

ArevaEPRDCPEm Resource

From: WELLS Russell (AREVA) [Russell.Wells@areva.com]
Sent: Thursday, April 07, 2011 7:18 AM
To: Tesfaye, Getachew
Cc: GUCWA Len (EXTERNAL AREVA); BENNETT Kathy (AREVA); DELANO Karen (AREVA); ROMINE Judy (AREVA); RYAN Tom (AREVA)
Subject: Response to U.S. EPR Design Certification Application RAI No. 475 (5558, 5557), FSAR Ch. 6
Attachments: RAI 475 Response US EPR DC.PDF

Getachew,

Attached please find AREVA NP Inc.'s response to the subject request for additional information (RAI). The attached file, "RAI 475 Response US EPR DC.pdf" provides a schedule since technically correct and complete responses to the 3 questions are not provided.

The following table indicates the respective pages in the response document, "RAI 475 Response US EPR DC.pdf," that contain AREVA NP's responses to the subject questions.

Question #	Start Page	End Page
RAI 475 — 06.02.01.01.A-1	2	2
RAI 475 — 06.02.02-86	3	3
RAI 475 — 06.02.02-87	4	4

A complete answer is not provided for 3 of the 3 questions. The schedule for technically correct and complete responses to these questions is provided below.

Question #	Response Date
RAI 475 — 06.02.01.01.A-1	June 29, 2011
RAI 475 — 06.02.02-86	June 29, 2011
RAI 475 — 06.02.02-87	June 29, 2011

Sincerely,

Russ Wells

U.S. EPR Design Certification Licensing Manager

AREVA NP, Inc.

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Russell.Wells@Areva.com

From: Tesfaye, Getachew [<mailto:Getachew.Tesfaye@nrc.gov>]

Sent: Monday, March 07, 2011 8:07 PM

To: ZZ-DL-A-USEPR-DL

Cc: Jensen, Walton; Ashley, Clinton; Jackson, Christopher; McKirgan, John; Carneal, Jason; Colaccino, Joseph; ArevaEPRDCPEm Resource

Subject: U.S. EPR Design Certification Application RAI No. 475 (5558, 5557), FSAR Ch. 6

Attached please find the subject request for additional information (RAI). A draft of the RAI was provided to you on February 22, 2011, and on March 4, 2011, you informed us that the RAI is clear and no further clarification is needed. As a result, no change is made to the draft RAI. The schedule we have established for review of your application assumes technically correct and complete responses within 30 days of receipt of RAIs. For any RAIs that cannot be answered within 30 days, it is expected that a date for receipt of this information will be provided to the staff within the 30 day period so that the staff can assess how this information will impact the published schedule.

Thanks,
Getachew Tesfaye
Sr. Project Manager
NRO/DNRL/NARP
(301) 415-3361

Hearing Identifier: AREVA_EPR_DC_RAIs
Email Number: 2816

Mail Envelope Properties (1F1CC1BBDC66B842A46CAC03D6B1CD410433031A)

Subject: Response to U.S. EPR Design Certification Application RAI No. 475 (5558, 5557), FSAR Ch. 6
Sent Date: 4/7/2011 7:18:27 AM
Received Date: 4/7/2011 7:18:46 AM
From: WELLS Russell (AREVA)

Created By: Russell.Wells@areva.com

Recipients:

"GUCWA Len (EXTERNAL AREVA)" <Len.Gucwa.ext@areva.com>
Tracking Status: None
"BENNETT Kathy (AREVA)" <Kathy.Bennett@areva.com>
Tracking Status: None
"DELANO Karen (AREVA)" <Karen.Delano@areva.com>
Tracking Status: None
"ROMINE Judy (AREVA)" <Judy.Romine@areva.com>
Tracking Status: None
"RYAN Tom (AREVA)" <Tom.Ryan@areva.com>
Tracking Status: None
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Tracking Status: None

Post Office: AUSLYNCMX02.adom.ad.corp

Files	Size	Date & Time
MESSAGE	2471	4/7/2011 7:18:46 AM
RAI 475 Response US EPR DC.PDF		12060

Options

Priority: Standard

Return Notification: No

Reply Requested: No

Sensitivity: Normal

Expiration Date:

Recipients Received:

Response to

Request for Additional Information No. 475 (5558), Revision 0

3/07/2011

U. S. EPR Standard Design Certification

AREVA NP Inc.

Docket No. 52-020

**SRP Section: 06.02.01.01.A - PWR Dry Containments, Including Subatmospheric
Containments**

SRP Section: 06.02.02 - Containment Heat Removal Systems

Application Section: 6.2

**QUESTIONS for Containment and Ventilation Branch 1 (AP1000/EPR Projects)
(SPCV)**

Question 06.02.01.01.A-1:

During an audit held at AREVA's office in Lynchburg, VA from Feb 15-16, 2011 the staff learned that the reactor cavity is isolated from the rest of the containment by design features so that steam from the break cannot reach it. If the reactor cavity volume is isolated from the rest of the containment, provide justification for including this volume in the 30 node GOTHIC model to evaluate peak containment pressure. If this volume should not be included, provide evaluations of the increase in containment pressure and temperature that would occur for design basis LOCA and MSLB events.

Response to Question 06.02.01.01.A-1:

A response to this question will be provided by June 29, 2011.

Question 06.02.02-86:

During an audit of the IRWST level calculation it was discovered that some containment areas that cannot readily return water to the IRWST (sump) were not included in the IRWST water level calculation. For example, the core spreading area and the reactor cavity were not included as areas for water hold-up. Based on the containment layout review the core spreading area connects to the containment atmosphere and can collect steam/water during an accident and this water would not be able to return to the IRWST (sump) and support NPSH. Additionally, the reactor cavity area was not evaluated to hold-up water because AREVA indicated that 'design features' exist which prevent water/steam from reaching the reactor cavity during a postulated break. A description of these design features was not provided. The NRC staff request that AREVA evaluate water hold-up from the IRWST that considers the core spreading area. In addition, the staff also requests that AREVA evaluate water hold-up from the IRWST that includes the reactor cavity area; or provide a description of the design features and justification that the design features prevent water from reaching the reactor cavity.

Response to Question 06.02.02-86:

A response to this question will be provided by June 29, 2011.

Question 06.02.02-87:

Provide justification that the break selection used in the IRWST water retention (level) calculations is the most limiting or worst case. Please explain why other breaks, such as the largest pipe at the top of the pressurizer or pressurizer surge line, will not retain more water.

Response to Question 06.02.02-87:

A response to this question will be provided by June 29, 2011.