



## U.S. NUCLEAR REGULATORY COMMISSION STANDARD REVIEW PLAN

### 8.1 ELECTRIC POWER - INTRODUCTION

#### REVIEW RESPONSIBILITIES

**Primary** - Organization responsible for electrical engineering

**Secondary** - None

#### I. AREAS OF REVIEW

The specific areas of review are as follows:

1. The applicant's description of the offsite power system with regard to the interrelationships between the nuclear unit, the utility grid, and the interconnecting grids.
2. The applicant's description of the onsite power systems with regard to the availability of sufficient power to mitigate design-basis events given a loss of the offsite power system and a single failure in the onsite power system.
3. The applicant's description of the capability to withstand and recover from a station blackout event of a specified duration.
4. The acceptance criteria to be implemented in the design of the above systems.

Revision 4 - May 2011

---

### USNRC STANDARD REVIEW PLAN

This Standard Review Plan, NUREG-0800, has been prepared to establish criteria that the U.S. Nuclear Regulatory Commission staff responsible for the review of applications to construct and operate nuclear power plants intends to use in evaluating whether an applicant/licensee meets the NRC's regulations. The Standard Review Plan is not a substitute for the NRC's regulations, and compliance with it is not required. However, an applicant is required to identify differences between the design features, analytical techniques, and procedural measures proposed for its facility and the SRP acceptance criteria and evaluate how the proposed alternatives to the SRP acceptance criteria provide an acceptable method of complying with the NRC regulations.

The standard review plan sections are numbered in accordance with corresponding sections in Regulatory Guide (RG)1.70, "Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants (LWR Edition)." Not all sections of RG 1.70 have a corresponding review plan section. The SRP sections applicable to a combined license application for a new light-water reactor (LWR) are based on RG 1.206, "Combined License Applications for Nuclear Power Plants (LWR Edition)."

These documents are made available to the public as part of the NRC's policy to inform the nuclear industry and the general public of regulatory procedures and policies. Individual sections of NUREG-0800 will be revised periodically, as appropriate, to accommodate comments and to reflect new information and experience. Comments may be submitted electronically by e-mail to [NRR\\_SRP@nrc.gov](mailto:NRR_SRP@nrc.gov).

Requests for single copies of SRP sections (which may be reproduced) should be made to the U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Reproduction and Distribution Services Section, or by fax to (301) 415-2289; or by email to [DISTRIBUTION@nrc.gov](mailto:DISTRIBUTION@nrc.gov). Electronic copies of this section are available through the NRC's public Web site at <http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr0800/>, or in the NRC's Agencywide Documents Access and Management System (ADAMS), at <http://www.nrc.gov/reading-rm/adams.html>, under ADAMS Accession No. ML111180542.

---

5. Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC). For design certification (DC) and combined license (COL) reviews, the staff reviews the applicant's proposed ITAAC associated with the structures, systems, and components (SSCs) related to this SRP section in accordance with SRP Section 14.3, "Inspections, Tests, Analyses, and Acceptance Criteria." The staff recognizes that the review of ITAAC cannot be completed until after the rest of this portion of the application has been reviewed against acceptance criteria contained in this SRP section. Furthermore, the staff reviews the ITAAC to ensure that all SSCs in this area of review are identified and addressed as appropriate in accordance with SRP Section 14.3.
6. COL Action Items and Certification Requirements and Restrictions. For a DC application, the review will also address COL action items and requirements and restrictions (e.g., interface requirements and site parameters).

For a COL application referencing a DC, a COL applicant must address COL action items (referred to as COL license information in certain DCs) included in the referenced DC. Additionally, a COL applicant must address requirements and restrictions (e.g., interface requirements and site parameters) included in the referenced DC.

### Review Interfaces

SRP Section 8.2, Section 8.3.1, Section 8.3.2, and Section 8.4 contains the specific review interfaces for each SRP section.

The specific acceptance criteria and review procedures are contained in the referenced SRP sections.

## II. ACCEPTANCE CRITERIA

### Requirements

Acceptance criteria are based on meeting the relevant requirements of the following Commission regulations:

1. Table 8-1 of this SRP section lists the acceptance criteria the staff currently applies to electric power systems. Implementation of these criteria in accordance with applicable regulatory guides and branch technical positions will provide assurance that systems will perform their design safety functions when required.
2. SRP Section 8.2, Section 8.3.1, Section 8.3.2, and Section 8.4 detail the specific acceptance criteria presented in Table 8-1. Each SRP section also describes the technical rationale for applying these criteria to reviews of electrical power systems.
3. Title 10 of the *Code of Federal Regulations* (10 CFR) 52.47(b)(1), which requires that a DC application contain the proposed ITAAC that are necessary and sufficient to provide reasonable assurance that, if the inspections, tests, and analyses are performed and the acceptance criteria met, a facility that incorporates the DC is built and will operate in conformity with the DC, the provisions of the Atomic Energy Act, and the NRC's rules and regulations;

4. 10 CFR 52.80(a), which requires that a COL application contain the proposed inspections, tests, and analyses, including those applicable to emergency planning, that the licensee shall perform, and the acceptance criteria that are necessary and sufficient to provide reasonable assurance that, if the inspections, tests, and analyses are performed and the acceptance criteria met, the facility has been constructed and will operate in conformity with the COL, the provisions of the Atomic Energy Act, and the NRC's rules and regulations.

### SRP Acceptance Criteria

Specific SRP acceptance criteria acceptable to meet the relevant requirements of the NRC's regulations identified above are as follows for the review described in this SRP section. The SRP is not a substitute for the NRC's regulations, and compliance with it is not required. However, an applicant is required to identify differences between the design features, analytical techniques, and procedural measures proposed for its facility and the SRP acceptance criteria and evaluate how the proposed alternatives to the SRP acceptance criteria provide acceptable methods of compliance with the NRC regulations.

Specific SRP acceptance criteria are contained in SRP Section 8.2, Section 8.3.1, Section 8.3.2, and Section 8.4.

### III. REVIEW PROCEDURES

The reviewer will select material from the procedures described below, as may be appropriate for a particular case.

These review procedures are based on the identified SRP acceptance criteria. For deviations from these acceptance criteria, the staff should review the applicant's evaluation of how the proposed alternatives provide an acceptable method of complying with the relevant NRC requirements identified in Subsection II.

The staff will review Section 8.1 of the safety analysis report (SAR) to ensure that it includes (1) a brief description of the utility grid and its interconnections to other grids and to the nuclear unit, (2) a brief general description of the onsite power system, (3) a brief description of the alternate alternating current (AAC) power source, if provided for station blackout, and the associated interconnections to safety buses, (4) and the design bases, criteria, standards, regulatory guides, and technical positions that will be implemented in the design of the electric power systems, including a description of the extent to which these criteria and guidelines are followed and a positive statement that the design conforms to each.

The staff will perform the review as follows:

1. The staff will establish that the utility grid is adequately described, and that the interconnections between the nuclear unit, the utility grid, and other grids are clearly defined. The descriptions should state whether facilities are existing or planned; if planned, the respective completion dates should be provided.
2. The staff will confirm that the onsite power system is briefly described and that Section 8.3.1 of the SAR presents more detailed information.
3. The staff will confirm that the AAC power source, if provided for station blackout, is briefly described and that Section 8.4 of the SAR provides more detailed information.

4. The staff will confirm that Table 8-1 lists the criteria and guidelines identified as being applicable to the design of electric power systems. The SAR should discuss the applicability of the criteria and guidelines listed and include a statement to the effect that they will be implemented or are implemented in the design of electrical power systems.
5. General Design Criterion (GDC) 17 found in Appendix A to 10 CFR Part 50 contains the requirements for the offsite and onsite electric power systems. Table 8-2 provides the staff interpretation of GDC 17.
6. Appendix 8-A to SRP Chapter 8 provides a typical agenda for a site visit conducted as part of the operating license review.
7. For review of a DC application, the reviewer should follow the above procedures to verify that the design, including requirements and restrictions (e.g., interface requirements and site parameters), set forth in the final safety analysis report (FSAR) meets the acceptance criteria. DCs have referred to the FSAR as the design control document (DCD). The reviewer should also consider the appropriateness of identified COL action items. The reviewer may identify additional COL action items; however, to ensure these COL action items are addressed during a COL application, they should be added to the DC FSAR.

For review of a COL application, the scope of the review is dependent on whether the COL applicant references a DC, an early site permit (ESP) or other NRC approvals (e.g., manufacturing license, site suitability report or topical report).

For review of both DC and COL applications, SRP Section 14.3 should be followed for the review of ITAAC. The review of ITAAC cannot be completed until after the completion of this section.

#### IV. EVALUATION FINDINGS

The reviewer verifies that the applicant has provided sufficient information and that the review and calculations (if applicable) support conclusions of the following type to be included in the staff's safety evaluation report. The reviewer also states the bases for those conclusions.

1. Section 8.1 of the SAR provides (a) a brief description of the utility grid and its interconnections to other grids and the nuclear unit, (b) a brief general description of the onsite alternating current and direct current power system, (c) a brief description of the AAC power source (if provided for station blackout), and (d) the design criteria that have been implemented in the design of the electric power systems.
2. The staff has determined that an electric power system design that conforms to the applicable GDC, regulatory guides, and branch technical positions set forth in Table 8-1 provides a sufficient basis for acceptance of the electric power system.
3. The staff concludes that the design criteria that have been implemented for the electric power system are in accordance with the acceptance criteria listed in Table 8-1 and are acceptable as noted below in the following Sections of Chapter 8.

For DC and COL reviews, the findings will also summarize the staff's evaluation of requirements and restrictions (e.g., interface requirements and site parameters) and COL action items relevant to this SRP section.

In addition, to the extent that the review is not discussed in other SER sections, the findings will summarize the staff's evaluation of the ITAAC, including design acceptance criteria, as applicable.

V. IMPLEMENTATION

The staff will use this SRP section in performing safety evaluations of DC applications and license applications submitted by applicants pursuant to 10 CFR Part 50 or 10 CFR Part 52. Except when the applicant proposes an acceptable alternative method for complying with specified portions of the Commission's regulations, the staff will use the method described herein to evaluate conformance with Commission regulations.

The provisions of this SRP section apply to the review of applications submitted six months or more after the date of issuance of this SRP section, unless superseded by a later revision.

VI. REFERENCES

1. See Table 8-1 for references.

---

**PAPERWORK REDUCTION ACT STATEMENT**

The information collections contained in the Standard Review Plan are covered by the requirements of 10 CFR Part 50 and 10 CFR Part 52, and were approved by the Office of Management and Budget, approval number 3150-0011 and 3150-0151.

**PUBLIC PROTECTION NOTIFICATION**

The NRC may not conduct or sponsor, and a person is not required to respond to, a request for information or an information collection requirement unless the requesting document displays a currently valid OMB control number.

**\_SRP Section 8.1, Rev. 4**  
**“Electric Power - Introduction”**  
**Description of Changes**

Revision 4 to SRP Section 8.1 updates Revision 3 of this Section, dated March 2007, to reflect the following changes:

1. Table 8-1 adds reference to a new BTP 8-8, “Onsite (emergency diesel generators) and offsite power sources allowed outage time extensions”
2. Replaced ‘plant’ with ‘facility’, ‘accordance’ with ‘conformity’, ‘regulations’ with ‘rules and regulations’ on Page 8.1-2; and ‘regulations’ with ‘rules and regulations’ on Page 8.1-3.
3. Modified the title for RG 1.153 in Page 8.1-11.
4. Simultaneously with the issuance of this revision the BTP 8-8 is also issued for use.

**STANDARD REVIEW PLAN**  
**TABLE 8-1**  
**ACCEPTANCE CRITERIA AND GUIDELINES FOR ELECTRIC POWER SYSTEMS**

The matrix of Table 8-1 identifies the acceptance criteria (denoted by “A”) and the guidelines (denoted by “G”) and their applicability to the various Sections of SRP Chapter 8. The acceptance criteria define the Commission’s requirements for power systems important to safety; the guidelines amplify these requirements and provide a more explicit basis upon which to evaluate the conformance of the power systems to these requirements. This table does not include acceptance criteria and guidelines for those aspects of the power systems that are reviewed in accordance with sections other than SRP Chapter 8.

The branch technical positions are listed in the table.

CRITERIA	APPLICABILITY (SAR/SRP Section)					REMARKS <sup>a</sup>
	TITLE	8.2	8.3.1	8.3.2	8.4	
1. General Design Criteria (GDCs), Appendix A to 10 CFR Part 50						
a. GDC 2	Design Bases for Protection Against Natural Phenomena	A	A	A		
b. GDC 4	Environmental and Dynamic Effects Design Bases	A	A	A		
c. GDC 5	Sharing of Structures, Systems, and Components	A	A	A		
d. GDC 17	Electric Power Systems	A	A	A	A	
e. GDC 18	Inspection and Testing of Electric Power Systems	A	A	A	A	

APPLICABILITY (SAR/SRP Section)						
CRITERIA	TITLE	8.2	8.3.1	8.3.2	8.4	REMARKS <sup>a</sup>
f. GDC 33 GDC 34 GDC 35 GDC 38 GDC 41 GDC 44	Reactor Coolant Makeup Residual Heat Removal Emergency Core Cooling Containment Heat Removal Containment Atmosphere Cleanup Cooling Water	A	A	A		As they relate to the operation of electric power systems, encompassed in GDC 17, to ensure that the safety functions of the systems described in GDCs 33, 34, 35, 38, 41, and 44 are accomplished.
g. GDC 50	Containment Design Bases		A	A		
<b>2. Regulations (10 CFR 50 and 10 CFR 52)</b>						
a. 10 CFR 50.34	Contents of Applications; Technical Information					<sup>b</sup> See Table footnote below
i. 50.34(f)(2)(v)	Additional TMI -Related Requirements (Item I.D.3)	A	A	A		<sup>b</sup> See Table Item 5.b and footnote below
ii. 50.34(f)(2)(xiii)	Additional TMI -Related Requirements (I.E.3.1)		A			<sup>b</sup> See Table Item 5.b and footnote below
iii. 50.34(f)(2)(xx)	Additional TMI -Related Requirements (Item II.G.1)		A			<sup>b</sup> See Table Item 5.b and footnote below
b. 10 CFR 50.55a	Codes and Standards		A	A		Paragraph (h) incorporates IEEE Std. 603 and specifies application of IEEE Std. 603 and IEEE Std. 279. See also: RG 1.153.
c. 10 CFR 50.63	Loss of All Alternating Current Power	A	A	A	A	
d. 10 CFR	Requirements for Monitoring the	A	A	A	A	Paragraph (a)(4), as it relates to the



APPLICABILITY (SAR/SRP Section)						
CRITERIA	TITLE	8.2	8.3.1	8.3.2	8.4	REMARKS <sup>a</sup>
50.65(a)(4)	Effectiveness of Maintenance at Nuclear Power Plants					assessment and management of the increase in risk that may result from proposed maintenance activities before performing the maintenance activities.
e. 10 CFR 52.47(b)(1)	Contents of Applications; Technical Information	A	A	A	A	Paragraph (b)(1), as it relates to ITAAC (for design certification) sufficient to assure that the SSCs in this area of review will operate in accordance with the certification.
f. 10 CFR 52.80(a)	Contents of Applications; Additional Technical Information	A	A	A	A	
<b>3. Regulatory Guides (RGs)</b>						
a. RG 1.6	Independence Between Redundant Standby (Onsite) Power Sources and Between Their Distribution Systems		G	G		
b. RG 1.9	Application, and Testing of Safety-Related Diesel Generators in Nuclear Power Plants		G		G	See also: IEEE Std. 387-1984 "IEEE Standard Criteria for Diesel Generator Units Applied as Standby Power Supplies for Nuclear Power Generating Stations, as endorsed by RG 1.9.
c. RG 1.32	Criteria for Power Systems for Nuclear Power Plants	G	G	G		See also: IEEE Std. 308-2001, "IEEE Standard Criteria for Class 1E Power Systems for Nuclear Power Generating Stations," as endorsed by RG 1.32.
d. RG 1.47	Bypassed and Inoperable Status Indication for Nuclear Power Plant		G	G		

APPLICABILITY (SAR/SRP Section)						
CRITERIA	TITLE	8.2	8.3.1	8.3.2	8.4	REMARKS <sup>a</sup>
	Safety Systems					
e. RG 1.53	Application of the Single-Failure Criterion to Nuclear Power Plant Protection Systems		G	G		See also: IEEE Std. 379-2003, "Application of the Single-Failure Criterion to Nuclear Power Generating Station Safety Systems," as endorsed by RG 1.53.
f. RG 1.63	Electric Penetration Assemblies in Containment Structures for Nuclear Power Plants		G	G		See also: IEEE Std. 242-1986, "IEEE Recommended Practice for Protection and Coordination of Industrial and Commercial Power Systems"; IEEE Std. 317-1983 (reaffirmed 1992), "IEEE Standard for Electric Penetration Assemblies in Containment Structures for Nuclear Power Generating Stations"; and Section 5.4 of IEEE Std. 741-1986, "Criteria for the Protection of Class 1E Power Systems and Equipment in Nuclear Power Generating Stations," as endorsed by RG 1.63.
g. RG 1.75	Criteria For Independence of Electrical Safety Systems		G	G		See also: IEEE Std. 384-1992, "IEEE Standard Criteria for Independence of Class 1E Equipment and Circuits," as endorsed by RG 1.75.
h. RG 1.81	Shared Emergency and Shutdown Electric Systems for Multi-Unit Nuclear Power Plants		G	G		

APPLICABILITY (SAR/SRP Section)						
CRITERIA	TITLE	8.2	8.3.1	8.3.2	8.4	REMARKS <sup>a</sup>
i. RG 1.106	Thermal Overload Protection for Electric Motors on Motor-Operated Valves		G	G		
j. RG 1.118	Periodic Testing of Electric Power and Protection Systems		G	G		See also: IEEE Std. 338-1987, "IEEE Standard Criteria for the Periodic Surveillance Testing of Nuclear Power Generating Station Safety Systems," as endorsed by RG 1.118
k. RG 1.128	Installation Design and Installation of Vented Lead-Acid Storage Batteries for Nuclear Power Plants			G		See also: IEEE Std. 484-2002, "IEEE Recommended Practice for Installation Design and Installation of Vented Lead-Acid Storage Batteries for Stationary Application," as endorsed by RG 1.128. In addition, IEEE Std. 485-1997, "IEEE Recommended Practice for Sizing Lead-Acid Batteries for Stationary Applications," provides a method acceptable to the staff for sizing stationary lead acid batteries.
l. RG 1.129	Maintenance, Testing, and Replacement of Vented Lead-Acid Storage Batteries for Nuclear Power Plants			G		See also: IEEE Std. 450-2002, "IEEE Recommended Practice for Maintenance, Testing and Replacement of Vented Lead-Acid Batteries for Stationary Application," as endorsed by RG 1.129.
m. RG 1.153	Criteria for Safety Systems		G	G		See also: IEEE Std. 603, "IEEE Standard Criteria for Safety Systems for Nuclear Power Generating Stations" as incorporated by 10 CFR 50.55a(h) and

APPLICABILITY (SAR/SRP Section)						
CRITERIA	TITLE	8.2	8.3.1	8.3.2	8.4	REMARKS <sup>a</sup>
n. RG 1.155	Station Blackout	G	G	G	G	endorsed by RG 1.153  See also: NUMARC 8700, "Guidelines and Technical Bases for NUMARC Initiatives Addressing Station Blackout at Light Water Reactors," Rev. 0, November 1987, as endorsed by RG 1.155.
o. RG 1.160	Monitoring the Effectiveness of Maintenance at Nuclear Power Plants	G	G	G	G	See also: NUMARC 93-01, "Industry Guideline for Monitoring the Effectiveness of Maintenance at Nuclear Power Plants," Revision 2, April 1996, as endorsed by RG 1.160
p. RG 1.182	Assessing and Managing Risk Before Maintenance Activities at Nuclear Power Plants	G	G	G	G	
q. RG 1.204	Guidelines for Lightning Protection of Nuclear Power Plants	G	G			See also: IEEE Std. 665-1995 (Reaffirmed 2001), "IEEE Standard for Generating Station Grounding," IEEE Std. 666-1991 (Reaffirmed 1996), "Design Guide for Electric Power Service Systems for Generating Stations"; IEEE Std. 1050-1996, "Guide for Instrumentation and Control Equipment Grounding in Generating Stations"; IEEE Std. C62.23-1995 (Reaffirmed 2001), "Application Guide for Surge Protection of Electric Generating Plants," as endorsed by RG 1.204.

APPLICABILITY (SAR/SRP Section)						
CRITERIA	TITLE	8.2	8.3.1	8.3.2	8.4	REMARKS <sup>a</sup>
r. RG 1.206	Combined License Applications for Nuclear Power Plants (LWR Edition)	G	G	G	G	RG 1.206 provides guidance for the review of SARs submitted by COL applicants and contains criteria applicable to all sections of SRP Chapter 8.
<b>4. Branch Technical Positions</b>						
a. BTP 8-1	Requirements on Motor-Operated Valves in the ECCS Accumulator Lines		G			
b. BTP 8-2	Use of Diesel-Generator Sets for Peaking		G			
c. BTP 8-3	Stability of Offsite Power Systems	G				
d. BTP 8-4	Application of the Single Failure Criterion to Manually-Controlled Electrically-Operated Valves		G			
e. BTP 8-5	Supplemental Guidance for Bypass and Inoperable Status Indication for Engineered Safety Features Systems		G	G		
f. BTP 8-6	Adequacy of Station Electric Distribution System Voltages	G	G			
g. BTP 8-7	Criteria for Alarms and Indications		G			

APPLICABILITY (SAR/SRP Section)						
CRITERIA	TITLE	8.2	8.3.1	8.3.2	8.4	REMARKS <sup>a</sup>
h. BTP 8-8	Associated with Diesel-Generator Unit Bypassed and Inoperable Status Onsite (emergency diesel generators) and offsite power sources allowed outage time extensions	G	G			
<b>5. NUREG Reports</b>						
a. NUREG-0718, Revision 1	Licensing Requirements for Pending Applications for Construction Permits and Manufacturing License		G	G		See TMI Item I.D.3, "Safety System Status Monitoring," regarding application of RG 1.47.
b. NUREG-0737	Clarification of TMI Action Plan Requirements		A			See TMI Items II.E.3.1, "Emergency Power Supply for Pressurizer Heaters," and II.G.1, "Emergency Power for Pressurizer Equipment."
c. NUREG/CR-0660	Enhancement of Onsite Diesel Generator Reliability		G			Reference Only
d. NUREG-1793	Final Safety Evaluation Report Related to Certification of the AP1000 Standard Design	G	G	G	G	NUREG-1793 provides the staff's safety review of the AP1000 standard design against the requirements of 10 CFR Part 52, Subpart B and describes the basis for acceptance of passive features and systems not found in current operating reactors.
<b>6. Commission Papers (SECY)</b>						

APPLICABILITY (SAR/SRP Section)						
CRITERIA	TITLE	8.2	8.3.1	8.3.2	8.4	REMARKS <sup>a</sup>
a. SECY-90-016	Evolutionary Light Water Reactor Certification Issues and Their Relationships to Current Regulatory Requirements, June, 1990	A	A		A	As it relates to the use of AAC power sources and application of RTNSS at ALWRs provided with passive safety systems.
b. SECY-94-084	Policy and Technical Issues Associated with the Regulatory Treatment of Non-Safety Systems in Passive Plant Designs, 1994	A	A		A	As it relates to the use of AAC power sources and application of RTNSS at ALWRs provided with passive safety systems.
c. SECY-95-132	Policy and Technical Issues Associated with the Regulatory Treatment of Non-Safety Systems (RTNSS) in Passive Plant Designs, 1995	A	A		A	As it relates to the use of AAC power sources and application of RTNSS at ALWRs provided with passive safety systems.
d. SECY-91-078	Electronic Power Research Institute Requirements Document and Additional Evolutionary Light Water Reactor (LWR) Certification Issues, 1991	A				As it relates to the inclusion of an alternate power source to non-safety-related loads at evolutionary plant designs.
e. SECY-05-0227	Final Rule – AP1000 Design Certification, 2005	A				As it relates to an exemption to the GDC 17 requirement for two physically independent offsite circuits at passive reactor designs.

<sup>a</sup> Related industry standards and guidelines are included for reference only. Refer to the specific SRP section for applicability. The staff will review new applications using the latest version of industry codes and standards endorsed by the NRC. Proposed use of unendorsed versions of codes and standards will be reviewed on a case-by-case basis.

<sup>b</sup> These criteria only apply for applicants subject to 10 CFR 50.34(f) requirements. For Part 50 applicants not listed in 10 CFR 50.34(b), the provisions of 50.34(b) will be made a requirement in the licensing process.



**TABLE 8-2  
NRC STAFF INTERPRETATION OF THE REQUIREMENTS OF GDC 17**

This criterion provides a deterministic delineation of the minimum requirements for the offsite and onsite electric power systems (i.e., in terms of specific system and circuit configuration and functional requirements). The only requirement in GDC 17 for explicitly meeting the single failure criterion relates to the onsite power system. In applying GDC 17, the staff has found it useful to also interpret the deterministic requirements for the offsite power system in terms of required conformance to the single failure criterion. The text of GDC 17, presented in the left column of Table 8-2, is keyed to the staff interpretation of its deterministic requirements and the corresponding staff interpretations regarding conformance to the single failure criterion.

GDC 17	Staff Interpretation
<p>Criterion 17—Electric power systems. An onsite electric power system and an offsite electric power system shall be provided to permit functioning of structures, systems, and components important to safety. The safety function for each system (assuming the other system is not functioning) shall be to provide sufficient capacity and capability to assure that (1) specified acceptable fuel design limits and design conditions of the reactor coolant pressure boundary are not exceeded as a result of anticipated operational occurrences and (2) the core is cooled and containment integrity and other vital functions are maintained in the event of postulated accidents.</p>	<p>a. Both an offsite and onsite power system shall be provided, each independent of the other and capable of providing power for all safety functions. (The offsite and onsite power systems considered together must meet the single failure criterion on a system basis without losing the capability to provide power for <u>all</u> safety functions. In addition, in view of requirement (b) below, the two systems considered together must be capable of sustaining a complete loss of offsite power (LOOP) and a single failure in onsite system, without losing the capability to provide power for the <u>minimum required</u> safety functions.)</p>
<p>The onsite electric power supplies, including the batteries, and the onsite electric distribution system, shall have sufficient independence, redundancy, and testability to perform their safety functions assuming a single failure.</p>	<p>b. The complete onsite electric power system (Class 1E) must be capable of sustaining a single failure without loss of capability to provide power for the <u>minimum required</u> safety function.</p>
<p>Electric power from the transmission network to the onsite electric distribution system shall be supplied by two physically independent circuits (not necessarily on separate rights of way) designed and located so as to minimize to the extent practical the likelihood of their simultaneous failure under operating and postulated accident and environmental conditions. A switchyard</p>	<p>c. The offsite system shall be comprised of two physically independent circuits connecting the transmission network (grid) to the onsite distribution system (safety buses). (Separate transmission line towers are required but common switchyard structures are acceptable. No requirement for meeting the single failure criterion, and in the absolute sense, this criterion cannot be met because there is only one power source, the grid.)</p>

GDC 17	Staff Interpretation
common to both circuits is acceptable.	
<p>Each of these circuits shall be designed to be available in sufficient time following a loss of all onsite alternating current power supplies and the other offsite electric power circuit, to assure that specified acceptable fuel design limits and design conditions of the reactor coolant pressure boundary are not exceeded.</p>	<p>d. Each of the two required offsite power circuits shall be designed to be available in sufficient time to effect safe shutdown in the event of a loss of all onsite power and the loss of the other offsite circuit. (The staff has designated the second circuit as the “delayed access circuit.” The offsite power system (i.e., the two circuits considered together, must meet the single failure criterion, but only with respect to the delayed access circuit function.))</p>
<p>One of these circuits shall be designed to be available within a few seconds following a loss-of-coolant accident to assure that core cooling, containment integrity, and other vital safety functions are maintained.</p>	<p>e. One of these circuits shall be designed to be available within a few seconds following a loss-of-coolant accident. (The staff has designated this circuit as the “immediate access circuit.” Because only one such circuit is required, the offsite power system need not meet the single failure criterion with respect to its immediate access function.)</p>
<p>Provisions shall be included to minimize the probability of losing electric power from any of the remaining supplies as a result of, or coincident with, the loss of power generated by the nuclear power unit, the loss of power from the transmission network, or the loss of power from the onsite electric power supplies.</p>	<p>f. Analyses (performed by the utility) must verify that the grid remains stable in the event of a loss of the nuclear unit generator, the largest other unit on the grid, or the most critical transmission line. (There is no specific requirement for meeting the single failure criterion. However, overlapping requirement (a) above requires the offsite/onsite power systems to meet this criterion on a system basis.)</p>