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ACCESSION NBR: 8701280402 DOC. DATE: 87/01/23 NOTARIZED: NO DOCKET # FACIL: 50-438 Bellefonte Nuclear Plant, Unit 1, Tennessee Valley Au 05000438 50-439 Bellefonte Nuclear Plant, Unit 2, Tennessee Valley Au 05000439 AUTHOR AFFILIATION AUTH. NAME Tennessee Valley Authority GRIDLEY, R. RECIPIENT AFFILIATION RECIP. NAME Record Services Branch (Document Control Desk) STOLZ, J. F.

SUBJECT: Forwards BAW-1979 "Bmall Break Loss-of-Coolant Accident DEE SUBJ F.16 For BAW- 1879 Analysis for B&W 205 FA Plants Response to NUREG-0737, Section II.K. 3. 31" & meets intent of Generic Ltr 83-35.

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NOTES: OIA 1cy. Application for permit renewal filed. 05000438 DIA 1cy. Application for permit renewal filed. 05000439

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## TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

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JAN 23 1986

U.S. Nuclear Regulatory Commission Attn: Document Control Desk Office of Nuclear Reactor Regulation Washington, D.C. 20555

Attention: Mr. J. F. Stolz

In the Matter of the Application of) Tennessee Valley Authority ) Docket Nos. 50-438 50-439

BELLEFONTE NUCLEAR PLANT (BLN) UNITS 1 AND 2 - NUREG-0737, ITEM II.K.3.31 - PLANT-SPECIFIC CALCULATIONS TO SHOW COMPLIANCE WITH 10 CFR 50.46

NUREG-0737, Item II.K.3.31 requires plant-specific calculations (using NRC's approved upgraded model from Item II.K.3.30) to be performed to show compliance with 10 CFR 50.46. In responding to this item for BLN, TVA worked with Babcock & Wilcox (B&W) to provide the BLN-specific calculations to satisfy Item II.K.3.31. Enclosed is B&W report No. BAW-1979, "Small Break Loss-of-Coolant Accident Analysis for B&W FA Plants Response to NUREG-0737, Section II.K.3.31," which documents the results of the BLN-specific analyses.

B&W report No. BAW-1979 does not contain the full spectrum of analyses originally required by Item II.K.3.31. The original requirements of Item II.K.3.31 were modified by NRC Generic Letter 83-35. This generic letter reduced the amount of analyses necessary to address Item II.K.3.31. The analyses discussed in B&W report No. BAW-1979: (1) meet the intent of Generic Letter 83-35, (2) demonstrate conformance to the acceptance criteria of 10 CFR 50.46, and (3) demonstrate that the small break loss-of-coolant accident analyses in the Final Safety Analysis Report using the previously approved evaluation model are more limiting than analyses using NRC's approved upgraded model.

If you have any questions concerning this matter, please get in touch with D. L. Terrill at (205) 574-8820.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

Home

R. Gridley, Director Nuclear Safety and Licensing

Enclosure cc: See page 2

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