

Exelon Generation Company, LLC      www.exeloncorp.com  
LaSalle County Station  
2601 North 21<sup>st</sup> Road  
Marseilles, IL 61341-9757

10 CFR 50.73

RA11-015

April 1, 2011

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

LaSalle County Station, Unit 1  
Facility Operating License No. NPF-11  
NRC Docket No. 50-373

Subject: Licensee Event Report 2011-002-00

In accordance with 10 CFR 50.73(a)(2)(i)(A), Exelon Generation Company (EGC), LLC, is submitting Licensee Event Report Number 2011-002-00.

There are no regulatory commitments in this report. Should you have any questions concerning this report, please contact Mr. Terrence W. Simpkin, Regulatory Assurance Manager at (815) 415-2800.

Respectfully,



*for Pete Karaba*

Peter J. Karaba  
Plant Manager  
LaSalle County Station

Enclosure: Licensee Event Report

Cc: Regional Administrator – NRC Region III  
NRC Senior Resident Inspector – LaSalle County Station

# LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA/Privacy Section (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects.resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

<b>1. FACILITY NAME</b> LaSalle County Station Unit 1	<b>2. DOCKET NUMBER</b> 05000373	<b>3. PAGE</b> 1 OF 3
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**4. TITLE**  
Unit Shutdown Required by Plant Technical Specifications Due to Pressure Boundary Leakage

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
02	09	2011	2011	- 002 -	0	04	01	2011	FACILITY NAME	DOCKET NUMBER
									FACILITY NAME	DOCKET NUMBER

<b>9. OPERATING MODE</b> 2	<b>11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §:</b> (Check all that apply)									
<b>10. POWER LEVEL</b> 6	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)						
	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)						
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)						
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)						
	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)						
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)						
	<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)						
<input type="checkbox"/> 20.2203(a)(2)(v)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER							
<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	Specify in Abstract below or in NRC Form 366A							

**12. LICENSEE CONTACT FOR THIS LER**

<b>FACILITY NAME</b> Jeffery C. Williams	<b>TELEPHONE NUMBER (Include Area Code)</b> (815) 415-2203
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**13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT**

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX
B	BN	ISV	A415	Y					

<b>14. SUPPLEMENTAL REPORT EXPECTED</b> <input type="checkbox"/> YES (If yes, complete 15. EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO	<b>15. EXPECTED SUBMISSION DATE</b>	MONTH	DAY	YEAR
		N/A		

**ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)**

On February 9, 2011, LaSalle Unit 1 was in Mode 2 (Startup) following a forced outage. During a walkdown of the drywell, a steam leak was observed coming from the Reactor Core Isolation Cooling Steam Supply Inboard Isolation Bypass/Warm up Valve (1E51-F076), a normally-closed, one inch, motor operated valve. The leak was determined to be on the valve bonnet extension-to-bonnet upper seal weld. At 1804 hours, the leak was classified as Pressure Boundary Leakage, and Technical Specification (TS) 3.4.5 Condition C was entered. TS 3.4.5 Required Action C.1 and C.2 require that the unit be in Mode 3 within 12 hours and Mode 4 within 36 hours.

On February 9, 2011, at 1830 hours, a plant shutdown of Unit 1 commenced. The unit entered Mode 3 at 2258 hours on February 9, and reached Mode 4 at 0353 hours on February 10, 2011. The seal weld was repaired, and the unit was restarted on February 10, 2011.

The equipment apparent cause evaluation determined that the cause was due to a weld defect or discontinuity from the original weld construction (i.e., manufacturing, installation/construction errors, etc.) of the upper seal weld that propagated through wall as a result of system loading and conditions (i.e., high pressure steam) during normal plant operations. Corrective action included repair of the defective upper seal weld area.

U.S. NUCLEAR REGULATORY COMMISSION

**LICENSEE EVENT REPORT (LER)  
CONTINUATION SHEET**

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE
LaSalle County Station, Unit 1	05000373	YEAR	SEQUENTIAL NUMBER	REV NO.	2    OF    3
		2011	- 002	- 0	

**NARRATIVE**

LaSalle County Station (LSCS) Unit 1 is a General Electric Boling Water Reactor with 3546 Megawatts Thermal Rated Core Power.

**A. CONDITION PRIOR TO EVENT:**

Unit(s): 1	Event Date: February 9, 2011	Event Time: 1804 CST
Reactor Mode(s): 2	Mode(s) Name: Startup	Power Level: 6 percent

**B. DESCRIPTION OF EVENT:**

On February 9, 2011, LaSalle Unit 1 was in Mode 2 (Startup) following a forced outage. During a walkdown of the drywell, a steam leak was observed coming from the Reactor Core Isolation Cooling (RI)[BN] Steam Supply Inboard Isolation Bypass/Warm up Valve (1E51-F076), a normally-closed, one inch, motor operated valve. The leak was determined to be on the valve bonnet extension-to-bonnet upper seal weld. At 1804 hours, the leak was classified as Pressure Boundary Leakage, and Technical Specifications (TS) 3.4.5 Condition C was entered. TS 3.4.5 Required Action C.1 and C.2 require that the unit be in Mode 3 within 12 hours and Mode 4 within 36 hours.

On February 9, 2011, at 1830 hours, a plant shutdown of Unit 1 commenced. The unit entered Mode 3 at 2258 hours on February 9, and reached Mode 4 at 0353 hours on February 10, 2011.

The event was reportable under 10 CFR 50.72(b)(2)(i) as an initiation of a nuclear plant shutdown required by TS. An ENS call (EN# 46605) was made at 1859 CST on February 9, 2011. The event is also reportable under 10 CFR 50.73(a)(2)(i)(A), as the completion of a nuclear plant shutdown required by TS.

**C. CAUSE OF EVENT:**

The equipment apparent cause evaluation determined that the cause was due to a weld defect or discontinuity from the original weld construction (i.e., manufacturing, installation/construction errors, etc.) of the upper seal weld that propagated through wall as a result of system loading and conditions (i.e., high pressure steam) during normal plant operations.

**D. SAFETY ANALYSIS:**

The safety significance of the event was minimal. Makeup capability was adequate to compensate for the leak. All Emergency Core Cooling Systems (ECCS) were operable and capable of fulfilling their intended safety functions during the period of excessive leakage. The event did not constitute a safety system functional failure.

**E. CORRECTIVE ACTIONS:**

Corrective Actions:

- The defective upper seal weld area was successfully repaired in a timely manner

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**NARRATIVE**

- An extent of condition review was performed to identify similar valves

Corrective Action to Prevent Recurrence:

- Options will be explored to procure a new and more robust replacement valve to ensure long term reliability

**F. PREVIOUS OCCURRENCES:**

LER 05000374/2009-02-00

On October 11, 2009, at 2333 CDT, an unidentified leakage on LaSalle Unit 2 exceeded the TS Limiting Condition for Operation 3.4.5.d limit of less than or equal to a two-gallon per minute increase in the previous 24-hour period while in Mode 1. A plant shutdown was commenced at 0300 hours on October 12, 2009, and Mode 4 was reached at 0005 hours on October 13, 2009. The unidentified leakage was determined to be from a packing leak on the Inboard Main Steam Isolation Valve Drain Line Isolation Valve (2B21-F016). The packing leak was repaired and the unit was restarted on October 14, 2009. Corrective actions to prevent recurrence included training maintenance personnel on the different types of gaskets used on pressure seal valves, and revisions to the subject maintenance procedures.

**G. COMPONENT FAILURE DATA:**

Manufacturer: Anderson Greenwood Company (A415)  
Model Number: N03-6498-510