1				
•062K4.01	AC Electrical Distribution	2.6 3.2		Bus lockouts
062K4.06	AC Electrical Distribution	2.9 3.3		One-line diagram of 6.9kV distribution, including sources of normal and alternative power
063K3.01	DC Electrical Distribution	3.7 4.1		ED/G

ES-401, R	REV 9		T20	31 PWR EXAMINATION OUTLINE	FORM ES-401-2	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
1		RO	SRC)		
/ 064K1.05	Emergency Diesel Generator	3.4	3.9		Starting air system	
L						
√064K6.08	Emergency Diesel Generator	3.2	3.3		Fuel oil storage tanks	
1						
1073A4.03	Process Radiation Monitoring	3.1	3.2		Check source for operability demonstration	
				- -		
076G2.2.3	Service Water	3.8	3.9		(multi-unit license) Knowledge of the design, procedural and operational differences between units.	
<u> </u>						
078K1.05	Instrument Air	3.4	3.5		MSIV air	
1						
103A3.01	Containment	3.9	4.2		Containment isolation	

ES-401, REV 9			T20	52 PWR EXAMINATION OUTLINE	FORM ES-401-2
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
1		RO	SRC)	
J002K1.03	Reactor Coolant	3.8	3.8		Borated water storage tank
015K2.01	Nuclear Instrumentation	3.3	3.7		NIS channels, components and interconnections
016K5.01	Non-nuclear Instrumentation	2.7	2.8		Separation of control and protection circuits
017G2.4.21	In-core Temperature Monitor	4.0	4.6		Knowledge of the parameters and logic used to assess the status of safety functions
033K3.03	Spent Fuel Pool Cooling	3.0	3.3		Spent fuel temperature
034K6.02	Fuel Handling Equipment	2.6	3.3		Radiation monitoring systems
045A2.17	Main Turbine Generator	2.7	2.9		Malfunction of electrohydraulic control
055A3.03	Condenser Air Removal	2.5	2.7		Automatic diversion of CARS exhaust
071A1.06	Waste Gas Disposal	2.5	2.8		Ventilation system
075K4.01	Circulating Water	2.5	2.8		Heat sink

- 10 [

ES-401, I	REV 9		T3 PWR EXAMINATION OUTLINE	FORM ES-401-2
KA	NAME / SAFETY FUNCTION:	IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
-		RO S	SRO	
#G2.1.17	Conduct of operations	3.9 4.	.0	Ability to make accurate, clear and concise verbal reports.
G2.1.37	Conduct of operations	4.3 4.	.6	Knowledge of procedures, guidelines or limitations associated with reactivity management
162.1.5	Conduct of operations	2.9 3.	.9	Ability to locate and use procedures related to shift staffing, such as minimum crew complement, overtime limitations, etc.
62.2.22	Equipment Control	4.0 4.	7	Knowledge of limiting conditions for operations and safety limits.
162.2.41	Equipment Control	3.5 3.	9	Ability to obtain and interpret station electrical and mechanical drawings
G2.3.11	Radiation Control	3.8 4.3	3	Ability to control radiation releases.
da2.3.12	Radiation Control	3.2 3.1	7	Knowledge of radiological safety principles pertaining to licensed operator duties
G2.3.5	Radiation Control	2.9 2.9	9	Ability to use radiation monitoring systems
G2.4.28	Emergency Procedures/Plans	3.2 4.1		Knowledge of procedures relating to emergency response to sabotage.
G2.4.34	Emergency Procedures/Plans	4.2 4.1	1 □ □ □ □ □ □ □ □ □ □ □ □ □ □	Knowledge of RO tasks performed outside the main control room during an emergency and the resultant operational effects

				(interview of the second seco			
ES-401, RI	EV 9	S	RO T	1G1 PWR EXAMINATION OUTLINE	FORM ES-401-2		
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RO	SRO)			
008AG2.4.11	Pressurizer Vapor Space Accident / 3	4.0	4.2		Knowledge of abnormal condition procedures.		
929EG2.4.18	ATWS / 1	3.3	4.0		Knowledge of the specific bases for EOPs.		
,þ40AA2.05	Steam Line Rupture - Excessive Heat Transfer / 4	4.1	4.5		When ESFAS systems may be secured		
054AG2.4.30	Loss of Main Feedwater / 4	2.7	4.1		Knowledge of events related to system operations/status that must be reported to internal orginizations or outside agencies.		
Ø57AA2.16	Loss of Vital AC Inst. Bus / 6	3	3.1		Normal and abnormal PZR level for various modes of plant operation		
077AA2.03	Generator Voltage and Electric Grid Disturbances / 6	3.5	3.6		Generator current outside the generator capability curve		

			A star a gran a star		
ES-401, RI	EV 9	SRO 1	1G2 PWR EXAMINATION OUTLINE	FORM ES-401-2	
KA	NAME / SAFETY FUNCTION:	IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
1		RO SRC)		
J 024AG2.2.25	Emergency Boration / 1	3.2 4.2		Knowledge of the bases in Technical Specifications for limiting conditions for operations and safety limits.	
037AG2.4.41	Steam Generator Tube Leak / 3	2.9 4.6		Knowledge of the emergency action level thresholds and classifications.	
1069AA2.01	Loss of CTMT Integrity / 5	3.7 4.3		Loss of containment integrity	
CA11AA2.1	RCS Overcooling - PTS / 4	2.9 3.3		Facility conditions and selection of appropriate procedures during abnormal and emergency operations.	

ES-401, REV 9				
			SRO T2G1 PWR EXAMINATION OUTLINE	FORM ES-401-2
KA	NAME / SAFETY FUNCTION:	I	IR K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
/	/	RO	SRO	
003G2.2.40	Reactor Coolant Pump	3.4	4.7	Ability to apply technical specifications for a system.
_1				
008A2.07	Component Cooling Water	2.5	2.8	Consequences of high or low CCW flow rate and tempera- ture; the flow rate at which the CCW standby pump will start
013G2.2.44	Engineered Safety Features Actuation	4.2		
		4.2	4.4 □□□□□□□□□□□□□	Ability to interpret control room indications to verify the status and operation of a system, and understand how operator actions and directives affect plant and system conditions
039A2.03	Main and Reheat Steam	3.4	3.7	Indications and alarms for main steam and area radiation monitors (during SGTR)
1061A2 05	Auxilian/Emergency Ecodyster	0.1		
9 00 // 12.00	Automaty/Entrychoy recuwater	3.1	3.4	Automatic control malfunction

ES-401, REV 9					
		s	RO	T2G2 PWR EXAMINATION OUTLINE	FORM ES-401
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRC	0	
4016A2.02	Non-nuclear Instrumentation	2.9	3.2		Loss of power supply
035G2.4.47	Steam Generator	4.2	4.2		Ability to diagnose and recognize trends in an accurate and timely manner utilizing the appropriate control room reference material.
075A2.02	Circulating Water	2.5	2.7		Loss of circulating water pumps

ES-401, REV 9			SRO	T3 PWR EXAMINATION OUTLINE	FORM FS-401-2	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRC)		
G2.1.40	Conduct of operations	2.8	3.9		Knowledge of refueling administrative requirements	
G2.2.15	Equipment Control	3.9	4.3		Ability to determine the expected plant configuration using design and configuration control documentaion	
G2.2.21	Equipment Control	2.9	4.1		Knowledge of pre- and post-maintenance operability requirements.	
G2.3.13	Radiation Control	3.4	3.8		Knowledge of radiological safety procedures pertaining to licensed operator duties	
G2.3.4	Radiation Control	3.2	3.7		Knowledge of radiation exposure limits under normal and emergency conditions	
G2.4.16	Emergency Procedures/Plans	3.5	4.4		Knowledge of EOP implementation hierarchy and coordination with other support procedures or guidelines.	
G2.4.44	Emergency Procedures/Plans	2.4	4.4		Knowledge of emergency plan protective action recommendations.	

ES-201

Examination Outline Quality Checklist

WRITTEN EXAM

Form ES-201-2

Facility	ST. LUCIE Date of Examination: FEBRUARY 20	>//	e of the contract data as					
Item	Task Description							
1. W	a. Verify that the outline(s) fit(s) the appropriate model, in accordance with ES-401.	M	N	C#				
R I T	 Assess whether the outline was systematically and randomly prepared in accordance with Section D.1 of ES-401 and whether all K/A categories are appropriately sampled. 	m	NA					
Ť	c. Assess whether the outline over-emphasizes any systems, evolutions, or generic topics.							
Ň	d. Assess whether the justifications for deselected or rejected K/A statements are appropriate.							
2. S	a. Using Form ES-301-5, verify that the proposed scenario sets cover the required number of normal evolutions, instrument and component failures, technical specifications, and major transients.							
H U L A T	 b. Assess whether there are enough scenario sets (and spares) to test the projected number and mix of applicants in accordance with the expected crew composition and rotation schedule without compromising exam integrity, and ensure that each applicant can be tested using at least one new or significantly modified scenario, that no scenarios are duplicated from the applicants' audit test(s), and that scenarios will not be repeated on subsequent days 							
R	c. To the extent possible, assess whether the outline(s) conform(s) with the qualitative and quantitative criteria specified on Form ES-301-4 and described in Appendix D.							
3. W / T	 a. Verify that the systems walk-through outline meets the criteria specified on Form ES-301-2: (1) the outline(s) contain(s) the required number of control room and in-plant tasks distributed among the safety functions as specified on the form (2) task repetition from the last two NRC examinations is within the limits specified on the form (3) no tasks are duplicated from the applicants' audit test(s) (4) the number of new or modified tasks meets or exceeds the minimums specified on the form (5) the number of alternate path, low-power, emergency, and RCA tasks meet the criteria on the form. 							
	 b. Verify that the administrative outline meets the criteria specified on Form ES-301-1: (1) the tasks are distributed among the topics as specified on the form (2) at least one task is new or significantly modified (3) no more than one task is repeated from the last two NRC licensing examinations 	1/						
	c. Determine if there are enough different outlines to test the projected number and mix of applicants and ensure that no items are duplicated on subsequent days.	1		-1				
4. G	 Assess whether plant-specific priorities (including PRA and IPE insights) are covered in the appropriate exam sections. 	m	N/A					
E	b. Assess whether the 10 CFR 55.41/43 and 55.45 sampling is appropriate.	m	NA					
E	c. Ensure that K/A importance ratings (except for plant-specific priorities) are at least 2.5.	m	MA					
A	d. Check for duplication and overlap among exam sections.	"/A	NA	NA				
- F	e. Check the entire exam for balance of coverage.	m	NA					
	 Assess whether the exam fits the appropriate job level (RO or SRO). 	11	NA					
a. Autho b. Facilit c. NRC (d. NRC (MICHAEL MEEKS Multiple Meuks N/A Supervisor LIALCOLUT, WIDNIAN /	c c	3/31 04/22 04/22 5(04/	1 2010 [2010 [2010 [2010 (0				
Note: # Independent NRC reviewer initial items in Column "c"; chief examiner concurrence required. * Not applicable for NRC-prepared examination outlines								



Administrative Topics Outline DRAFT

Form ES-301-1

	a magazatatatatatatatatatatata					
Facility: St. Lucie		Date of Examination: 2/21/11				
Examination Level (circle c	Examination Level (circle one): RO/SRO Operating Test Number: NRC					
	1					
Administrative Topic	Type	Describe activity to be performed				
(see note)	Code					
Conduct of Operations	M, R	A1				
Conduct of Operations		Perform a Manual Calorimetric – Unit 2				
	N, R	A2				
Conduct of Operations		Determine time SDC entry conditions are required based on available CST level.				
	N, R	A3				
Equipment Control		Develop Equipment Clearance Order for 2A HPSI Pump				
	M, R	A4				
Radiation Control		(SRO) Determine Exposure Limits Under Emergency Conditions				
		A5				
		(RO) Determine Exposure Limits Under Normal Conditions				
	N, S or R	A6				
Emergency Plan		(SRO) Respond to Security Event				
NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless						
they are retaking only the administrative topics, when all 5 are required.						
* I ype Codes & Criteria:	(C)ontrol ro	(C)ontrol room, (S)imulator, or Class(R)oom				
	(D) Irect from	(D) ITECL FROM DATIK (≤ 3 for HOS; ≤ 4 for SHOS & HO retakes) (N) every or (M) odified from bank (> 1)				
	(P) revious 2 evams (≤ 1 : randomly selected)					

DRAFT

Administrative Topics Outline DRAFT

ADMINISTRATIVE JPM SUMMARY

- A1: Conditions given on Unit 2 at 30% power. Direction given to perform a manual calorimetric. Plant data is given on cue sheet so calculation can be performed in the classroom in a group setting.
- A2: Given CST level and plant conditions determine time SDC entry conditions required.
- A3: Develop ECO to replace defective shaft seals on the 2A High Pressure Safety Injection pump. Identification of applicable Technical Specifications when removing the 2A HPSI from service are also required.
- A4: SRO: A LOCA has occurred with an isolable leak on the Charging pump. Given the dose rate and time to isolate the leak the SRO is to determine if an individual can perform this evolution without exceeding the Emergency Plan guidelines.
- A5: RO: Radiological conditions are given to repair the refueling machine. Four individuals (two FPL and two contract personnel) are assigned to perform the repair. The individuals past exposure is given. The RO is to determine if the individuals can perform the repairs without the Site Vice Presidents approval.
- A6: TIME CRITICAL. Conditions will be given that armed intruders have entered the protected area. Direction is given to implement 0-AOP-72.01, 'Response to Security Events'. This will lead into implementing EPIPS.

Control Room/In-Plant Systems Outline DRAFT

Form ES-301-2

Facility	: St. Lucie		Date of E	xamination:	Feb. 2011			
Exam	Level (circle one):	RO, SRO(I), SRO(U)	Operating	g Test No.:	HLC-20 NRC			
Contro	l Room Systems [@] (8	for RO; 7 for SRO-I; 2 or 3 for S	SRO-U, inclu	ding 1 ESF)				
	System / JPM Title Type Code* Safety Function							
S-1	Align ECCS for Hot and Cold Leg Injection (2008 NRC exam) D, EN, A, S, L, P							
S-2 (All)	Perform Control Room Actions for Control RoomM, L, S, AInaccessibility – Unit 2 (Modified 0821004)							
S-3 (RO only)	Loss of Safety Rela center)	nted AC Bus – Train A (2A5 480	N, S	6				
S-4 (All)	Verify Containment	Spray – Unit 2		N, S, A, EN	5			
S-5 (All)	Establish Alternate HPSI Header – Uni	Charging Flowpath to RCS Threet 2.	ough 'A'	P, D, A, S, L	2			
S-6	Start 2A1 and 2A2	RCP post LOOP		N, S, A	4р			
C-1	Respond to high C(radioactive in-leaka	CW surge tank level, Unit 1 due ge. (0821030)	to	D, C	9			
C-2	Respond to failure of Wide Range Nuclear InstrumentationD, CUnit 1 (0821036)							
In-Plant Systems [@] (3 for RO; 3 for SRO-I; 3 or 2 for SRO-U)								
P-1 (All)	Restore Auxiliary Feedwater Flow following Steam Binding – N, L, E 4 Unit 1							
P-2 (All)	Local Operation of Boron Concentration Control – Unit 2 R, D, E 1 (0821212)							
P-3	Disconnect 1B Insti maintenance – Unit	rument Inverter from service for 1 (0821067)	preventive	D	6			

Control Room/In-Plant Systems Outline DRAFT

Form ES-301-2

@ All control room (and in-plant) systems must SRO-U systems must serve different safety to overlap those tested in the control room.	be different and serve different safety functions; all 5 unctions; in-plant systems and functions may
* Type Codes	Criteria for RO / SRO-I / SRO-U
(A)Iternate path	4-6 / 4-6 / 2-3
(C)ontrol room	
(D)irect from bank	\leq 9 / \leq 8 / \leq 4
(E)mergency or abnormal in-plant	≥1/≥1/≥1
(EN)gineered safety feature	- / - / > 1 (control room system)
(L)ow-Power / Shutdown	$\geq 1/\geq 1/\geq 1$
(N)ew or (M)odified from bank including 1(A)	≥2/≥2/≥1
(P)revious 2 exams	\leq 3 / \leq 3 / \leq 2 (randomly selected)
(R)CA	$\geq 1/\geq 1/\geq 1$
(S)imulator	

JPM SUMMARY DESCRIPTION

S1- Align ECCS for Hot and Cold Leg Injection

The applicant is required to align the 'B' train for Hot and Cold leg injection. V3523 will not open and the 'A' train will be required to be lined up for Hot and Cold Leg injection.

S2- Perform Control Room Actions for Control Room Inaccessibility – Unit 2

Perform Operator actions in the Control Room prior to Control Room evacuation due to a fire. Direction is given there is not enough time to perform SPTA's. JPM is alternate path in that one RCP cannot be stopped from its control switch. Stopping the RCP requires the 6.9KV bus to be deenergized.

S3- Loss of Safety Related AC Bus - Train 'A' (2A5 480V Load center)

A loss of the 2A5 480V LC occurs. The applicant is directed to perform General Actions for loss of the bus. After the General actions are performed the applicant is to re-energize the bus by closing the applicable breakers. The JPM will be terminated when the bus is re-energized.

S4- Verify Containment Spray – Unit 2

Conditions will be given that require verification of Containment Spray. Numerous failures associated with Containment Spray must be identified and corrected.

S5- Establish Alternate Charging Flowpath to RCS Through 'A' HPSI Header – Unit 2.

The Unit is in 2-EOP-15, Functional Recovery. A pipe break in the Charging header has resulted in the normal Charging flow unavailable. The applicant will use Appendix T from 2-EOP-99, Appendices, Tables and Figures, to establish a Charging flowpath using the 'A' HPSI header. This JPM is faulted in that the 2A Charging pump trips 5 seconds after starting. Applicant should refer back to 5.C and start the 2C Charging pump and continue with the lineup.

S6- Start 2A1 and 2A2 RCP post LOOP

After a LOOP, when offsite power is regained, direction to start 2A1 and 2A2 RCP's will be given. When the 2A2 RCP is started it will develop a severe oil leak which will require the pump to be tripped.

C1- Respond to CCW excessive activity, Unit 1

A CCW surge tank level high / compartment level low alarm is received. Direction is given to perform actions required by 1-AOP-14.01, "Component Cooling Water Abnormal Operations" and 1-AOP 14.02, "Component Cooling Water Excessive Activity". Procedure will direct the applicant to a leaking Sample Heat Exchanger that will be isolated.

C2- Respond to Linear Range NI channel malfunction, Unit 1

Numerous annunciators will be given in the initial conditions. The applicant will be required to identify the failure associated with the annunciators and carry out the appropriate actions using 1-AOP-99.01, Loss of Technical Specification Instrumentation.

P1- Restore Auxiliary Feedwater Flow following Steam Binding – Unit 1

The 1B Auxiliary Feedwater Pump had indications of steam binding. Direction is given to restore AFW flow following steam binding.

P2- Local Operation of Boron Concentration Control – Unit 2

Due to instrumentation problem in the Control Room blending of the VCT will be required locally. Off normal procedure 2-AOP-02.01 Boron Concentration Control System Abnormal Operations will be implemented to increase VCT level.

P3- Disconnect 1B Instrument Inverter from service for preventive maintenance – Unit 1

The 1B Instrument Inverter is to be removed from service for maintenance. The Instrument bus will be placed on the Maintenance Bypass Bus IAW 1-NOP-49.05B, 120VAC Instrument Bus 1MB (Class IE) Normal Operation.