

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

March 31, 2011

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555-0001

Serial No. 11-072
SPS Lic JSA
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION UNITS 1 AND 2
ANNUAL CHANGES, TESTS, AND EXPERIMENTS REPORT
REGULATORY COMMITMENT EVALUATION REPORT

Virginia Electric and Power Company submits the annual report of Changes, Tests, and Experiments pursuant to 10 CFR 50.59(d)(2) and Regulatory Commitment Changes identified in Commitment Evaluation Summaries implemented at Surry Power Station during 2010. Attachment 1 provides a description and summary of the Regulatory Evaluations and Regulatory Commitment Changes.

Should you have any questions regarding this report, please do not hesitate to contact Barry Garber at (757) 365-2725.

Very truly yours,



B. L. Stanley,
Director Nuclear Station Safety and Licensing
Surry Power Station

Attachment

Commitments made in this letter: None.

cc: United States Nuclear Regulatory Commission, Region II
Marquis One Tower, Suite 1200
245 Peachtree Center Avenue, NE
Atlanta, Georgia 30303-1257

NRC Senior Resident Inspector
Surry Power Station

JE47
NRR

Attachment 1

Surry Units 1 & 2

2010 – 10 CFR 50.59 Changes, Tests and Experiments

10-001 Regulatory Evaluation

04/22/10

Description: Regulatory Evaluation 10-001 documents increasing the Surry Units 1 and 2 Technical Specifications' cumulative core burnup applicability limit to 48 Effective Full Power Years (EFPY).

Summary: This evaluation involves increasing the Surry Units 1 and 2 Technical Specifications' (TS) cumulative core burnup applicability limit for RCS Pressure/Temperature Limits (P/T), Low Temperature Overpressure Protection System (LTOPS) Setpoints and LTOPS Enabling Temperature (T-Enable) to 48 EFPY. The scope of this evaluation applies to the TS Bases, UFSAR and includes changes to applicable site procedures.

10-002 Regulatory Evaluation

05/12/10

Description: Regulatory Evaluation 10-002 documents the deferral of Turbine Valve Freedom Tests until the Fall 2010 refueling outage.

Summary: This evaluation documents deferral of Operations Surveillance Procedure for Turbine Inlet Valve Freedom Tests for #4 Governor Valve and its associated Stop Valve until the Fall 2010 Unit 1 refueling outage.

10-003 Regulatory Evaluation

10/21/10

Description: Regulatory Evaluation 10-003 documents the change to the UFSAR which includes changes to the Locked Rotor Accident (LRA) dose consequences analysis that resolves issues with the use of a non-conservative SG PORV capacity and incorporates a new EAB X/Q, daughter product activities and new LOCADOSE code version. The revised LRA models the end of releases to the environment at 9 hours instead of 8 hours.

Summary: The change in the LRA dose consequences analysis is primarily related to the post accident cooldown capability of the SG PORV. No actual changes are being made to the reactor coolant pump or the SG PORVs or any procedures. The increased cooldown release duration and the corresponding increase in dose consequences for the LRA does not create a malfunction with a different result than previously evaluated in the FSAR. In addition, the changes in LRA dose analysis were determined not to require NRC review and approval because the changes resulted in a less than minimal increase in consequences, and the changes in methodology were either previously approved by the NRC, conservative, or the results were essentially the same.

Attachment 1

Surry Units 1 & 2

2010 – 10 CFR 50.59 Changes, Tests and Experiments

Note: A review determined the following 2008 Commitment Evaluation was not previously sent.

Commitment Evaluation Summary

07/22/08

Description: This Commitment Evaluation revised minor details in the strategies that were committed to in letter 07-0004D, Supplemental Response Providing Information Regarding Implementation Details for the Phase 2 and 3 B.5.b Mitigation Strategies.

Summary: Changes in hose sizes and lengths were implemented to ensure margin for B.5.b Mitigation Strategies.

Commitment Evaluation Summary

09/14/10

Description: This Commitment Evaluation revised the method and location of the radiation surveys performed to monitor residual radiation levels on the inside of the Steam Generator Storage Facility (SGSF).

Summary: Residual radiation levels were monitored semi-annually through radiation survey ports located on the roof of the SGSF. The change implements a semi-annual radiation survey at ground level on contact with the exterior of the SGSF utilizing a Micro-R/Micro-Sv Survey Meter.