ArevaEPRDCPEm Resource

From:	WELLS Russell (AREVA) [Russell.Wells@areva.com]
Sent:	Friday, May 06, 2011 7:55 PM
То:	Tesfaye, Getachew
Cc:	WILLIFORD Dennis (AREVA); PEDERSON Ronda (AREVA); Miernicki, Michael; BENNETT
	Kathy (AREVA); DELANO Karen (AREVA); HALLINGER Pat (EXTERNAL AREVA); ROMINE
	Judy (AREVA); RYAN Tom (AREVA)
Subject:	Draft Response to U.S. EPR Design Certification Application RAI No. 449, FSARCh. 19 NEW
	PHASE 4 RAI,Questions 19-339 and 19-340
Attachments:	RAI 449 Questions 19-339 and 19-340 Response US EPR DC - DRAFT.PDF

Getachew,

Attached is a draft response for RAI No. 449, Questions 19-339 and 19-340 as shown below in advance of the June 3, 2011 final date.

Let me know if the staff has questions or if this can be sent as a final response.

Sincerely,

Russ Wells U.S. EPR Design Certification Licensing Manager **AREVA NP, Inc.** 3315 Old Forest Road, P.O. Box 10935 Mail Stop OF-57 Lynchburg, VA 24506-0935 Phone: 434-832-3884 (work) 434-942-6375 (cell) Fax: 434-382-3884 Russell.Wells@Areva.com

From: WELLS Russell (RS/NB)
Sent: Friday, April 22, 2011 4:17 PM
To: 'Tesfaye, Getachew'
Cc: WILLIFORD Dennis (RS/NB); PEDERSON Ronda (EP/PE); BENNETT Kathy (RS/NB); DELANO Karen (RS/NB); ROMINE Judy (RS/NB); RYAN Tom (RS/NB)
Subject: Response to U.S. EPR Design Certification Application RAI No. 449, FSARCh. 19 NEW PHASE 4 RAI, Supplement 2

Getachew,

AREVA NP Inc. (AREVA NP) provided a schedule for a technically correct and complete response to RAI No. 449 on January 24, 2011. Supplement 1 was submitted to the NRC on February 24, 2011 to revise the schedule.

To allow additional time to interact with NRC staff and to complete a new technical report related to design features to protect against the impact of a large commercial aircraft impact, the schedule for a technically correct and complete response to the 2 questions has been changed as provided below.

Question #	Response Date

RAI 449 — 19-339	June 3, 2011
RAI 449 — 19-340	June 3, 2011

Sincerely,

Russ Wells U.S. EPR Design Certification Licensing Manager **AREVA NP, Inc.** 3315 Old Forest Road, P.O. Box 10935 Mail Stop OF-57 Lynchburg, VA 24506-0935 Phone: 434-832-3884 (work) 434-942-6375 (cell) Fax: 434-382-3884 <u>Russell.Wells@Areva.com</u>

From: WELLS Russell (RS/NB)
Sent: Thursday, February 24, 2011 3:57 PM
To: 'Tesfaye, Getachew'
Cc: ROMINE Judy (RS/NB); BENNETT Kathy (RS/NB); SALAS Pedro (RS/NB); BRYAN Martin (External RS/NB); SALAS Pedro (RS/NB); PEDERSON Ronda (EP/PE)
Subject: Response to U.S. EPR Design Certification Application RAI No. 449, FSARCh. 19 NEW PHASE 4 RAI, Supplement 1

Getachew,

AREVA NP Inc. provided a schedule for technically correct and complete response to RAI No. 449 on January 24, 2011. To allow time for interaction between AREVA and the NRC staff, a revised schedule for submittal of the final response is provided in this e-mail.

The schedule for technically correct and complete responses to the questions has been revised and is provided below:

Question #	Response Date
RAI 449 — 19-339	April 29, 2011
RAI 449 — 19-340	April 29, 2011

Sincerely,

Russ Wells U.S. EPR Design Certification Licensing Manager AREVA NP, Inc. 3315 Old Forest Road, P.O. Box 10935 Mail Stop OF-57 Lynchburg, VA 24506-0935 Phone: 434-832-3884 (work) 434-942-6375 (cell) Fax: 434-382-3884 <u>Russell.Wells@Areva.com</u> From: BRYAN Martin (External RS/NB)
Sent: Monday, January 24, 2011 12:13 PM
To: 'Tesfaye, Getachew'
Cc: DELANO Karen (RS/NB); ROMINE Judy (RS/NB); BENNETT Kathy (RS/NB); SALAS Pedro (RS/NB)
Subject: Response to U.S. EPR Design Certification Application RAI No. 449, FSARCh. 19 NEW PHASE 4 RAI

Getachew,

Attached please find AREVA NP Inc.'s response to the subject request for additional information (RAI). The attached file, "RAI 449 Response US EPR DC.pdf" provides a schedule since a technically correct and complete response to the 2 question (s) is not provided.

The following table indicates the respective pages in the response document, "RAI 449 Response US EPR DC.pdf," that contain AREVA NP's response to the subject questions.

Question #	Start Page	End Page
RAI 449 — 19-339	2	2
RAI 449 — 19-340	3	3

A complete answer is not provided for the 2 questions. The schedule for a technically correct and complete response to these questions is provided below.

Question #	Response Date
RAI 449 — 19-339	February 24, 2011
RAI 449 — 19-340	February 24, 2011

Sincerely,

Martin (Marty) C. Bryan U.S. EPR Design Certification Licensing Manager AREVA NP Inc. Tel: (434) 832-3016 702 561-3528 cell Martin.Bryan.ext@areva.com

From: Tesfaye, Getachew [mailto:Getachew.Tesfaye@nrc.gov]
Sent: Wednesday, December 08, 2010 9:12 AM
To: ZZ-DL-A-USEPR-DL
Cc: Xu, Jim; Hawkins, Kimberly; Ford, Tanya; Carneal, Jason; Colaccino, Joseph; ArevaEPRDCPEm Resource
Subject: U.S. EPR Design Certification Application RAI No. 449 (5104), FSARCh. 19 NEW PHASE 4 RAI

Attached please find the subject requests for additional information (RAI). A draft of the RAI was provided to you on October 10, 2010, and on December 6, 2010, you informed us that the RAI is clear and no further clarification is needed. As a result, no change is made to the draft RAI. The schedule we have established for review of your application assumes technically correct and complete responses within 30 days of receipt of RAIs, excluding the time period of **December 24, 2010 thru January 3, 2011, to account for the holiday season** as discussed with AREVA NP Inc. For any RAIs that cannot be answered **within 45 days**, it is expected that a date for receipt of this information will be provided to the staff within the 40-day period so that the staff can assess how this information will impact the published schedule.

Thanks,

Getachew Tesfaye Sr. Project Manager NRO/DNRL/NARP (301) 415-3361 Hearing Identifier: AREVA_EPR_DC_RAIs Email Number: 2939

Mail Envelope Properties (1F1CC1BBDC66B842A46CAC03D6B1CD410456C982)

Subject:Draft Response to U.S. EPR Design Certification Application RAI No. 449,FSARCh. 19 NEW PHASE 4 RAI,Questions 19-339 and 19-340Sent Date:5/6/2011 7:54:51 PMReceived Date:5/6/2011 7:55:00 PMFrom:WELLS Russell (AREVA)

Created By: Russell.Wells@areva.com

Recipients:

"WILLIFORD Dennis (AREVA)" < Dennis.Williford@areva.com> Tracking Status: None "PEDERSON Ronda (AREVA)" <Ronda.Pederson@areva.com> **Tracking Status: None** "Miernicki, Michael" < Michael.Miernicki@nrc.gov> Tracking Status: None "BENNETT Kathy (AREVA)" <Kathy.Bennett@areva.com> Tracking Status: None "DELANO Karen (AREVA)" <Karen.Delano@areva.com> **Tracking Status: None** "HALLINGER Pat (EXTERNAL AREVA)" <Pat.Hallinger.ext@areva.com> Tracking Status: None "ROMINE Judy (AREVA)" <Judy.Romine@areva.com> Tracking Status: None "RYAN Tom (AREVA)" <Tom.Ryan@areva.com> Tracking Status: None "Tesfaye, Getachew" < Getachew. Tesfaye@nrc.gov> Tracking Status: None

Post Office: AUSLYNCMX02.adom.ad.corp

Files	Size	Date & Time
MESSAGE	5993	5/6/2011 7:55:00 PM
RAI 449 Questions 19-339 and	19-340 Response US EPF	R DC - DRAFT.PDF

343376

OptionsPriority:StandardReturn Notification:NoReply Requested:NoSensitivity:NormalExpiration Date:Recipients Received:

Response to

Request for Additional Information No. 449 (5104), Questions 19-339 through 19-340

12/8/2010

U. S. EPR Standard Design Certification AREVA NP Inc. Docket No. 52-020 SRP Section: 19 - Probabilistic Risk Assessment and Severe Accident Evaluation Application Section: 19.2.7

QUESTIONS for Structural Engineering Branch 2 (ESBWR/ABWR Projects) (SEB2)

Question 19-339:

OPEN ITEM

AIRCRAFT IMPACT ASSESSMENT

U.S. EPR FSAR, Tier 2, Revision 2, Section 19.2.7.4 identifies design features credited for meeting the acceptance criteria of 10 CFR 50.150(a)(1). The descriptions of the design features are typically provided in other sections of FSAR. Cross-references to respective sections of the FSAR with regards to the description of the credited design features should be provided in Section 19.2.7.4 to assist the staff review. The applicant is requested to include in FSAR Section 19.2.7.4 references to other FSAR sections where the design features credited for meeting the acceptance criteria of 10 CFR 50.150(a)(1) are described. The applicant is also requested to describe in the FSAR how each of the identified design features is used to meet the acceptance criteria of 10 CFR 50.150(a)(1).

Response to Question 19-339:

U.S. EPR FSAR Tier 2, Section 19.2.7.4 will be revised to include an additional description and cross references to the respective U.S. EPR FSAR sections that describe the design features credited for conformance with 10 CFR 50.150. In addition, U.S. EPR FSAR Tier 2, Section 19.2.7.4 will be revised to describe how each of the identified design features meets the acceptance criteria of 10 CFR 50.150.

New Technical Report ANP-10317, "Design Requirements for the U.S. EPR Aircraft Hazard Protection Structures," documents design requirements that are credited for conformance with 10 CFR 50.150. U.S. EPR FSAR Tier 2, Section 19.2.8 will be revised to add a reference to Technical Report ANP-10317. This report will also be added to the list of reports referenced in U.S. EPR FSAR Tier 2, Table 1.6-1.

FSAR Impact:

U.S. EPR FSAR Tier 2, Section 19.2.7.4, Section 19.2.8, and Table 1.6-1 will be revised as described in the response and indicated on the enclosed markup.

Question 19-340:

OPEN ITEM

AIRCRAFT IMPACT ASSESSMENT

FSAR, Tier 2, Revision 2, Section 19.2.7.5, under physical damage, indicates that analyses were performed for the containment, safeguard and fuel buildings, and consideration of physical separation and redundant trains. However, it is not clear how these analyses address the protection of all design features as provided in FSAR, Tier 2, Revision 2,Section 19.2.7.4 such as: ECCS, decay heat removal systems, emergency feedwater tanks and emergency core cooling water. The applicant is requested to clarify the above issue and revise the FSAR section as needed.

Response to Question 19-340:

U.S. EPR FSAR Tier 2, Section 19.2.7.5 states that:

"Finite Element Analyses indicate that interior areas of the Safeguard Building, Fuel Building, or Containment Building are not susceptible to damage due to physical perforation of aircraft components into the structures."

Therefore, components housed in these structures, such as the emergency core cooling system (ECCS) components, decay heat removal systems, emergency feedwater (EFW) tanks, and emergency core cooling water, are not susceptible to damage resulting from physical perforation of aircraft components into the structures.

U.S. EPR FSAR Tier 2, Section 19.2.7.4 and Section 19.2.7.5 will be revised to clarify that the systems and components listed in U.S. EPR FSAR Tier 2, Section 19.2.7.4 are housed in the Safeguard, Fuel, and Containment Buildings.

FSAR Impact:

U.S. EPR FSAR Tier 2, Section 19.2.7.4 and Section 19.2.7.5 will be revised as described in the response and indicated on the enclosed markup.

U.S. EPR Final Safety Analysis Report Markups



Table 1.6-1—Reports Referenced Sheet 3 of 4

Report No. (See Notes 1, 2, and 3)	RAI 449, Q. 19-339 RAI 456, Q. 19-342 Title	Date Submitted to NRC	FSAR Section Number(s)
<u>ANP-10317</u>	Design Requirements for the U.S. EPR Aircraft Hazard Protection Structures	<u>5/11</u>	<u>19.2.7.4</u>
<u>ANP-10318P</u>	<u>Pipe Rupture External Loading Effects</u> on U.S. EPR Essential Structures, Systems, and Components Technical Report	<u>3/11</u>	<u>3.6.2</u>
BAW-10132-A	Analytical Methods Description – Reactor Coolant System Hydrodynamic Loadings During a Loss-of-Coolant Accident	7/20/79	App. 3C
BAW-10133P-A BAW-10133-A Revision 1, Addendum 1 and 2	Mark-C Fuel Assembly LOCA-Seismic Analysis	10/30/00	4.2
BAW-10147P-A BAW-10147-A Revision 1	Fuel Rod Bowing in Babcock & Wilcox Fuel Designs	6/28/83	4.2, 4.4
BAW-10156-A Revision 1	LYNXT, Core Transient Thermal- Hydraulic Program	8/18/93	4
BAW-10163P-A BAW-10163-A	Core Operating Limit Methodology for Westinghouse Designed PWRs	6/2/89	4.3 and 16
BAW-10164P-A BAW-10164NP-A Revision 6	RELAP5/MOD2-B&W - An Advanced Computer Program for Light Water Reactor LOCA and Non-LOCA Transient Analysis	11/20/07	3.9.1, 6.2, and 8.4
BAW-10168P-A BAW-10168-A Revision 3	BWNT Loss-of-Coolant Accident Model for Recirculating Steam Generator Plants	1/31/97	6.2
BAW-10169P-A BAW-10169-A	B&W Safety Analysis Methodology for Recirculating Steam Generator Plants	11/28/89	6.2
BAW-10172P-A BAW-10172NP-A	Mark-BW Mechanical Design Report	12/19/89 (Note 4)	4.2
BAW-10183P-A BAW-10183-A	Fuel Rod Gas Pressure Criterion (FRGPC)	7/24/95	4.2 and 4.4
BAW-10186P-A BAW-10186NP-A Revision 2	Extended Burnup Evaluation	1/30/04	4.2 and 4.4



mitigation design alternatives from previous industry studies and from U.S. EPR PRA insights was performed against broad acceptance criteria. None of the SAMDA candidates met the criteria; therefore, the overall conclusion is that no additional plant modifications are cost beneficial to implement due to the robust design of the U.S. EPR with respect to prevention and mitigation of severe accidents.

19.2.7 Beyond Design Basis Large Commercial Aircraft Impact Assessment

19.2.7.1 Introduction

The U.S. EPR design has been evaluated to demonstrate that it has inherent protection to avoid or mitigate, to the extent practical and with reduced reliance on operator actions, the effects of a large commercial aircraft impact.

19.2.7.2Assessment Scope

The scope of the assessment was to demonstrate—using realistic analyses— that the U.S. EPR design has design features and functional capabilities such that with reduced reliance on operator actions:

- The reactor core remains cooled, OR the containment remains intact.
- Spent fuel cooling, OR spent fuel pool integrity is maintained.

19.2.7.3 Methodology

RAI 456, Q. 19-345

The methodology used to demonstrate compliance with 10 CFR 50.150 is NEI 07-13, Revision 7, "Methodology for Performing Aircraft Impact Assessments for New Plant Designs," dated May 2009 (Reference 18), applying the aircraft impact loading function provided by the NRC (Reference 19). <u>The methodology of NEI 07-13</u>, <u>Revision 7 was followed with no exceptions</u>.

The methodology is subdivided into two major evaluations:

• Containment and Spent Fuel Pool Evaluation.

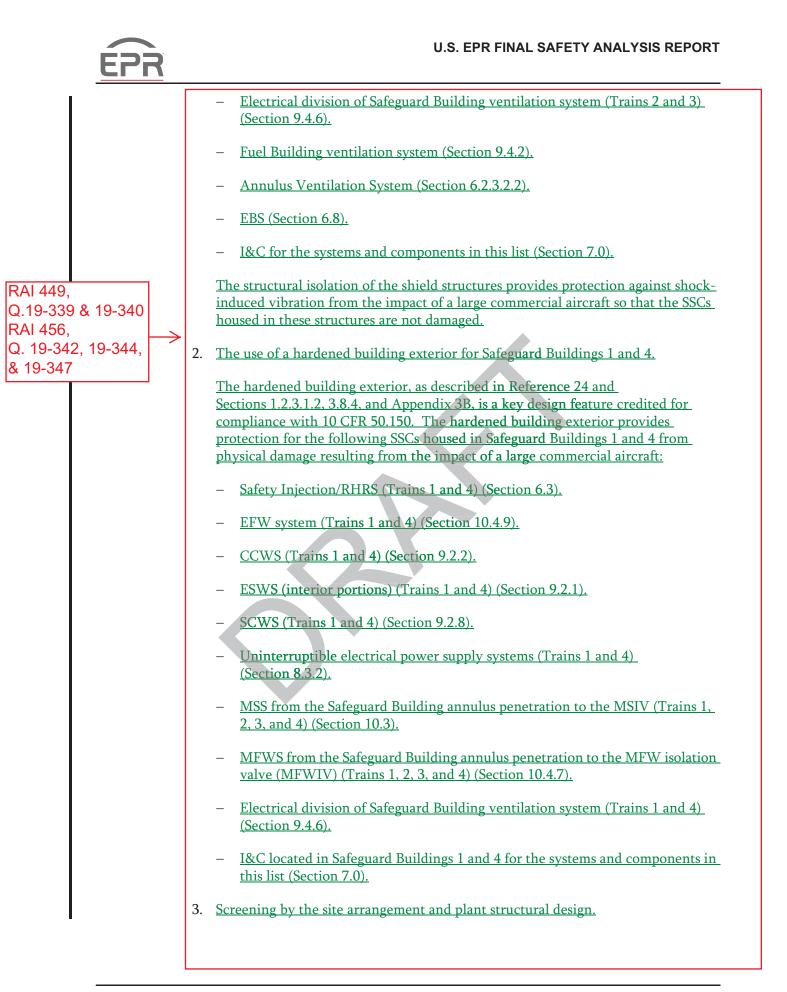
Two distinct types of structural failure modes were evaluated for the containment structure and spent fuel pool: local failure (i.e., scabbing and perforation) caused by aircraft fuselage or engine impact and global structural failure (i.e., plastic collapse) caused by impact of the complete aircraft.

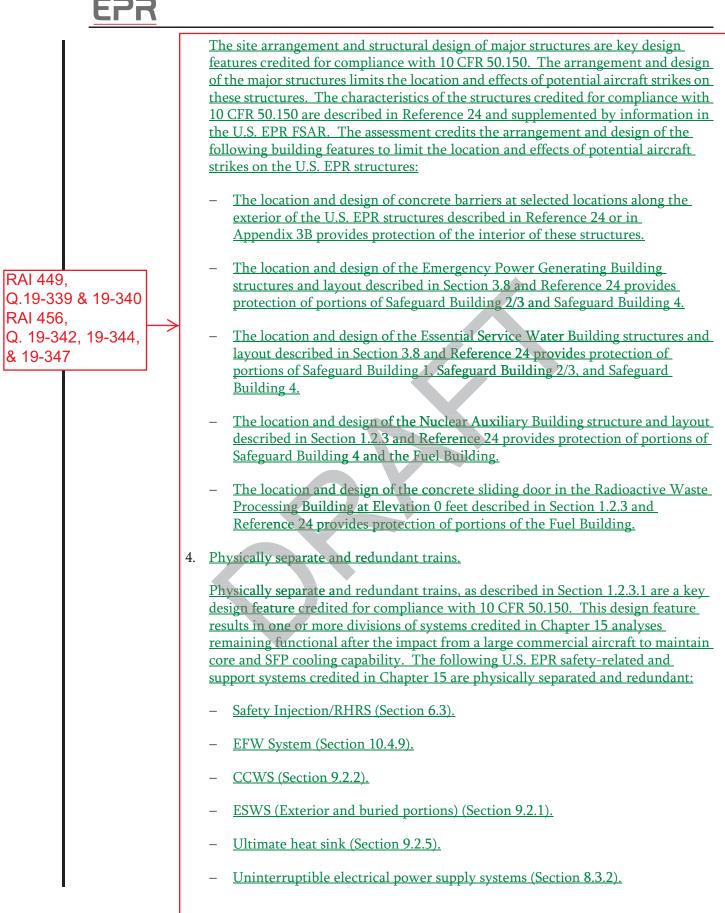
• Heat Removal Evaluation.

The evaluation considered physical, shock, and fire effects of a large commercial aircraft impact that can cause damage to systems needed to maintain cooling of fuel in the vessel and the spent fuel pool.



19.2.7.4	Design Features Credited for Conformance with 10 CFR 50.150
	The U.S. EPR design incorporates system redundancy, diversity, and independence.
	The key features incorporated to mitigate the effects of potential impact of aircrafts
	that are credited for compliance with 10 CFR 50.150 are as follows:
	1. <u>The use of individual hardened and isolated shield structures specific to the</u> <u>Containment, Fuel Building, and Safeguard Building 2/3.</u>
AI 449, 0.19-339 & 19-340 AI 456, 0. 19-342, 19-344, 19-347	 The hardened and isolated shield structures, as described in ANP-10317, "Design Requirements for the U.S. EPR Aircraft Hazard Protection Structures" (Reference 24) and Sections 1.2.3.1.2, 3.8.4, Appendix 3B, and Appendix 3E.1.7, are a key design feature credited for compliance with 10 CFR 50.150. The use of hardened and isolated shield structures provides protection for the Containment, Fuel Building, and Safeguard Building 2/3 structures and the following credited SSCs that are housed in these structures: Containment vessel (Section 3.8.2).
	– <u>RCS (Section 5.0).</u>
	 Emergency core cooling water source, IRWST (Section 6.3).
	 Main steam system (MSS) from the SGs to the Safeguard Building annulus penetration (Trains 1, 2, 3, and 4) (Section 10.3).
	 Main feedwater system (MFWS) from the SGs to the Safeguard Building annulus penetration (Trains 1, 2, 3, and 4) (Section 10.4.7).
	– <u>SFP (Section 9.1).</u>
	- Fuel pool cooling and purification system (Section 9.1.3).
	– <u>MCR (Section 6.4).</u>
	– <u>MCR HVAC (Section 9.4.1).</u>
	– Safety injection/RHRS (Trains 2 and 3) (Section 6.3).
	- EFW system (Trains 2 and 3) (Section 10.4.9).
	- <u>CCWS (Trains 2 and 3) (Section 9.2.2).</u>
	- ESWS (interior portions) (Trains 2 and 3) (Section 9.2.1).
	 <u>Uninterruptible electrical power supply systems (Trains 2 and 3)</u> <u>(Section 8.3.2).</u>
	 Safety chilled water system (SCWS) (Trains 2 and 3) (Section 9.2.8).







- <u>Emergency power supply system (EPSS) and EDG (Section 8.3.1).</u>

In the event of an aircraft impact threat while the reactor is at power operation, NEI 07-13 (Reference 18) allows the assumption that the operators will have advance warning to take manual action to shutdown the reactor prior to impact. Because the systems necessary to scram the reactor are housed in the hardened and isolated Shield Building structures, there is no potential for impact damage that would prevent a scram. Following shutdown, one or more trains of the safetyrelated and support systems in this section are available to maintain core cooling and SFP cooling.

For an aircraft impact that occurs during shutdown with the reactor head removed and the reactor pit not flooded, the same safety-related and support systems in this section are used and one or more trains of these systems remain available to maintain core cooling.

5. Fire barriers and fire protection features.

Selected fire barriers, fire dampers, fire doors, and penetration seals are three-hour rated to prevent fire damage in one division from spreading to an adjacent division. Selected structural elements and blast dampers are 5 psid rated to prevent explosion effects from spreading to adjacent areas. The credited fire barriers, fire dampers, fire doors, penetration seals, structural elements, and blast dampers are identified on the fire zone layout figures in Appendix 9A.

The U.S. EPR design incorporates system redundancy, diversity, and independence. Two key features incorporated to mitigate the effects of potential impact of aircraftsare:

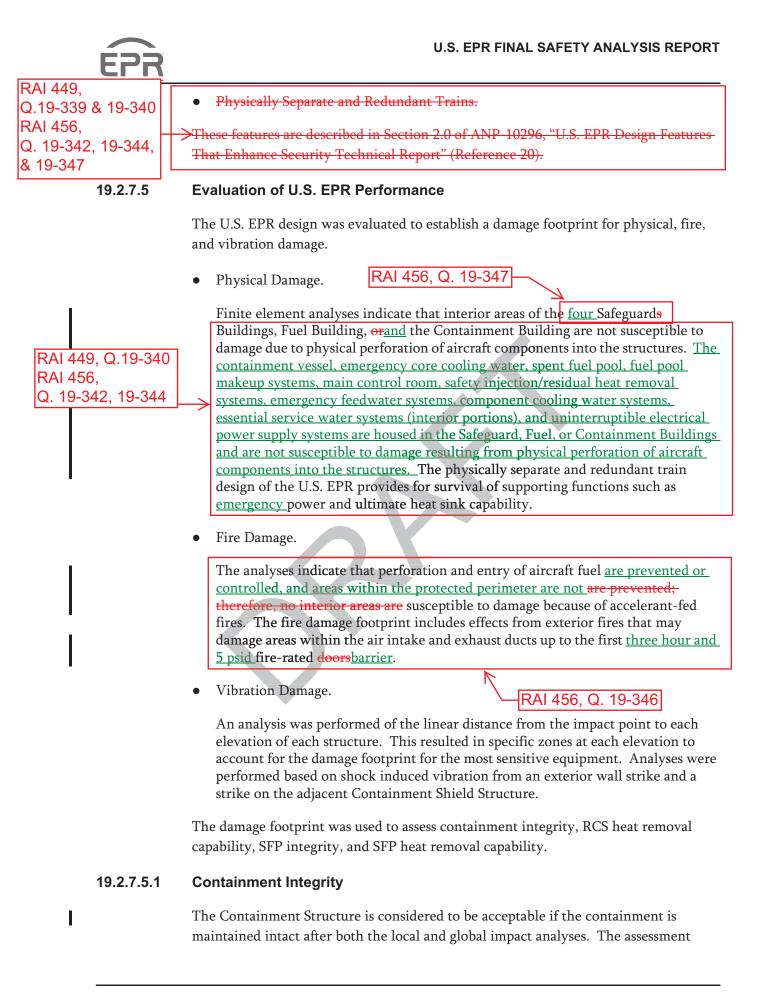
- The use of a hardened shield building structure.
- The site arrangement of major structures.

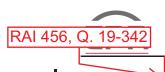
The following U.S. EPR design features are credited for compliance with 10 CFR 50.150:

- Hardened Shield Building over Containment.
- Hardened Shield Building over Fuel Building.
- Hardened Shield Building over Control Room.
- Hardened Shield Building over ECCS Components.
- Hardened Decay Heat Removal Systems.
- Hardened and Internalized Emergency Feedwater Tanks.
- Hardened and Internalized Emergency Core Cooling Water.

Q.19-339 & 19-340

RAI 449.





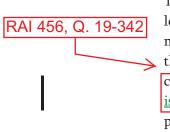
RAI 456, Q. 19-342

concluded that the hardened and isolated containment shield structure was not perforated, and no significant structural damage occurred because of either local or global impacts. The Containment Building, inside the Containment Shield Structure, was not impacted by the aircraft or any associated debris. Therefore, the containment performance, including ultimate pressure capacity, is unaffected. Under these conditions, no physical damage or fire damage inside containment needs to be considered.

19.2.7.5.2 **RCS Heat Removal Capability**

The reactor coolant system heat removal is considered sufficient if the heat removal capability analyses performed conclude that sufficient heat removal equipment is available consistent with the PRA success criteria. The analyses performed demonstrated the ability of the U.S. EPR design, after the impact by a large commercial aircraft, to maintain functionality of one or more divisions of systems credited in U.S. EPR FSAR Tier 2, Chapter 15 with providing reactor core cooling under accident conditions. <u>The U.S. EPR design has features such as hardened and</u> isolated shield structures, a strategic site arrangement and plant structural design, fire barriers, and the physically separate and redundant trains. These features contribute to the success of one or more divisions of systems credited in Chapter 15 to maintain functionality to provide reactor core cooling after the impact of a large commercial aircraft. In addition, an aircraft impact does not create any new events that have not been analyzed in Chapter 15. NEI 07-13 does not require postulating a Chapter 15 event concurrent with an aircraft impact that does not perforate the structures containing RCS piping. Therefore, the RCS heat removal capability evaluation demonstrates additional margin in the U.S. EPR design. RAI 456, Q. 19-343

19.2.7.5.3 SFP Integrity



The SFP integrity is considered to be maintained if the fuel pool liner does not have a leakage path below the minimum water level, the fuel is protected and there would be no unacceptable releases of radionuclides to the environment. Analyses demonstrate that no physical damage to the interior of the Fuel Building results from the aircraft crash. The prevention of aircraft perforation of the exterior wall of the hardened and isolated shield structure surrounding the Fuel Building ensures that the SFP is not perforated and that SFP integrity is maintained.

19.2.7.5.4 **SFP Heat Removal Capability**

RAI 456, Q. 19-342

With the SFP integrity maintained, SFP cooling is provided consistent with the PRA. The availability of the make-up systems is assured due to the integrity of the hardened. and isolated shield structure surrounding the Fuel Building. The shield structure provides physical and fire damage protection against an aircraft impact. The isolation of this structure provides continued functionality of the SFP makeup and protection



against shock induced vibrations. Fuel Building exterior walls. The fire protection system provides the capability to fill the Spent Fuel Pool.

19.2.7.6 Conclusions

RAI 456, Q. 19-342

RAI 456, Q. 19-344

The U.S. EPR has inherent protection to avoid or mitigate₃, to the extent practical and with reduced reliance on operator actions, the effects of an aircraft impact. <u>Although the regulations require meeting only two of the acceptance criteria, the assessment summarized above confirms the U.S. EPR design meets the four acceptance criteria in 10 CFR 50.150(a)(1) by following the methodology described in NEI-07-13 (Reference 18). <u>The assessment confirmed that the U.S. EPR design meets the four acceptance criteria.</u> The reactor remains cooled, <u>AND</u> the containment remains intact₃; <u>ANDand</u> spent fuel cooling is maintained, <u>ANDand</u> spent fuel pool integrity isare maintained. Accordingly, the U.S. EPR design features and functional capabilities provide for adequate protection of public health and safety in the event of an impact of a large commercial aircraft as required by 10 CFR 1050.150. <u>In fact, by exceeding the minimum acceptance criteria, the U.S. EPR design maintains significant margin beyond the minimum requirements specified in 10 CFR 50.150.</u></u>

19.2.8 References

- 1. ANP-10268P-A, Revision 0, "U.S. EPR Severe Accident Evaluation Topical Report," AREVA NP Inc, February, 2008.
- 2. Fauske and Associates, Inc., 1994a. MAAP4—Modular Accident Analysis Program for LWR Power Plants, vol. 2, Part 1: Code Structure and Theory, prepared for Electric Power Research Institute, May 1994.
- 3. SECY-90-016, "Evolutionary Light Water (LWR) Certification Issues and Their Relationship to Current Regulatory Requirements," U.S. Nuclear Regulatory Commission, issued January 12, 1990, and the corresponding SRM, issued June 26, 1990.
- SECY-93-087, "Policy, Technical, and Licensing Issues Pertaining to Evolutionary and Advanced Light-Water (ALWR) Designs," U.S. Nuclear Regulatory Commission, issued April 2, 1993, and the corresponding SRM, issued July 21, 1993.
- WASH-1400 (NUREG-75/014), "Reactor Safety Study-An Assessment of Accident Risks in U.S. Commercial Nuclear Power Plants," U.S, Nuclear Regulatory Commission, October 1975.
- 6. NUREG-1116, "A Review of Current Understanding of the Potential for Containment Failure Arising from In-Vessel Steam Explosions," Steam Explosion Review Group (SERG), U.S. Nuclear Regulatory Commission, February 1985.
- 7. Theofanous, T. G., et al, DOE/ID-10489, "The Study of Steam Explosions in Nuclear Systems," Department of Energy, June 1996.



- 8. Deleted.
- 9. Wilks, S.S., "Determination of Sample Sizes for Setting Tolerance Limits," Ann. Math. Stat., Vol. 12, pp. 91-96, 1941.
- EPRI TR-103413, "The MELTSPREAD-1 Computer Code for the Analysis of Transient Spreading and Cooling of High-Temperature Melts – Code Manual," Electric Power Research Institute, December 1993.
- 11. Breitung, W., et al, "Flame Acceleration and Deflagration-to-Detonation Transition in Nuclear Safety," NEA/CSNI-(2000) 7, October 2000.
- 12. Tutu, N.K., T. Ginsberg, and L. Fintrok (1988). "Low Pressure Cutoff for Melt Dispersal from Reactor Cavities," Fourth Proceedings of Nuclear Thermal Hydraulics, 29-37.
- 13. Pilch, M. M., et al, "Resolution of the Direct Containment Heating Issue for All Westinghouse Plants With Large Dry Containments or Subatmospheric Containments," NUREG/CR-6338, SAND95-2381, 1996.
- Meyer, L., et al, "Melt Dispersion and Direct Containment Heating (DCH) Experiments in the DISCO-H Test Facility," FZK Report FZKA 6988, ISSN 0947-8620, May 2004.
- 15. ANP-10290, Revision 1, "Environmental Report Standard Design Certification," AREVA NP Inc, September 2009.
- 16. NEI 05-01 (Rev A), "Severe Accident Mitigation Alternatives (SAMA) Analysis, Guidance Document," Nuclear Energy Institute, November 2005.
- 17. NUREG/BR-0184, "Regulatory Analysis Technical Evaluation Handbook," U.S, Nuclear Regulatory Commission, January 1997.
- 18. NEI 07-13, "Methodology for Performing Aircraft Impact Assessments for New Plant Designs," Revision 7, May 2009.

19. Letter from D. Matthews, NRC to R. Ford, AREVA NP, "Approval of AREVA NP Inc. Safeguards Protection Program and Reviewing Official, and Transmittal of Beyond Design Basis, Large Commercial Aircraft Characteristics Specified by the Commission", December 21, 2007.

- 20. <u>Deleted.ANP-10296</u>, Revision 0, "U.S. EPR Design Features that Enhance-Security," AREVA NP Inc., December 2008.
- 21. <u>Achenbach, J.A., Miller, R.B., Srinivas, V., "Large-Scale Hydrogen Burn</u> <u>Equipment Experiments," EPRI NP-4354, Electric Power Research Institute, 1985.</u>
- 22. <u>NUREG/CR-5334, "Severe Accident Testing of Electrical Penetration Assemblies,"</u> <u>SAND89-0327, November 1989.</u>

RAI 449, Q.19-339 RAI 456, Q. 19-342



