



411 Fayetteville Street Mall
Raleigh NC 27602

10 CFR 50.55a

Serial: RA-11-012

March 23, 2011

United States Nuclear Regulatory Commission
ATTENTION: Document Control Desk
Washington, DC 20555-0001

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-261 / RENEWED LICENSE NO. DPR-23

**RESPONSE TO REQUEST FOR ADDITIONAL INFORMATION – REACTOR
VESSEL BELTLINE WELD RELIEF REQUEST (RR-23) (ME5407)**

Ladies and Gentlemen:

By letter dated January 27, 2011, Carolina Power & Light Company (CP&L), now doing business as Progress Energy Carolinas, Inc., submitted a proposed alternative to the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Section XI inservice inspection (ISI) requirements regarding examination of certain reactor pressure vessel welds at H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2. (ML110330085)

A Request for Additional Information (RAI) was received via electronic correspondence on March 7, 2011. (ML110660469) CP&L's response to the RAI is attached.

No new regulatory commitments have been made in this letter.

If you have questions regarding this submittal, please contact Donna Alexander, Interim Manager, Nuclear Regulatory Affairs, at (919) 546-5357.

Sincerely,

Donna Alexander
Interim Manager, Nuclear Regulatory Affairs

DBM

Attachment: Response to Request for Additional Information – Reactor Vessel Beltline
Weld Relief Request (RR-23) (ME5407)

cc USNRC Region II
USNRC Resident Inspector – HBRSEP, Unit No. 2
B. Mozafari, NRR Project Manager –HBRSEP, Unit No. 2

A 047
NRR

Attachment

**Response to Request for Additional Information –
Reactor Vessel Beltline Weld Relief Request (RR-23) (ME5407)**

**Response to Request for Additional Information –
Reactor Vessel Beltline Weld Relief Request (RR-23) (ME5407)**

Regarding observed indications from the most recent (3rd) inservice inspection (ISI) interval examinations as documented in Table 2 of Proposed Alternative on page 3 of the request dated January 27, 2011:

NRC Request:

Clearly state the location and size of the one indication that was found in the reactor pressure vessel beltline area. Also, state whether this indication was observed in the 1st and/or 2nd ISI interval inspections.

Response:

Data from the 3rd ISI interval inspection shows a sub-surface indication located in the Intermediate Shell-to-Upper Shell circumferential weld 10-273 (identified as RPV 4 in the inspection report). The indication dimensions were recorded as 0.85 inches long, circumferentially, and 0.28 inches deep. The center of the indication is located 26.47 inches counter clockwise from the centerline of longitudinal weld 1-273A (identified as RPV 14 in the inspection report), with an “S” dimension of 2.11 inches.

This indication was observed in the second interval inspection (1990).

This indication was not observed in the first interval inspection (1982).

NRC Request:

State whether the size of the indication changed during the course of the three inspections, and if so, whether the size of the indication could be attributed to improved inspection procedures.

Response:

The indication detected in the 2nd ISI interval inspection was at approximately the same location in weld 10-273. The indication dimensions were recorded as 0.59 inches long, circumferentially, and 0.49 inches deep, with an “S” dimension of 2.35 inches. The difference in size recorded could be attributed to the increased rigor of the demonstration process required by Section XI, Appendix VIII, which was not in effect at the time of the first two interval inspections. The performance demonstration requirements in Appendix VIII substantially improve the ability of an examiner to detect and characterize flaws in examined components.