

MAR 28 1974

Docket Nos. 50-438
and 50-439

Tennessee Valley Authority
ATTN: Mr. James E. Watson
Manager of Power
818 Power Building
Chattanooga, Tennessee 37401

Gentlemen:

In order that we may continue our review of your application for a license to construct the Bellefonte Nuclear Plant, Units 1 and 2, additional information is required. The information requested is described in the enclosure and pertains to the Preliminary Safety Analysis Report (PSAR).

In order to maintain our licensing review schedule, we will need a completely adequate response to all enclosed requests by April 15, 1974. Please inform us within 7 days after receipt of this letter of your confirmation of the schedule date or the date you will be able to meet. If you cannot meet our specified date or if your reply is not fully responsive to our request, it is highly likely that the overall schedule for completing the licensing review for the project will have to be extended. Since reassignment of the staff's efforts will require completion of the new assignment prior to returning to this project, the extension will most likely be greater than the delay in your response.

Please contact us if you have any questions regarding the information requested.

Sincerely,

Original Signed by

A. Schwencer, Chief
Light Water Reactors Branch 2-3
Directorate of Licensing

Enclosure:
Request for Additional Information

ccs: See next page

LB

OFFICE						
SURNAME						
DATE						

ccs: Mr. R. H. Marquis
General Counsel
629 New Sprakle Building
Knoxville, Tennessee 37902

William E. Garner, Esquire
Route 4
Scottsboro, Alabama 35768

Mr. Lyle A. Taylor
3301 Helena, N.W.
Huntsville, Alabama 35810

bcc: E. G. Beasley, Jr.
307 U.B.A.
Tennessee Valley Authority
Knoxville, Tennessee 37902

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OFFICE ▶	x7886/LWR 2-3					
SURNAME ▶	DDavis:cjb					
DATE ▶	3/ /74					

REQUEST FOR ADDITIONAL INFORMATION
TENNESSEE VALLEY AUTHORITY
BELLEFONTE NUCLEAR PLANT, UNITS 1 & 2
DOCKET NOS. 50-438 AND 50-439

6.0 ENGINEERED SAFETY FEATURES

- 6.96 As defined in the PSAR, the annulus volume is processed through a recirculation system.
- a. Provide the fraction of the total containment leakage that is assumed to be treated by the recirculation system.
 - b. Provide the fraction that is assumed to be treated by the Auxiliary Building filter system.
 - c. Provide your proposed Technical Specification that define these allowable leakage fractions.
- 6.97 After a postulated design basis accident, define the areas, in addition to the annulus, that will be maintained at a minimum negative differential pressure of at least .25 inches water gauge using ESF filtration systems.
- 6.98 Provide the length of time required to bring the enclosure building region (annulus and additional ESF filtered areas) to a minimum negative differential pressure of .25 inches water gauge using ESF filter systems after a postulated LOCA.
- 6.99 Provide your Technical Specification that will define the allowable bypass leakage through guard pipes on the main steam lines. If the test to define this leakage is for zero leakage, the sensitivity of the test should be treated as bypass leakage in your proposed Technical Specifications and accident analysis.
- 6.100 Provide your proposed Technical Specification that will define the allowable in-leakage for the secondary containment structure.