General Emergency | Site Area Emergency | Alert | Unusual Event

R — Abnorm. Rad Release / Rad Effluent

1 — Offsite Rad Conditions

General Emergency

RG1.1 — Modes: 1 2 3 4 5 D

ANY monitor reading > Table R-1 "GE" column for ≥ 15 min. (Note 1)
- Do **not** delay declaration awaiting dose assessment results
- If dose assessment results are available, declaration should be based on dose assessment instead of radiation monitor values (see EAL RG1.2)

RG1.2 — Modes: 1 2 3 4 5 D

Dose assessment using actual meteorology indicates doses > 1,000 mRem TEDE or 5,000 mRem thyroid CDE at or beyond the SITE BOUNDARY

RG1.3 — Modes: 1 2 3 4 5 D

Field survey results indicate closed window dose rates > 1,000 mRem/hr expected to continue for ≥ 60 min. at or beyond the SITE BOUNDARY (Note 1)
OR
Analyses of field survey samples indicate thyroid CDE > 5,000 mRem for 1 hr of inhalation at or beyond the SITE BOUNDARY (Note 1)

Site Area Emergency

RS1.1 — Modes: 1 2 3 4 5 D

ANY monitor reading > Table R-1 "SAE" column for ≥ 15 min. (Note 1)
- Do **not** delay declaration awaiting dose assessment results
- If dose assessment results are available, declaration should be based on dose assessment instead of radiation monitor values (see EAL RS1.2)

RS1.2 — Modes: 1 2 3 4 5 D

Dose assessment using actual meteorology indicates doses > 100 mRem TEDE or 500 mRem thyroid CDE at or beyond the SITE BOUNDARY

RS1.3 — Modes: 1 2 3 4 5 D

Field survey results indicate closed window dose rates > 100 mRem/hr expected to continue for ≥ 60 min. at or beyond the SITE BOUNDARY (Note 1)
OR
Analyses of field survey samples indicate thyroid CDE > 500 mRem for 1 hr of inhalation at or beyond the SITE BOUNDARY (Note 1)

Alert

RA1.1 — Modes: 1 2 3 4 5 D

ANY gaseous monitor reading > Table R-1 "Alert" column for ≥ 15 min. (Note 2)

RA1.2 — Modes: 1 2 3 4 5 D

ANY liquid monitor > Table R-1 "Alert" column for ≥ 15 min. (Note 2)

RA1.3 — Modes: 1 2 3 4 5 D

Confirmed sample analyses for gaseous or liquid releases indicate concentrations or release rates > 200 x ODCM limits for ≥ 15 min. (Note 2)

None

None

Unusual Event

RU1.1 — Modes: 1 2 3 4 5 D

ANY gaseous monitors > Table R-1 "UE" column for ≥ 60 min. (Note 2)

RU1.2 — Modes: 1 2 3 4 5 D

ANY liquid monitor reading > Table R-1 "UE" column for ≥ 60 min. (Note 2)

RU1.3 — Modes: 1 2 3 4 5 D

Confirmed sample analyses for gaseous or liquid releases indicate concentrations or release rates > 2 x ODCM limits for ≥ 60 min. (Note 2)

None

None

2 — Onsite Rad Conditions & Spent Fuel Events

Table R-1 Effluent Monitor Classification Thresholds

	Monitor	GE	SAE	ALERT	UE
GASEOUS	Radwaste/RB Vent Effluent	5.5E+7 μCi/s	5.5E+6 μCi/s	200 x Alarm	2 x Alarm
GASEOUS	Main Stack Effluent	1.0E+10 μCi/s	1.0E+9 μCi/s	200 x Alarm	2 x Alarm
LIQUID	Service Water Effluent	N/A	N/A	200 x DRMS High (red)	2 x DRMS High (red)
LIQUID	Liquid RadWaste Effluent	N/A	N/A	N/A	2 x DRMS High (red)
LIQUID	Cooling Tower Blowdown	N/A	N/A	200 x DRMS High (red)	2 x DRMS High (red)

Alert

RA2.1 — Modes: 1 2 3 4 5 D

Alarm on **ANY** of the following radiation monitors due to damage to irradiated fuel or loss of water level:
- 2RMS-RE111
- 2RMS-RE112
- 2RMS-RE113
- 2RMS-RE114
- 2RMS-RE140
- 2HVR*RE14A
- 2HVR*RE14B

RA2.2 — Modes: 1 2 3 4 5 D

A water level drop in a reactor refueling pathway that will result in irradiated fuel becoming uncovered

Unusual Event

RU2.1 — Modes: 1 2 3 4 5 D

UNPLANNED water level drop in a reactor refueling pathway as indicated by inability to restore and maintain SFP level > low water level alarm (Note 3)
AND
Area radiation monitor reading rise on **ANY** of the following:
- 2RMS-RE111
- 2RMS-RE112
- 2RMS-RE113
- 2RMS-RE114
- 2RMS-RE140

RU2.2 — Modes: 1 2 3 4 5 D

UNPLANNED area radiation readings rise by a factor of 1,000 over NORMAL LEVELS

3 — CR/CAS Rad

General Emergency: None

Site Area Emergency: None

Alert

RA3.1 — Modes: 1 2 3 4 5 D

Dose rates > 15 mRem/hr in **EITHER** of the following areas requiring continuous occupancy to maintain plant safety functions:
Control Room
OR
CAS

Unusual Event: None

H — Hazards & Other Conditions Affecting Plant Safety

1 — Natural or Destructive Phenomena

Table H-1 Safe Shutdown Areas
- Reactor Building (including Primary Containment)
- Control Room
- Diesel Generator Engine and Board Rooms
- Standby Switchgear and Battery Rooms
- HPCS Switchgear and Battery Rooms
- Remote Shutdown Rooms
- Control Building HVAC Rooms
- Service Water Pump Rooms
- Electrical Protection Assembly Room
- PGCC Relay Room

Notes
1. The ED should **not** wait until the applicable time has elapsed, but should declare the event as soon as it is determined that the condition will likely exceed the applicable time
2. The ED should **not** wait until the applicable time has elapsed, but should declare the event as soon as it is determined that the release duration has exceeded, or will likely exceed, the applicable time. In the absence of data to the contrary, assume that the release duration has exceeded the applicable time if an ongoing release is detected and the release start time is unknown.
3. If loss of water level in the refueling pathway occurs while in Mode 4, 5 or D, consider classification under EALs CU3.1, CU3.2 or CU3.3
4. The ED should **not** wait until the applicable time has elapsed, but should declare the event as soon as it is determined that the condition has exceeded, or will likely exceed, the applicable time.
5. If the equipment in the stated area was already inoperable, or out of service, before the event occurred, then EAL HA3.1 should **not** be declared as it will have **no** adverse impact on the ability of the plant to safely operate or safely shutdown beyond that already allowed by Technical Specifications at the time of the event.

Alert

HA1.1 — Modes: 1 2 3 4 5 D

Seismic event > OBE (0.075g) as indicated by **EITHER**:
Computer Point ERSNC02, OBE Detected
OR
ANY amber LED light lit at the Seismic Monitor Panel, Response Spectrum Annunciator
AND
Earthquake confirmed by **ANY** of the following:
- Earthquake felt in plant
- JAFNPP seismic instrumentation
- Control Room indication of degraded performance of systems required for the safe shutdown of the plant

HA1.2 — Modes: 1 2 3 4 5 D

Tornado striking
OR
Sustained high winds > 90 mph
resulting in **EITHER**:
VISIBLE DAMAGE to **ANY** SAFETY-RELATED STRUCTURE, SYSTEM, OR COMPONENT within **ANY** Table H-1 area
OR
Control Room indication of degraded performance of **ANY** SAFETY-RELATED STRUCTURE, SYSTEM, OR COMPONENT within **ANY** Table H-1 area

HA1.3 — Modes: 1 2 3 4 5 D

Internal flooding resulting in **EITHER**:
An electrical shock hazard that precludes access to operate or monitor **ANY** SAFETY-RELATED STRUCTURE, SYSTEM or COMPONENT within **ANY** Table H-1 area
OR
Control Room indication of degraded performance of **ANY** SAFETY-RELATED STRUCTURE, SYSTEM or COMPONENT within **ANY** Table H-1 area

HA1.4 — Modes: 1 2 3 4 5 D

Turbine failure-generated PROJECTILEs resulting in **EITHER**:
VISIBLE DAMAGE to or penetration of **ANY** SAFETY-RELATED STRUCTURE, SYSTEM or COMPONENT within **ANY** Table H-1 area
OR
Control Room indication of degraded performance of **ANY** SAFETY-RELATED STRUCTURE, SYSTEM or COMPONENT within **ANY** Table H-1 area

HA1.5 — Modes: 1 2 3 4 5 D

Lake water level > 254 ft
OR
Intake water level < 233 ft

HA1.6 — Modes: 1 2 3 4 5 D

Vehicle crash resulting in **EITHER**:
VISIBLE DAMAGE to **ANY** SAFETY-RELATED STRUCTURE, SYSTEM or COMPONENT within **ANY** Table H-1 area
OR
Control Room indication of degraded performance of **ANY** SAFETY-RELATED STRUCTURE, SYSTEM or COMPONENT within **ANY** Table H-1 area

Unusual Event

HU1.1 — Modes: 1 2 3 4 5 D

Seismic event identified by **ANY two** of the following:
- Annunciator 842121 SEISMIC ACCELERATION EXCEEDED indicates seismic event detected
- Confirmation of earthquake received on NMP-1 or JAFNPP seismic instrumentation
- Earthquake felt in plant

HU1.2 — Modes: 1 2 3 4 5 D

Tornado striking within PROTECTED AREA boundary
OR
Sustained high winds > 90 mph

HU1.3 — Modes: 1 2 3 4 5 D

Internal flooding that has the potential to affect **ANY** SAFETY-RELATED STRUCTURE, SYSTEM, OR COMPONENT required by Technical Specifications for the current operating mode in **ANY** Table H-1 area

HU1.4 — Modes: 1 2 3 4 5 D

Turbine failure resulting in **ANY** of the following:
- Casing penetration
- Damage to turbine seals
- Damage to generator seals

HU1.5 — Modes: 1 2 3 4 5 D

Lake water level > 248.2 ft
OR
Intake water level < 237 ft

2 — Fire or Explosion

General Emergency: None

Site Area Emergency: None

Alert

HA2.1 — Modes: 1 2 3 4 5 D

FIRE or EXPLOSION resulting in **EITHER**:
VISIBLE DAMAGE to **ANY** SAFETY-RELATED STRUCTURE, SYSTEM or COMPONENT within **ANY** Table H-1 area
OR
Control Room indication of degraded performance of **ANY** SAFETY-RELATED STRUCTURE, SYSTEM or COMPONENT within **ANY** Table H-1 area

Unusual Event

HU2.1 — Modes: 1 2 3 4 5 D

FIRE **not** extinguished within 15 min. of Control Room notification or verification of a Control Room FIRE alarm in **ANY** Table H-1 area or Turbine Building (Note 4)

HU2.2 — Modes: 1 2 3 4 5 D

EXPLOSION of sufficient force to damage permanent structures or equipment within the PROTECTED AREA

3 — Hazardous Gas

General Emergency: None

Site Area Emergency: None

Alert

HA3.1 — Modes: 1 2 3 4 5 D

Access to **ANY** Table H-1 area is prohibited due to toxic, corrosive, asphyxiant or flammable gases which jeopardize operation of **ANY** SAFETY-RELATED STRUCTURE, SYSTEM or COMPONENT (Note 5)

Unusual Event

HU3.1 — Modes: 1 2 3 4 5 D

Toxic, corrosive, asphyxiant or flammable gases in amounts that have or could adversely affect NORMAL PLANT OPERATIONS

HU3.2 — Modes: 1 2 3 4 5 D

Recommendation by local, county or state officials to evacuate or shelter site personnel based on an offsite event

4 — Security

General Emergency

HG4.1 — Modes: 1 2 3 4 5 D

A HOSTILE ACTION has occurred such that plant personnel are unable to operate equipment required to maintain safety functions

HG4.2 — Modes: 1 2 3 4 5 D

A HOSTILE ACTION has caused failure of Spent Fuel Cooling systems
AND
IMMINENT fuel damage is likely

Site Area Emergency

HS4.1 — Modes: 1 2 3 4 5 D

A HOSTILE ACTION is occurring or has occurred within the PROTECTED AREA as reported by the Security Site Supervisor

Alert

HA4.1 — Modes: 1 2 3 4 5 D

A HOSTILE ACTION is occurring or has occurred within the Owner Controlled Area as reported by the Security Site Supervisor
OR
A validated notification from NRC of an AIRLINER attack threat within 30 min. of the site

Unusual Event

HU4.1 — Modes: 1 2 3 4 5 D

A SECURITY CONDITION that does **not** involve a HOSTILE ACTION as reported by the Security Site Supervisor
OR
A credible site-specific security threat notification
OR
A validated notification from NRC providing information of an aircraft threat

5 — Control Room Evacuation

General Emergency: None

Site Area Emergency

HS5.1 — Modes: 1 2 3 4 5 D

Control Room evacuation has been initiated
AND
Control of the plant **cannot** be established within 15 min. (Note 4)

Alert

HA5.1 — Modes: 1 2 3 4 5 D

Control Room evacuation has been initiated

Unusual Event: None

6 — Judgment

General Emergency

HG6.1 — Modes: 1 2 3 4 5 D

Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve actual or IMMINENT substantial core degradation or melting with potential for loss of containment integrity or HOSTILE ACTION that results in an actual loss of physical control of the facility. Releases can be reasonably expected to exceed EPA Protective Action Guideline exposure levels (1,000 mRem TEDE or 5,000 mRem thyroid CDE) offsite for more than the immediate site area

Site Area Emergency

HS6.1 — Modes: 1 2 3 4 5 D

Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve actual or likely major failures of plant functions needed for protection of the public or HOSTILE ACTION that results in intentional damage or malicious acts; (1) toward site personnel or equipment that could lead to the likely failure of or; (2) that prevent effective access to equipment needed for the protection of the public. **ANY** releases are **not** expected to result in exposure levels which exceed EPA Protective Action Guideline exposure levels (1,000 mRem TEDE or 5,000 mRem thyroid CDE) beyond the SITE BOUNDARY

Alert

HA6.1 — Modes: 1 2 3 4 5 D

Other co[nditions exist which in the judgment of the] Emergency Director indicate that events are in progress or have occurred which involve an actual or potential substantial degradation of the level of safety of the plant or a security event that involves probable life threatening risk to site personnel or damage to site equipment because of HOSTILE ACTION. **ANY** releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels (1,000 mRem TEDE or 5,000 mRem thyroid CDE)

Unusual Event

HU6.1 — Modes: 1 2 3 4 5 D

Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which indicate a potential degradation of the level of safety of the plant or indicate a security threat to facility protection has been initiated. **No** releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs

E — ISFSI

General Emergency: None

Site Area Emergency: None

Alert: None

Unusual Event

EU1.1 — Modes: 1 2 3 4 5 D

Damage to a loaded cask CONFINEMENT BOUNDARY as indicated by measured dose rates > then **ANY** of the following:
- 400 mRem/hr at 3 feet from the HSM surface
- 100 mRem/hr outside HSM door on centerline
- 20 mRem/hr end shield wall exterior

Modes: 1 Power Operation | 2 Startup | 3 Hot Shutdown | 4 Cold Shutdown | 5 Refuel | D Defueled

Constellation Energy — Nine Mile Point Nuclear Station Unit 2

EPIP-EP-002
Attachment 1
EAL Matrix Unit 2
Page 1 of 2

Approved: ____________ Director, Emergency Planning ____________ Date

General Emergency | Site Area Emergency | Alert | Unusual Event

S — System Malfunct.

1 — Loss of AC Power

SG1.1 — Modes: 1 2 3

Loss of **all** offsite and **all** onsite AC power, Table S-1, to 4.16 KV emergency buses 2ENS*SWG101 and 2ENS*SWG103
AND EITHER:
Restoration of 4.16 KV emergency bus 2ENS*SWG101 or 2ENS*SWG103 within 4 hours is **not** likely
OR
RPV water level **cannot** be restored and maintained above -14 in. or RPV water level **cannot** be determined

SS1.1 — Modes: 1 2 3

Loss of **all** offsite and **all** onsite AC power, Table S-1, to 4.16 KV emergency buses 2ENS*SWG101 and 2ENS*SWG103 for ≥ 15 min. (Note 4)

SA1.1 — Modes: 1 2 3

AC power capability to 4.16 KV emergency buses 2ENS*SWG101 and 2ENS*SWG103 reduced to a single power source, Table S-1, for ≥ 15 min. (Note 4)
AND
ANY additional single power source failure will result in a loss of **all** power to 4.16 KV emergency buses 2ENS*SWG101 and 2ENS*SWG103

SU1.1 — Modes: 1 2 3

Loss of **all** offsite AC power, Table S-1, to 4.16 KV emergency buses 2ENS*SWG101 and 2ENS*SWG103

Table S-1 AC Power Sources

	Source
Onsite	2EGS*EG1
Onsite	2EGS*EG3
Onsite	2EGS*EG2 (with 2ENS*SWG102 crosstied to 2ENS*SWG101 or 2ENS*SWG103)
Offsite	Reserve Transformer A
Offsite	Reserve Transformer B
Offsite	Aux Boiler Transformer

2 — Loss of DC Power

General Emergency: None

SS2.1 — Modes: 1 2 3

< 105 VDC on **both** 2BYS*SWG002A and 2BYS*SWG002B for ≥ 15 min. (Note 4)

Alert: None

3 — Criticality & RPS Failure

SG3.1 — Modes: 1 2

An automatic scram fails to shut down the reactor as indicated by reactor power > 4%
AND
All manual actions fail to shut down the reactor as indicated by reactor power > 4%
AND EITHER of the following exist or have occurred:
RPV water level **cannot** be restored and maintained above -39 in. or RPV water level **cannot** be determined
OR
Suppression pool temperature and RPV pressure **cannot** be maintained below the Heat Capacity Temperature Limit (N2-EOP-PC Figure M)

SS3.1 — Modes: 1 2

An automatic scram failed to shut down the reactor as indicated by reactor power > 4%
AND
Manual actions taken at the reactor control console (mode switch in shutdown, manual scram push buttons and ARI) failed to shut down the reactor as indicated by reactor power > 4%

SA3.1 — Modes: 1 2

An automatic scram failed to shut down the reactor
AND
Manual actions taken at the reactor control console (mode switch in shutdown, manual scram push buttons or ARI) successfully shut down the reactor as indicated by reactor power ≤ 4%

SU3.1 — Modes: 3

An UNPLANNED sustained positive period observed on nuclear instrumentation

4 — Inability to Reach or Maintain Shutdown Conditions

General Emergency: None

Site Area Emergency: None

Alert: None

SU4.1 — Modes: 1 2 3

Plant is **not** brought to required operating mode within Technical Specifications LCO required action completion time

5 — Inst.

Table S-2 Significant Transients
- Automatic turbine runback > 25% thermal reactor power
- Electric load rejection > 25% full electrical load
- Reactor scram
- ECCS injection
- Thermal power oscillations > 10%

SS5.1 — Modes: 1 2 3

Loss of > approximately 75% of annunciation or indication on all of the following Control Room panels for ≥ 15 min. (Note 4):
- 2CEC*PNL601
- 2CEC*PNL602
- 2CEC*PNL603
- 2CEC*PNL851
- 2CEC*PNL852

AND
A significant transient is in progress, Table S-2
AND
Compensatory indications are unavailable (Plant Computer, SPDS)

SA5.1 — Modes: 1 2 3

UNPLANNED loss of > approximately 75% of annunciation or indication on all of the following Control Room panels for ≥ 15 min. (Note 4):
- 2CEC*PNL601
- 2CEC*PNL602
- 2CEC*PNL603
- 2CEC*PNL851
- 2CEC*PNL852

AND EITHER:
A significant transient is in progress, Table S-2
OR
Compensatory indications are unavailable (Plant Process Computer, SPDS)

SU5.1 — Modes: 1 2 3

UNPLANNED loss of > approximately 75% of annunciation or indication on all of the following Control Room panels for ≥ 15 min. (Note 4):
- 2CEC*PNL601
- 2CEC*PNL602
- 2CEC*PNL603
- 2CEC*PNL851
- 2CEC*PNL852

6 — Comm.

General Emergency: None

Table S-3 Communications Systems

System	Onsite (internal)	Offsite (external)
PBX (normal dial telephones)	X	X
Gaitronics	X	
Station radio (portable)	X	
Control Room installed satellite phones (non portable)		X
ENS		X
RECS		X
UHF radios		X

SU6.1 — Modes: 1 2 3

Loss of **all** Table S-3 onsite (internal) communication methods affecting the ability to perform routine operations
OR
Loss of **all** Table S-3 offsite (external) communication methods affecting the ability to perform offsite notifications

7 — Fuel Clad Degradation

General Emergency: None

SU7.1 — Modes: 1 2 3

Reactor coolant activity > 4 μCi/gm I-131 Equivalent

SU7.2 — Modes: 1 2 3

Offgas radiation DRMS high (red) alarm for > 15 min.

8 — RCS Leakage

General Emergency: None

Site Area Emergency: None

Alert: None

SU8.1 — Modes: 1 2 3

Unidentified or reactor coolant pressure boundary leakage > 10 gpm
OR
Identified reactor coolant leakage > 25 gpm

F — Fission Product Barrier Degradation

FG1.1 — Modes: 1 2 3

Loss of **ANY** two fission product barriers
AND
Loss or potential loss of third fission product barrier (Table F-1)

FS1.1 — Modes: 1 2 3

Loss or potential loss of **ANY** two fission product barriers (Table F-1)

FA1.1 — Modes: 1 2 3

ANY loss or **ANY** potential loss of **EITHER** Fuel Clad barrier **OR** RCS barrier (Table F-1)

FU1.1 — Modes: 1 2 3

ANY loss or **ANY** potential loss of Containment barrier (Table F-1)

Table F-1 Fission Product Barrier Matrix

	Fuel Clad Barrier		Reactor Coolant System Barrier		Containment Barrier	
	Loss	Potential Loss	Loss	Potential Loss	Loss	Potential Loss
A RPV Water Level	1. Primary Containment Flooding is required	1. RPV water level **cannot** be restored and maintained above -14 in. following depressurization of the RPV or RPV water level **cannot** be determined	1. RPV water level **cannot** be restored and maintained above -14 in. or RPV water level **cannot** be determined	None	None	1. Primary Containment Flooding is required
B Primary Containment Pressure / Temperature	None	None	2. Primary Containment pressure > 1.68 psig due to RCS leakage	None	1. Primary Containment pressure rise followed by a rapid UNPLANNED drop in Primary Containment pressure 2. Primary Containment pressure response **not** consistent with LOCA conditions	2. Primary Containment pressure > 45 psig and rising 3. Explosive mixture exists inside Primary Containment (≥ 6% H_2 and ≥ 5% O_2) 4. Suppression pool temperature and RPV pressure **cannot** be maintained below the Heat Capacity Temperature Limit (N2-EOP-PC Figure M)
C Isolation	None	None	3. Release pathway exists outside Primary Containment resulting from isolation failure in **ANY** of the following systems (excluding normal process system flowpaths from an UNISOLABLE system): • Main steam line • RCIC steam line • RWCU • Feedwater 4. RPV blowdown is required	1. UNISOLABLE primary system leakage outside Primary Containment as indicated by exceeding **EITHER**: RB area temperature above an isolation setpoint **OR** RB area radiation above an alarm setpoint	3. Failure of **all** Primary Containment isolation valves in **ANY** one line to close following auto or manual initiation **AND** Direct downstream pathway outside Primary Containment and to the environment exists 4. Intentional Primary Containment venting per EOPs 5. UNISOLABLE primary system leakage outside Primary Containment as indicated by exceeding **EITHER**: RB area maximum safe temperature value (N2-EOP-SC Detail S) **OR** RB area radiation > 8.00E+3 mR/hr	None
D Rad	2. Drywell area radiation ≥ 3100 R/hr (3.1 E6 mRem/hr) 3. Reactor coolant activity > 300 μCi/gm I-131 Equivalent	None	5. Drywell area radiation ≥ 41 R/hr (4.1 E4 mRem/hr)	None	None	5. Drywell area radiation ≥ 6.0 E4 R/hr (6.0 E7 mRem/hr)
E Judgment	4. **ANY** condition in the opinion of the Emergency Director that indicates loss of the Fuel Clad barrier	2. **ANY** condition in the opinion of the Emergency Director that indicates potential loss of the Fuel Clad barrier	6. **ANY** condition in the opinion of the Emergency Director that indicates loss of the Reactor Coolant System barrier	2. **ANY** condition in the opinion of the Emergency Director that indicates potential loss of the Reactor Coolant System barrier	6. **ANY** condition in the opinion of the Emergency Director that indicates loss of the Containment barrier	6. **ANY** condition in the opinion of the Emergency Director that indicates potential loss of the Containment barrier

EAL Identifier

XXX.X
- Category (R, H, E, S, F, C)
- Emergency classification (G, S, A, U)
- Subcategory number (1 if no subcategory)
- Sequential number within subcategory/classification

MODE 1, 2 or 3