



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
245 PEACHTREE CENTER AVENUE NE, SUITE 1200
ATLANTA, GEORGIA 30303-1257

March 31, 2011

Mr. J. Randy Johnson
Vice President - Farley
Southern Nuclear Operating Company, Inc.
7388 North State Highway 95
Columbia, AL 36319

SUBJECT: NOTIFICATION OF JOSEPH M. FARLEY NUCLEAR PLANT - COMPONENT
DESIGN BASES INSPECTION - NRC INSPECTION REPORT 05000348,
364/2011010

Dear Mr. Johnson:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a component design bases inspection at your Farley Nuclear Plant during the weeks of August 29 – September 2, September 12 – 16 and September 26 – 30, 2011. The inspection team will be led by Shane Sandal, a Senior Reactor Inspector from the NRC's Region II Office. This inspection will be conducted in accordance with the baseline inspection procedure, Procedure 71111.21, Component Design Bases Inspection, issued December 6, 2010.

The inspection will evaluate the capability of risk significant / low margin components to function as designed and to support proper system operation. The inspection will also include a review of selected operator actions, operating experience, and modifications.

During a telephone conversation on March 28, 2011, Mr. Sandal confirmed with Mr. Oldfield of your staff, arrangements for an information gathering site visit and the three-week onsite inspection. The schedule is as follows:

- Information gathering visit: Week of August 8 – 12, 2011
- Onsite weeks: August 29 – September 2, September 12 – 16 and September 26 – 30, 2011

The purpose of the information gathering visit is to meet with members of your staff to identify risk-significant components and operator actions. Information and documentation needed to support the inspection will also be identified. Mr. George MacDonald, a Region II Senior Reactor Analyst, will accompany Mr. Sandal during the information gathering visit to review probabilistic risk assessment data and identify risk significant components which will be examined during the inspection.

The enclosure lists documents that will be needed prior to the information gathering visit. Please provide the referenced information to the Region II office by August 1, 2011. Contact Mr. Sandal with any questions concerning the requested information. The inspectors will try to minimize your administrative burden by specifically identifying only those documents required for inspection preparation.

Additional documents will be requested during the information gathering visit. The additional information will need to be made available to the team in the Region II office prior to the inspection team's preparation week of August 22. Mr. Sandal, will also discuss the following inspection support administrative details: availability of knowledgeable plant engineering and licensing personnel to serve as points of contact during the inspection; method of tracking inspector requests during the inspection; licensee computer access; working space; arrangements for site access; and other applicable information.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact Mr. Sandal at (404) 997-4513 or me at (404) 997-4519.

Sincerely,
/RA/

Binoy B. Desai, Chief
Engineering Branch 1
Division of Reactor Safety

Docket No.: 50-348, 50-364
License No.: NPF-2, NPF-8

Enclosure: Information Request For Joseph M. Farley Nuclear Plant – Component Design
Bases Inspection

cc w/enc/: (See page 3)

cc w/encl:

B. D. McKinney, Jr.
Regulatory Response Manager
Southern Nuclear Operating Company, Inc.
Electronic Mail Distribution

M. J. Ajluni
Nuclear Licensing Director
Southern Nuclear Operating Company, Inc.
Electronic Mail Distribution

T. D. Honeycutt
Regulatory Response Supervisor
Southern Nuclear Operating Company, Inc.
Electronic Mail Distribution

Todd L. Youngblood
Plant Manager
Joseph M. Farley Nuclear Plant
Electronic Mail Distribution

R. D. Baker
Licensing Supervisor
Southern Nuclear Operating Company, Inc.
Electronic Mail Distribution

E. G. Anners
Licensing Engineer
Southern Nuclear Operating Company, Inc.
Electronic Mail Distribution

Jeffrey T. Gasser
Chief Nuclear Officer
Southern Nuclear Operating Company, Inc.
Electronic Mail Distribution

N. J. Stringfellow
Licensing Manager
Southern Nuclear Operating Company, Inc.
Electronic Mail Distribution

L. Mike Stinson
Vice President
Fleet Operations Support
Southern Nuclear Operating Company, Inc.
Electronic Mail Distribution

Paula Marino
Vice President
Engineering
Southern Nuclear Operating Company, Inc.
Electronic Mail Distribution

Moanica Caston
Vice President and General Counsel
Southern Nuclear Operating Company, Inc.
Electronic Mail Distribution

Chris Clark
Commissioner
Georgia Department of Natural Resources
Electronic Mail Distribution

John G. Horn
Site Support Manager
Joseph M. Farley Nuclear Plant
Southern Nuclear Operating Company, Inc.
Electronic Mail Distribution

Ted V. Jackson
Emergency Response and Radiation
Program Manager
Environmental Protection Division
Georgia Department of Natural Resources
Electronic Mail Distribution

Tom W. Pelham
Performance Improvement Supervisor
Joseph M. Farley Nuclear Plant
Southern Nuclear Operating Company, Inc.
Electronic Mail Distribution

Cynthia A. Sanders
Radioactive Materials Program Manager
Environmental Protection Division
Georgia Department of Natural Resources
Electronic Mail Distribution

James C. Hardeman
Environmental Radiation Program Manager
Environmental Protection Division
Georgia Department of Natural Resources
Electronic Mail Distribution

(cc w/encl cont'd –See page 4)

(cc w/encl cont'd)
William D. Oldfield
Principal Licensing Engineer
Joseph M. Farley Nuclear Plant
Electronic Mail Distribution

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
Joseph M. Farley Nuclear Plant
U.S. NRC
7388 N. State Highway 95
Columbia, AL 36319

Mr. Mark Culver
Chairman
Houston County Commission
P. O. Box 6406
Dothan, AL 36302

James A. Sommerville
Program Coordination Branch Chief
Environmental Protection Division
Georgia Department of Natural Resources
Electronic Mail Distribution

James L. McNees, CHP
Director
Office of Radiation Control
Alabama Dept. of Public Health
P. O. Box 303017
Montgomery, AL 36130-3017

State Health Officer
Alabama Dept. of Public Health
RSA Tower - Administration
Suite 1552
P.O. Box 30317
Montgomery, AL 36130-3017

F. Allen Barnes
Director
Environmental Protection Division
Georgia Department of Natural Resources
Electronic Mail Distribution

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Sincerely,
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Binoy B. Desai, Chief
Engineering Branch 1
Division of Reactor Safety

Docket No.: 50-348, 50-364
License No.: NPF-2, NPF-8

Enclosure: Information Request For Joseph M. Farley Nuclear Plant – Component Design Bases Inspection

cc w/enc/: (See page 3)

Distribution w/encl:

RIDSNRRDIRS
PUBLIC
RidsNrrPMFarley Resource
S. Shaeffer, RII

PUBLICLY AVAILABLE NON-PUBLICLY AVAILABLE SENSITIVE NON-SENSITIVE
ADAMS: Yes ACCESSION NUMBER: ML110900685 _____ SUNSI REVIEW COMPLETE

OFFICE	RII:DRS	RII:DRS									
SIGNATURE	RA	RA									
NAME	SANDAL	DESAI									
DATE	3/ 31/ 2011	3/31/2011									
E-MAIL COPY?	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES	

INFORMATION REQUEST FOR JOSEPH M. FARLEY NUCLEAR PLANT COMPONENT DESIGN BASES INSPECTION

Please provide the information electronically in “.pdf” files, Excel, or other searchable format on CDROM (or FTP site, Sharepoint, etc.) The CDROM (or website) should be indexed and hyperlinked to facilitate ease of use.

1. From your most-recent probabilistic safety analysis (PSA) excluding external events and fires:
 - a. Two risk rankings of components from your site-specific probabilistic safety analysis (PSA): one sorted by Risk Achievement Worth (RAW), and the other sorted by Birnbaum Importance.
 - b. A list of the top 500 cutsets.
2. From your most-recent probabilistic safety analysis (PSA) including external events and fires:
 - a. Two risk rankings of components from your site-specific probabilistic safety analysis (PSA): one sorted by Risk Achievement Worth (RAW), and the other sorted by Birnbaum Importance.
 - b. A list of the top 500 cutsets.
3. Risk ranking of operator actions from your site specific PSA sorted by RAW. Provide human reliability worksheets for these items.
4. List of time critical operator actions with a brief description of each action.
5. List of Emergency and Abnormal Operating Procedures revised (significant) since March 1, 2009 with a brief description of each revision.
6. List of components with low design margins (i.e., pumps closest to the design limit for flow or pressure, diesel generator close to design required output, heat exchangers close to rated design heat removal, MOV risk-margin rankings, etc.) and associated evaluations or calculations.
7. List of station operating experience evaluations/reviews performed and documented in the station’s corrective action program for industry events and safety related equipment failures/vulnerabilities [as communicated by NRC generic communications, industry communications, 10 CFR part 21 notifications, etc.] since March 1, 2009.
8. List and brief description of safety related SSC design modifications implemented since March 1, 2009.
9. List and brief description of common-cause component failures that have occurred since March 1, 2009.
10. List and brief description of operability evaluations completed since March 1, 2009.
11. List of equipment on the site’s Station Equipment Reliability Issues List, including a description of the reason(s) why each component is on that list and summaries (if available) of your plans to address the issue(s).

Enclosure

12. List and brief description of equipment currently in degraded or nonconforming status as described in RIS 05-020.
13. List and reason for equipment classified in maintenance rule (a)(1) status since March 1, 2009 to present.
14. Copies of System Descriptions (or the like design basis documents) for Safety-Related Systems
15. Copy of UFSAR(s)
16. Copy of Technical Specification(s)
17. Copy of Technical Specifications Bases
18. Copy of Technical Requirements Manual(s)
19. List and brief description of Root Cause Evaluations that have been performed since March 1, 2009
20. In-service Testing Program Procedure(s)
21. Corrective Action Program Procedure(s)
22. One line diagram of electrical plant (electronic and full size – hard copy week of August 8)
23. Index and legend for electrical plant one-line diagrams
24. Primary AC calculation(s) for safety-related buses
25. Primary DC calculation(s) for safety-related buses
26. PI&D's for ECCS systems (electronic and 1/2 size – hard copy week of August 8)
27. Index and Legend for PI&Ds
28. Copy of design bases documents for ECCS systems
29. Copy of Operability determination procedure(s)
30. Copies of condition reports associated with findings from previous CDBI (if applicable)
31. Index (procedure number, titles, and current revision) of station Emergency Operating Procedures (EOPs), Abnormal Operating Procedures (AOPs), and Annunciator Response Procedures (ARPs)
32. Contact information for a person to discuss PRA information prior to the information gathering trip (name, title, phone number, and e-mail address)