UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
475 ALLENDALE ROAD
KING OF PRUSSIA, PA 19406-1415

March 29, 2011

Mr. Paul Freeman
Site Vice President, North Region
Seabrook Nuclear Power Plant
NextEra Energy Seabrook, LLC
c/o Mr. Michael O'Keefe
P.O. Box 300
Seabrook, NH 03874

SUBJECT: SEABROOK NUCLEAR GENERATING STATION - NOTIFICATION OF CONDUCT OF A TRIENNIAL FIRE PROTECTION BASELINE INSPECTION

Dear Mr. Freeman:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region I staff will conduct a triennial fire protection baseline inspection at Seabrook Nuclear Generating Station, in July and August, 2011. The inspection team will be led by Roy Fuhrmeister, a fire protection specialist from the NRC Region I Office. The team will be composed of personnel from NRC Region I. The inspection will be conducted in accordance with IP 71111.05T, the NRC's baseline fire protection inspection procedure.

The schedule for the inspection is as follows:

- Information gathering visit - July 13, 14, 2011
- Week of onsite inspection - July 25 – 29 and August 8 – 12, 2011

The purposes of the information gathering visit are to obtain information and documentation needed to support the inspection, to become familiar with the Seabrook Nuclear Generating Station fire protection programs, fire protection features, post-fire safe shutdown capabilities and plant layout, mitigating strategies to address Section B.5.b of the Interim Compensatory Measures Order, EA-02-026, of February 25, 2002/10 CFR 50.54(hh)(2); and, as necessary, obtain plant specific site access training and badging for unescorted site access. A list of the types of documents the team may be interested in reviewing, and possibly obtaining, are listed in Enclosures 1 and 2. The team leader will contact your staff with specific document requests.

During the information gathering visit, the team will also discuss the following inspection support administrative details: office space size and location; specific documents requested to be made available to the team in their office spaces; arrangements for reactor site access (including radiation protection training, security, safety, and fitness for duty requirements); and the availability of knowledgeable plant engineering and licensing organization personnel to serve as points of contact during the inspection.

We request that during the onsite inspection week you ensure that copies of analyses, evaluations, or documentation regarding the implementation and maintenance of the Seabrook Nuclear Generating Station fire protection program, including post-fire safe shutdown capability, be readily accessible to the team for their review. Of specific interest for the fire protection portion of the inspection are those documents which establish that your fire protection program satisfies NRC regulatory requirements and conforms to applicable NRC and industry fire protection guidance. For the B.5.b portion of the inspection, those documents implementing your mitigating strategies and demonstrating the management of your commitments for the strategies are of specific interest. Also, personnel should be available at the site during the inspection who are knowledgeable regarding those plant systems required to achieve and maintain safe shutdown conditions from inside and outside the control room (including the electrical aspects of the relevant post-fire safe shutdown analyses), reactor plant fire protection systems and features, and the Seabrook Nuclear Generating Station fire protection program and its implementation.

This letter does not contain new or amended information collection requirements subject to the Paperwork Reduction Act of 1995 (44 U.S.C. 3501 et seq.). Existing information collection requirements were approved by the Office of Management and Budget, control number 3150-0011. The NRC may not conduct or sponsor, and a person is not required to respond to, a request for information or an information collection requirement unless the requesting document displays a currently valid Office of Management and Budget control number.

Your cooperation and support during this inspection will be appreciated. If you have questions concerning this inspection, or the inspection team's information or logistical needs, please contact Roy Fuhrmeister, the team leader, in the Region I Office at 610-337-5059.

Sincerely,

John F. Rogge, Chief
Engineering Branch 3

Docket No.: 50-443
License No.: NPF-86

Enclosures:
1. Listing of Reactor Fire Protection Program Supporting Documentation
2. Mitigating Strategies Supporting Documentation

cc: Distribution via ListServ

We request that during the onsite inspection week you ensure that copies of analyses, evaluations, or documentation regarding the implementation and maintenance of the Seabrook Nuclear Generating Station fire protection program, including post-fire safe shutdown capability, be readily accessible to the team for their review. Of specific interest for the fire protection portion of the inspection are those documents which establish that your fire protection program satisfies NRC regulatory requirements and conforms to applicable NRC and industry fire protection guidance. For the B.5.b portion of the inspection, those documents implementing your mitigating strategies and demonstrating the management of your commitments for the strategies are of specific interest. Also, personnel should be available at the site during the inspection who are knowledgeable regarding those plant systems required to achieve and maintain safe shutdown conditions from inside and outside the control room (including the electrical aspects of the relevant post-fire safe shutdown analyses), reactor plant fire protection systems and features, and the Seabrook Nuclear Generating Station fire protection program and its implementation.

This letter does not contain new or amended information collection requirements subject to the Paperwork Reduction Act of 1995 (44 U.S.C. 3501 et seq.). Existing information collection requirements were approved by the Office of Management and Budget, control number 3150-0011. The NRC may not conduct or sponsor, and a person is not required to respond to, a request for information or an information collection requirement unless the requesting document displays a currently valid Office of Management and Budget control number.

Your cooperation and support during this inspection will be appreciated. If you have questions concerning this inspection, or the inspection team's information or logistical needs, please contact Roy Fuhrmeister, the team leader, in the Region I Office at 610-337-5059.

Sincerely,

/RA/

John F. Rogge, Chief
Engineering Branch 3

Docket No.: 50-443
License No.: NPF-86

Enclosures:
1. Listing of Reactor Fire Protection Program Supporting Documentation
2. Mitigating Strategies Supporting Documentation

cc: Distribution via ListServ

Distribution w/encl: (via e-mail)
W. Dean, RA
D. Lew, DRA
D. Roberts, DRP
J. Clifford, DRP
P. Wilson, DRS
S. Weerakkody, DRS
A. Burritt, DRP
L. Cline, DRP
A. Turilin, DRP
C. Douglas, DRP
W. Raymond, DRP, SRI
J. Johnson, DRP, RI
A. Cass, DRP, Resident OA
J. Trapp, RI OEDO
D. Bearde, DRS
RidsNrrPMSeabrook Resource
RidsNrrDorlLpl1-2 Resource
ROPreports Resource

SUNSI Review Complete: __JFR__ MD 3.4 Non-Public ADAMS ML110880663
DOCUMENT NAME: G:\DRS\Engineering Branch 3\Fuhrmeister\Seabrook\90DayLetter.rtf
After declaring this document "An Official Agency Record" it will be released to the Public. To receive a copy of this document, indicate in the box: "C" = Copy without enclosure "E" = Copy with enclosure "N" = No copy

OFFICE	RI/DRS		RI/DRS			
NAME	RLFuhrmeister/JFR		JFRogge			
DATE	03/29/11		03/29/11			

ENCLOSURE 1

Reactor Fire Protection Program Supporting Documentation

[Note: This is a broad list of the documents the NRC inspection team may be interested in reviewing, and possibly obtaining, during the information gathering site visit.]

1. The current version of the Fire Protection Program and Fire Hazards Analysis.

2. Corrective action closeouts as part of the perator manual actions corrective actions.

3. List of identified fire induced circuit failure configurations for select fire areas.

4. Corrective actions for fire-induced circuit failures, both single and multiple spurious actuations for select fire areas.

5. Cable routing for components and equipment credited for safe shutdown for select fire areas.

6. List of protected safe shutdown train equipment for select fire areas.

7. Current versions of the fire protection program implementing procedures (e.g., administrative controls, surveillance testing, and fire brigade).

8. Fire brigade training program and pre-fire plans.

9. Post-fire safe shutdown systems and separation analysis.

10. Post-fire alternative shutdown analysis.

11. Piping and instrumentation (flow) diagrams showing the components used to achieve and maintain hot standby and cold shutdown for fires outside the control room and those components used for those areas requiring alternative shutdown capability.

12. Plant layout and equipment drawings which identify the physical plant locations of hot standby and cold shutdown equipment.

13. Plant layout drawings which identify plant fire area delineation, areas protected by automatic fire suppression and detection, and the locations of fire protection equipment.

14. Plant layout drawings which identify the general location of the post-fire emergency lighting.

15. Plant operating procedures which would be used and describe shutdown from inside the control room with a postulated fire occurring in any plant area outside the control room, procedures which would be used to implement alternative shutdown capability in the event of a fire in either the control or cable spreading room.

16. Maintenance and surveillance testing procedures for alternative shutdown capability and fire barriers, detectors, pumps and suppression systems.

17. Maintenance procedures which routinely verify fuse breaker coordination in accordance with the post-fire safe shutdown coordination analysis.

18. A list of significant fire protection and post-fire safe shutdown related design change packages and Generic Letter 86-10 evaluations.

19. The reactor plant's Individual Plant Examination of External Events (IPEEE), results of any post-IPEEE reviews, and listings of actions taken/plant modifications conducted in response to IPEEE information.

20. Organization charts of site personnel down to the level of fire protection staff personnel.

21. If applicable, layout/arrangement drawings of potential reactor coolant/recirculation pump lube oil system leakage points and associated lube oil collection systems.

22. Licensing basis documents for fire protection (safety evaluation reports, pertinent sections of the final safety analysis report, exemptions, deviations, letters to/from the NRC regarding fire protection/fire safe shutdown, etc.).

23. Procedures/instructions that control the configuration of the reactor plant's fire protection program, features, and post-fire safe shutdown methodology and system design.

24. A list of applicable codes and standards related to the design of plant fire protection features and evaluations of code deviations.

25. Procedures/instructions that govern the implementation of plant modifications, maintenance, and special operations, and their impact on fire protection.

26. Internal and external self-assessments, audits, peer-assessments or similar reviews related to post-fire safe shutdown capability or the fire protection program completed since the previous NRC fire protection triennial inspection.

27. Recent quality assurance (QA) surveillances of fire protection activities.

28. A listing of open and closed fire protection condition reports (problem reports/NCRs/EARs/problem identification and resolution reports) since the last triennial inspection.

ENCLOSURE 2

Mitigating Strategies Supporting Documentation

[Note: This is a broad list of the documents the NRC inspection team may be interested in reviewing, and possibly obtaining, during the information gathering site visit.]

1. A list of all modifications to regulatory commitments made to meet the requirements of Section B.5.b of the ICM Order, EA-02-026, dated February 25, 2002, the subsequently imposed license conditions, and 10 CFR 50.54(hh)(2).

2. Lists of procedures/guidelines that were revised or generated to implement the mitigation strategies. These could be extensive damage mitigation guidelines (EDMGs), severe accident management guidelines (SAMGs), emergency operating procedures (EOPs), abnormal operating procedures (AOPs), etc.

3. A matrix that shows the correlation between the mitigation strategies identified in Nuclear Energy Institute 06-12 and the site-specific procedures or guidelines that are used to implement each strategy.

4. Engineering evaluations/calculations that were used to verify engineering bases for the mitigation strategies.

5. Piping and instrumentation diagram (P&ID) or simplified flow diagrams for systems relied upon in the mitigation strategies. These could be the type used for training.

6. A list of modification packages or simplified drawings/descriptions of modifications that were made to plant systems to implement the mitigation strategies.

7. Copies of procedures used to inventory equipment (hoses, fittings, pumps, etc.) required to be used to implement the mitigation strategies.

8. A list of B.5.b strategies, if any, which have implementing details that differ from that documented in the submittals and the safety evaluation report.

9. A copy of site general arrangement drawing(s) that show the majority of buildings/areas referenced in B.5.b documents.

10. Training records/ training matrix/ lesson plans related to B.5.b.

11. Copies of Memoranda of Understanding (MOUs) (e.g., with local fire departments) required to implement any mitigating strategies.