



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 26, 2011

Mr. Michael J. Pacilio
President and Chief Nuclear Officer
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: CLINTON POWER STATION, UNIT NO. 1 – LICENSE AMENDMENT NO. 193
FOR VARIOUS ADMINISTRATIVE TECHNICAL SPECIFICATIONS CHANGES
(TAC NO. ME 4030)

Dear Mr. Pacilio:

The U.S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment No. 193 to Facility Operating License No. NPF-62 for the Clinton Power Station (CPS) Unit No. 1. The amendment is in response to your application dated June 4, 2010.

The amendment removes an expired time-related item and several typographical errors from the CPS Technical Specifications.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Nicholas J. DiFrancesco".

Nicholas J. DiFrancesco, Project Manager
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-461

Enclosures:

1. Amendment No. 193 to NPF-62
2. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-461

CLINTON POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 193
License No. NPF-62

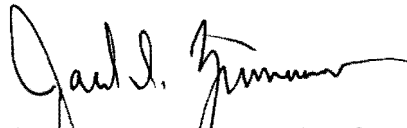
1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (the licensee), dated June 4, 2010 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-62 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No193 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Jacob I. Zimmerman, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance: May 26, 2011

ATTACHMENT TO LICENSE AMENDMENT NO. 193

FACILITY OPERATING LICENSE NO. NPF-62

DOCKET NO. 50-461

Replace the following pages of the Facility Operating License and Appendix "A" Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

Insert

License NPF-62

License NPF-62

Page 3

Page 3

TSs

TSs

3.0-5

3.0-5

3.3-34

3.3-34

3.6-54b

3.6-54b

3.6-69

3.6-69

3.8-24b

3.8-24b

- (4) Exelon Generation Company, pursuant to the Act and to 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (5) Exelon Generation Company, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
- (6) Exelon Generation Company, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility. Mechanical disassembly of the GE141 isotope test assemblies containing Cobalt-60 is not considered separation; and
- (7) Exelon Generation Company, pursuant to the Act and 10 CFR Part 30, to intentionally produce, possess, receive, transfer, and use Cobalt-60.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Exelon Generation Company is authorized to operate the facility at reactor core power levels not in excess of 3473 megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 193, are hereby incorporated into this license. Exelon Generation Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3.0 SR APPLICABILITY (continued)

SR 3.0.3 (continued) When the Surveillance is performed within the delay period and the Surveillance is not met, the LCO must immediately be declared not met, and the applicable Condition(s) must be entered.

SR 3.0.4 Entry into a MODE or other specified condition in the Applicability of an LCO shall only be made when the LCO's Surveillances have been met within their specified Frequency, except as provided by SR 3.0.3. When an LCO is not met due to Surveillances not having been met, entry into a MODE or other specified condition in the Applicability shall only be made in accordance with LCO 3.0.4. This provision shall not prevent entry into MODES or other specified conditions in the Applicability that are required to comply with ACTIONS or that are part of a shutdown of the unit.

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>D. As required by Required Action A.1 and referenced in Table 3.3.5.1-1.</p>	<p>D.1 -----NOTE----- Only applicable if HPCS pump suction is not aligned to the suppression pool. -----</p> <p>Declare HPCS System inoperable.</p> <p><u>AND</u></p> <p>D.2.1 Place channel in trip.</p> <p><u>OR</u></p> <p>D.2.2 Align the HPCS pump suction to the suppression pool.</p>	<p>1 hour from discovery of loss of HPCS initiation capability</p> <p>24 hour</p> <p>24 hours</p>

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
<p>SR 3.6.5.1.3 Verify bypass leakage is less than or equal to the bypass leakage limit. However, during the first unit startup following bypass leakage testing performed in accordance with this SR, the acceptance criterion is $\leq 10\%$ of the drywell bypass leakage limit.</p>	<p>24 months following 2 consecutive tests with bypass leakage greater than the bypass leakage limit until 2 consecutive tests are less than or equal to the bypass leakage limit</p> <p><u>AND</u></p> <p>48 months following a test with bypass leakage greater than the bypass leakage limit</p> <p><u>AND</u></p> <p>-----NOTE----- SR 3.0.2 is not applicable for extensions > 12 months. -----</p> <p>In accordance with the Surveillance Frequency Control Program</p>

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Two or more drywell post-LOCA vacuum relief subsystems inoperable for reasons other than Condition A.	C.1 Restore drywell post-LOCA vacuum relief subsystems to OPERABLE status.	72 hours
D. Required Action and associated Completion Time of Condition A not met.	D.1 Be in MODE 3.	12 hours
	<u>AND</u> D.2 Be in MODE 4.	36 hours
E. Required Action and associated Completion Time of Condition B or C not met.	-----NOTE----- LCO 3.0.4.a is not applicable when entering MODE 3. -----	
	E.1 Be in MODE 3.	12 hours

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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 193 TO FACILITY OPERATING LICENSE NO. NPF-62

EXELON GENERATION COMPANY, LLC

CLINTON POWER STATION, UNIT NO. 1

DOCKET NO. 50-461

1.0 INTRODUCTION

By letter to the U.S. Nuclear Regulatory Commission (NRC, the Commission) dated June 4, 2010, (Agencywide Document Access and Management System Accession (ADAMS) No. ML101550500) Exelon Generation Company, LLC (EGC, the licensee) requested an amendment to Technical Specifications (TSs) for the Clinton Power Station (CPS), Unit No. 1. The proposed amendment would remove an expired time-related item and several typographical errors from the TSs.

2.0 REGULATORY EVALUATION

Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include TSs as part of the license. The NRC regulatory requirements related to the content of the TSs are contained in Title 10, *Code of Federal Regulations* (10 CFR), Part 50, Section 50.36, "Technical Specifications." The TS requirements in 10 CFR 50.36 include the following categories: (1) safety limits, limiting safety systems settings and control settings, (2) limiting conditions for operation, (3) surveillance requirements (SRs), (4) design features, (5) administrative controls, (6) decommissioning, (7) initial notification, and (8) written reports.

3.0 TECHNICAL EVALUATION

3.1 Proposed change to TS 3.0, "SR Applicability"

The licensee is proposing to remove the single line and parenthetical "(continued)" at the bottom of TS page 3.0-5, and insert a double line to indicate the end of the section. This error was introduced into the TS during the implementation of Amendment 163 (Reference 1). The NRC staff has determined that the proposed change corrects an editorial issue related to the format of TS page 3.0-5 and is administrative.

3.2 Proposed change to TS 3.3.5.1, "ECCS Instrumentation"

The licensee is proposing to correct an administrative issue in TS 3.3.5.1, "ECCS Instrumentation" by changing the position of the Logical Connector "OR" between TS 3.3.5.1, Required Actions D.2.1 and D.2.2. The logical "OR" connector will be placed in alignment with

the Required Action wording. The NRC staff has concluded that this change will provide clarity, will not change the current TS requirement, and is consistent with NUREG 1434, "Standard Technical Specifications General Electric Plants, BWR [Boiling Water Reactor]/6" (Reference 2). This error was introduced into the CPS TS in Amendment 95 (Reference 3), which converted the CPS TS to the Improved Technical Specifications format.

3.3 Proposed Change to TS 3.6.5.1, "Drywell"

The licensee is proposing to delete Note 1 in Surveillance Requirement (SR) 3.6.5.1.3. SR 3.6.5.1.3 Note 1 states, "The next required performance of this SR may be delayed to November 23, 2008." EGC is requesting deletion of Note 1 from SR 3.6.5.1.3, as the allowance provided by this Note which was added in TS Amendment 160 (Reference 4) has expired. The NRC staff has concluded that this SR is being met and no longer needs to be modified by this Note.

3.4 Proposed change to TS 3.6.5.6, "Drywell Post-LOCA Vacuum Relief System"

The licensee is requesting the addition of the words "not met" at the end of Condition D. These words were erroneously omitted from the markups of the TS pages that were submitted in the request that led to Amendment 187 (Reference 5). The NRC staff has concluded that this change corrects an editorial error and is administrative.

3.5 Proposed change to TS 3.8.4, "DC Sources-Operating"

The licensee is requesting the insertion of a new TS Page 3.8.24b. The new TS page 3.8.24b is intentionally left blank. Since the CPS TS are double-sided, an additional blank page should have accompanied Page 3.8-24a during the implementation of Amendment 187. The NRC staff has determined that the purpose of the blank page is to maintain the pagination on the pages subsequent to the newly inserted TS page.

3.6 Summary of Changes

The proposed changes in 3.1 through 3.5 are nontechnical, administrative changes that are editorial in nature or involve the reorganization or reformatting of requirements without affecting technical content or operational requirements. The NRC staff finds that the proposed changes are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments

involve no significant hazards consideration, and there has been no public comment on such finding (September 7, 2010, 75 FR 54395;). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

5.0 REFERENCES

1. License Amendment 163, "Clinton Power Station, Unit 1 - Issuance of Amendment Re: Increased Flexibility in Mode Restraints (TAC NOS. MC3024)," ADAMS Accession No. ML050610297
2. NUREG-1434, Revision 3.0, "Standard Technical Specifications General Electric Plants, BWR/6," ADAMS Accession No. ML041910204
3. License Amendment 95, "Notice of issuance of Amend 95 to License NPF-62 re changes made to TS to replace existing TS w/TS based on NUREG-1434, "Improved BWR-6 TS" dated Sept 1992," ADAMS Legacy Accession No. 9412150114
4. License Amendment 160, "Clinton Power Station, Unit 1 – Issuance of Amendment (TAC NO. MB 7675)," ADAMS Accession No. ML033360470
5. License Amendment 187, "Clinton Power Station, Unit 1 – Issuance of Amendment Re: Technical Specification Change TSTF-423, Risk Informed Modification to Selected Required Action End States for BWR Plants, Using the Consolidate Line Item Improvement Process (TAC NO. MD5859)," ADAMS Accession No. ML090710332

Principal Contributor: Kristi Bucholtz

Date of issuance: May 26, 2011

May 26, 2011

Mr. Michael J. Pacilio
President and Chief Nuclear Officer
Exelon Generation Company, LLC
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Sincerely,

/RA/

Nicholas J. DiFrancesco, Project Manager
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Amendment: ML111330556

* SE Memo Date

NRR-058

OFFICE	LPL3-2/PM ATBC	LPL3-2/PM	LPL3-2/LA	DIRS/ITSB*	OGC NLO	LPL3-2/BC
NAME	CNorton ABillochcolon	NDiFrancesco	SRohrer	RElliot	RH	JZimmerman
DATE	5/16/11	5/26/11	5/17/11	11/18/10	5/25/11	5/26/11

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