



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

March 29, 2011

Mr. David A. Heacock
President and Chief Nuclear Officer
Virginia Electric and Power Company
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: SURRY POWER STATION, UNIT NO. 2 – REQUEST FOR ADDITIONAL
INFORMATION REGARDING THE 2009 REFUELING OUTAGE STEAM
GENERATOR TUBE INSPECTIONS FOR END OF CYCLE 22 (TAC NO.
ME5689)

Dear Mr. Heacock:

By letter dated May 24, 2010, Virginia Electric and Power Company, the licensee, submitted information summarizing the results of the 2009 steam generator tube inspections at Surry Power Station, Unit 2.

The U.S. Nuclear Regulatory Commission staff has reviewed the information the licensee provided and determined that additional information is needed in order to complete the review. The requested additional information is enclosed. Please respond within 30 days of the date of this letter.

If you have any questions, please feel free to contact me at 301-415-1438.

Sincerely,

A handwritten signature in cursive script, appearing to read "Karen Cotton".

Karen Cotton, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-281

Enclosure:
Request for Additional Information

cc w/encl: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION
SURRY POWER STATION, UNIT NO. 2
2009 STEAM GENERATOR TUBE INSPECTIONS
TAC NO. ME5689
DOCKET NO. 50-281

By letter dated May 24, 2010 (Agencywide Documents Access and Management System, Accession No. ML101530533), Virginia Electric and Power Company, the licensee, submitted information summarizing the results of the 2009 steam generator (SG) tube inspections at Surry Power Station, Unit No. 2.

In order to complete the review, the U.S. Nuclear Regulatory Commission (NRC) staff requests the following information:

1. Please clarify the difference between an overexpansion and overroll.
2. With respect to tubes that may have potentially elevated residual stresses as a result of fabrication, please clarify the number of high row tubes (i.e., tubes in rows where the U-bends were not stress relieved following bending) in SG A that have this condition (either Tier 1 or Tier 2 tubes). It is the NRC staff's understanding that none of the low row tubes (i.e., tubes in rows where the U-bends were stress relieved) in either SGs A, B, or C have potentially elevated residual stresses (i.e., no eddy current offset) as a result of fabrication and that there are 2 high row tubes in SG B and 14 high row tubes in SG C that have potentially elevated residual stresses (i.e., 2 sigma tubes). Please confirm the NRC staff's understanding.
3. Please discuss the results of the secondary side inspections (other than foreign object search and retrieval) in SG C. Please discuss how the 2009 inspection results compare to the inspection results obtained in 2008 for this SG.
4. During a prior outage, corrosion degradation was observed in the channel head area and in an unplugged tube (which was subsequently replugged) in SG A. Please discuss whether these areas were inspected/monitored during the 2009 outage. If so, please discuss the results.
5. Based on a review of Tables 9 and 10, it appears that some tubes with no degradation were plugged since they had foreign objects present. Please identify these tubes (i.e., tubes with loose part indications and no associated wear that were plugged) and the tubes that were plugged because they were not expanded in the tubesheet (NTE tubes).

Enclosure

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/RA/

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*by memo dated

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