Duke Energy®

T. PRESTON GILLESPIE, JR.
Vice President
Oconee Nuclear Station

Duke Energy
ON01VP / 7800 Rochester Hwy.
Seneca, SC 29672

864-873-4478
864-873-4208 fax
T.Gillespie@duke-energy.com

April 22, 2011

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287
Licensee Event Report 269/2011-03, Revision 0
Problem Investigation Process No.: O-10-03882

Gentlemen:

Pursuant to 10 CFR 50.73 Sections (a)(1) and (d), please find attached Licensee Event Report (LER) 269/2011-03, Revision 0, regarding the inoperability of the Standby Shutdown Facility (SSF) due to a design oversight resulted in a condition which exceeded the 7-days allowed by Technical Specification (TS) 3.10.1 (D). Consequently, this report is being submitted in accordance with 10 CFR 50.73(a)(2)(i)(b), as an operation prohibited by the station's TSs. This event is considered to have no consequence with respect to the health and safety of the public.

Other than the commitment to submit a LER supplement, there are no regulatory commitments contained in this report. Duke Energy Carolinas, LLC, expects to provide the supplement within 60 days, but does not consider that time frame to be a commitment.

Any questions regarding the content of this report should be directed to Stephen C. Newman, Regulatory Compliance Group, Oconee Nuclear Station, at (864) 873-4388.

Sincerely,

T P Gillespie

T. Preston Gillespie, Jr.
Vice President
Oconee Nuclear Station

Attachment

JE22
NRR

cc: Mr. Victor McCree
Administrator, Region II
U.S. Nuclear Regulatory Commission
Marquis One Tower
245 Peachtree Center Ave., NE, Suite 1200
Atlanta, GA 30303-1257

Mr. John Stang
Project Manager
U.S. Nuclear Regulatory Commission
Office of Nuclear Reactor Regulation
Washington, D.C. 20555

Mr. Andrew Sabisch
NRC Senior Resident Inspector
Oconee Nuclear Station

INPO (Word File via E-mail)

NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION
(10-2010)

# LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)



| 1. FACILITY NAME | 2. DOCKET NUMBER | 3. PAGE |
|---|---|---|
| Oconee Nuclear Station, Unit 1 | 05000-0269 | 1 of 1 |

**4. TITLE**
Inoperability of the Standby Shutdown Facility, due to a design oversight, for a period that exceeded that allowed by the Technical Specifications

| 5. EVENT DATE | | | 6. LER NUMBER | | | 7. REPORT DATE | | | 8. OTHER FACILITIES INVOLVED | |
|---|---|---|---|---|---|---|---|---|---|---|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REV NO | MONTH | DAY | YEAR | FACILITY NAME | DOCKET NUMBER |
| | | | | | | | | | Unit 2 | 05000-0270 |
| 02 | 24 | 2011 | 2011 | – 03 | – 00 | 04 | 22 | 2011 | Unit 3 | 05000-0287 |

**9. OPERATING MODE**
Unit 1 - 1
Unit 2 - 1
Unit 3 - 1

**10. POWER LEVEL**
Unit 1 - 100%
Unit 2 – 100%
Unit 3 - 100%

**11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §:** *(Check all that apply)*

| | | | |
|---|---|---|---|
| ☐ 20.2201(b) | ☐ 20.2203(a)(3)(i) | ☐ 50.73(a)(2)(i)(C) | ☐ 50.73(a)(2)(vii) |
| ☐ 20.2201(d) | ☐ 20.2203(a)(3)(ii) | ☐ 50.73(a)(2)(ii)(A) | ☐ 50.73(a)(2)(viii)(A) |
| ☐ 20.2203(a)(1) | ☐ 20.2203(a)(4) | ☐ 50.73(a)(2)(ii)(B) | ☐ 50.73(a)(2)(viii)(B) |
| ☐ 20.2203(a)(2)(i) | ☐ 50.36(c)(1)(i)(A) | ☐ 50.73(a)(2)(iii) | ☐ 50.73(a)(2)(ix)(A) |
| ☐ 20.2203(a)(2)(ii) | ☐ 50.36(c)(1)(ii)(A) | ☐ 50.73(a)(2)(iv)(A) | ☐ 50.73(a)(2)(x) |
| ☐ 20.2203(a)(2)(iii) | ☐ 50.36(c)(2) | ☐ 50.73(a)(2)(v)(A) | ☐ 73.71(a)(4) |
| ☐ 20.2203(a)(2)(iv) | ☐ 50.46(a)(3)(ii) | ☐ 50.73(a)(2)(v)(B) | ☐ 73.71(a)(5) |
| ☐ 20.2203(a)(2)(v) | ☐ 50.73(a)(2)(i)(A) | ☐ 50.73(a)(2)(v)(C) | ☐ OTHER |
| ☐ 20.2203(a)(2)(vi) | ☒ 50.73(a)(2)(i)(B) | ☐ 50.73(a)(2)(v)(D) | Specify in Abstract below or in NRC Form 366A |

**12. LICENSEE CONTACT FOR THIS LER**

| NAME | TELEPHONE NUMBER (Include Area Code) |
|---|---|
| S. C. Newman, ONS Regulatory Compliance Group | (864) 873-4388 |

**13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT**

| CAUSE | SYSTEM | COMPONENT | MANU-FACTURER | REPORTABLE TO EPIX | CAUSE | SYSTEM | COMPONENT | MANU-FACTURER | REPORTABLE TO EPIX |
|---|---|---|---|---|---|---|---|---|---|
| | | | | | | | | | |

**14. SUPPLEMENTAL REPORT EXPECTED**
☒ YES (If yes, complete EXPECTED SUBMISSION DATE) ☐ NO

**15. EXPECTED SUBMISSION DATE**

| MONTH | DAY | YEAR |
|---|---|---|
| 06 | 30 | 2011 |

**16. ABSTRACT** (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On May 13, 2010, at approximately 2245 hours, an unanticipated Standby Shutdown Facility (SSF) differential lockout relay (86D) actuation occurred while performing an emergency power switching logic function test. The 86D lockout relay is actuated by various safety and non-safety trip signals. The non-safety trip signals are disabled during emergency operation of the SSF diesel generator (D/G). One of the 86D non-safety input signals is for high bearing temperature. The safety function of the SSF is to mitigate certain design basis events for 1, 2, or all 3 units.

The preliminary root cause was attributed to a design oversight involving the Fall 2008 replacement of two (2) chart recorders whose output contacts provided the high bearing temperature trip signal to the SSF D/G. The design change package failed to consider the recorder's 25-60 second boot-up delay. During the test, the chart recorders were load shed and repowered (as expected) but, due to the boot-up delay, had defaulted to a tripped state that led to the lockout relay actuation. Once the lockout relay is actuated, the lockout relay must be manually reset in order for the SSF diesel to start in any mode.

Since the period of SSF inoperability exceeded the 7-days allowed by Technical Specification (TS) 3.10.1 (D), the event is reportable as a condition prohibited by the TSs. The high bearing temperature trip function of the chart recorder has been eliminated. Additional corrective actions are pending following the completion of an ongoing root cause evaluation. The root cause(s) and applicable corrective actions from this evaluation will be submitted in a supplement to this report. This event is considered to have no consequence with respect to the health and safety of the public.