



**INDIANA
MICHIGAN
POWER®**

A unit of American Electric Power

Indiana Michigan Power
One Cook Place
Bridgman, MI 49106
IndianaMichiganPower.com

March 25, 2011

AEP-NRC-2011-25
10 CFR 50.90

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

SUBJECT: Change to Proposed Technical Specification Page Regarding Large-Break Loss-Of-Coolant Accident Analysis Approval (TAC No. ME1017)

- References:**
- 1) Letter from L. J. Weber, Indiana Michigan Power Company (I&M), to U.S. Nuclear Regulatory Commission (NRC) Document Control Desk, "Donald C. Cook Nuclear Plant Unit 2, Docket No. 50-316, License Amendment Request Regarding Large Break Loss-of-Coolant Accident Analysis Methodology," AEP-NRC-2009-23, dated March 19, 2009 (ADAMS Accession Number ML090930453).
 - 2) Letter from L. J. Weber, I&M, to NRC Document Control Desk, "Donald C. Cook Nuclear Plant Unit 2, Docket No. 50-316, Response to Request for Additional Information Regarding a License Amendment Request Associated With the Large-Break Loss-Of-Coolant Accident Analysis Methodology (TAC No. ME1017)," AEP-NRC-2009-71, dated November 20, 2009 (ADAMS Accession Number ML093360524).

Dear Sir or Madam:

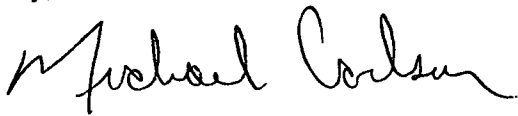
By Reference 1, Indiana Michigan Power Company (I&M) proposed to amend Appendix A, Technical Specifications (TS), to the Donald C. Cook Nuclear Plant (CNP) Unit 2 Facility Operating License, DPR-74. I&M requested NRC approval to adopt a new analysis of a postulated large break loss-of-coolant accident (LBLOCA) for CNP Unit 2, and proposed to modify the associated TSs. By Reference 2, I&M provided a response to an NRC Request for Additional Information (RAI) regarding the proposed amendment. Enclosure 5 of Reference 2 provided a revision of the mark-up of TS Page 5.6-3 originally transmitted by Reference 1. This letter transmits a mark-up of TS Page 5.6-3 with an additional change such that it refers to the expected amendment number and provides a location for the amendment date. Enclosure 1 to this letter provides an affirmation statement. Enclosure 2 provides the mark-up of TS Page 5.6-3.

A002
NRC

Copies of this letter and its enclosures are being transmitted to the Michigan Public Service Commission and the Michigan Department of Natural Resources and Environment, in accordance with the requirements of 10 CFR 50.91. This letter contains no new or modified NRC commitments.

Should you have any questions, please contact Mr. Michael K. Scarpello, Regulatory Affairs Manager, at (269) 466-2649.

Sincerely,



Michael H. Carlson
Site Support Services Vice President

JRW/jmr

Enclosures:


1. Affirmation
2. Mark-up of Donald C. Cook Nuclear Plant Unit 2 Technical Specification Page 5.6-3

c: J. T. King, MPSC
S. M. Krawec, Ft. Wayne AEP, w/o enclosures
MDNRE – WHMD/RPS
NRC Resident Inspector
M. A. Satorius, NRC Region III
P. S. Tam, NRC, Washington DC

AFFIRMATION

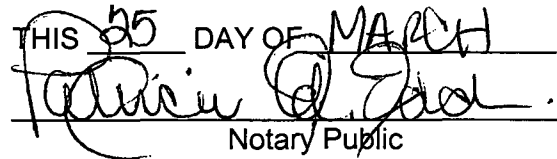
I, Michael H. Carlson, being duly sworn, state that I am Site Support Services Vice President of Indiana Michigan Power Company (I&M), that I am authorized to sign and file this request with the Nuclear Regulatory Commission on behalf of I&M, and that the statements made and the matters set forth herein pertaining to I&M are true and correct to the best of my knowledge, information, and belief.

Indiana Michigan Power Company



Michael H. Carlson
Site Support Services Vice President

SWORN TO AND SUBSCRIBED BEFORE ME

THIS 25 DAY OF MARCH, 2011


Notary Public

My Commission Expires 11-5-2011

PATRICIA ANN EDDIE
Notary Public, State of Michigan
County of Berrien
My Commission Expires Nov. 5, 2011
Acting in the County of berrien

Enclosure 2 to AEP-NRC-2011-25

Mark-up of Donald C. Cook Nuclear Plant Unit 2 Technical Specification Page 5.6-3

5.6 Reporting Requirements

5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

5. LCO 3.1.6, "Control Bank Insertion Limits";
 6. LCO 3.2.1, "Heat Flux Hot Channel Factor ($F_Q(Z)$)";
 7. LCO 3.2.2, "Nuclear Enthalpy Rise Hot Channel Factor ($F_{\Delta H}^N$)";
 8. LCO 3.2.3, "AXIAL FLUX DIFFERENCE (AFD)";
 9. LCO 3.3.1, "Reactor Trip System (RTS) Instrumentation," Functions 6 and 7 (Overtemperature ΔT and Overpower ΔT , respectively) Allowable Value parameter values;
 10. LCO 3.4.1, "RCS Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits"; and
 11. LCO 3.9.1, "Boron Concentration."
- b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:
1. WCAP-9272-P-A, "Westinghouse Reload Safety Evaluation Methodology," (Westinghouse Proprietary);
 2. WCAP-8385, "Power Distribution Control and Load Following Procedures - Topical Report," (Westinghouse Proprietary);
 3. WCAP-10216-P-A, "Relaxation of Constant Axial Offset Control/ F_Q Surveillance Technical Specification," (Westinghouse Proprietary);
 4. ~~WCAP-10266-P-A, "The 1981 Version of Westinghouse Evaluation Mode Using BASH Code," (Westinghouse Proprietary);~~ Plant-specific adaptation (approved by Amendment 297, dated XXXXX XX, 2011) of WCAP-16009-P-A, "Realistic Large-Break LOCA Evaluation Methodology Using the Automated Statistical Treatment of Uncertainty Method (ASTRUM)," Revision 0 (Westinghouse Proprietary), approved by letter from H. N. Berkow, NRC, to J. A. Gresham, Westinghouse Electric Company, dated November 5, 2004;
 5. WCAP-12610-P-A, "VANTAGE+ Fuel Assembly Reference Core Report," (Westinghouse Proprietary);
 6. WCAP-8745-P-A, "Design Bases for the Thermal Overpower ΔT and Thermal Overtemperature ΔT Trip Functions," (Westinghouse Proprietary); and