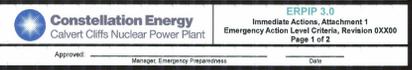


	GENERAL EMERGENCY	SITE AREA EMERGENCY	ALERT	UNUSUAL EVENT																																				
R Abnorm. Rad Levels / Rad Effluents	1 Offsite Rad Conditions	RG1.1 ANY radiation monitor reading > Table R-1 column "GE" for ≥ 15 min. (Note 1) • Do not delay declaration awaiting dose assessment results • If dose assessment results are available, declaration should be based on dose assessment instead of radiation monitor values (see EAL RG1.2) RG1.2 Dose assessment using actual meteorology indicates doses > 1,000 mRem TEDE or 5,000 mRem thyroid CDE at or beyond the site boundary RG1.3 Field survey results indicate closed window dose rates > 1,000 mRem/hr expected to continue for ≥ 60 min. at or beyond the site boundary OR Analyses of field survey samples indicate thyroid CDE > 5,000 mRem for 1 hr of inhalation at or beyond the site boundary (Note 1)	RS1.1 ANY radiation monitor reading > Table R-1 column "SAE" for ≥ 15 min. (Note 1) • Do not delay declaration awaiting dose assessment results • If dose assessment results are available, declaration should be based on dose assessment instead of radiation monitor values (see EAL RS1.2) RS1.2 Dose assessment using actual meteorology indicates doses > 100 mRem TEDE or 500 mRem thyroid CDE at or beyond the site boundary RS1.3 Field survey results indicate closed window dose rates > 100 mRem/hr expected to continue for ≥ 60 min. at or beyond the site boundary OR Analyses of field survey samples indicate thyroid CDE > 500 mRem for 1 hr of inhalation at or beyond the site boundary (Note 1)	RA1.1 ANY gaseous monitor reading > Table R-1 column "Alert" for ≥ 15 min. (Note 2) RA1.2 Liquid monitor reading > Table R-1 column "Alert" for ≥ 15 min. (Note 2) RA1.3 Confirmed sample analyses for gaseous or liquid releases indicate concentrations or release rates > 200 x ODCM limits for ≥ 15 min. (Note 2)	RU1.1 ANY gaseous monitor reading > Table R-1 column "UE" for ≥ 60 min. (Note 2) RU.2 Liquid monitor reading > Table R-1 column "UE" for ≥ 60 min. (Note 2) RU.3 Confirmed sample analyses for gaseous or liquid releases indicate concentrations or release rates > 2 x ODCM limits for ≥ 60 min. (Note 2)																																			
	2 Onsite Rad Conditions & Spent Fuel Events	<table border="1"> <caption>Table R-1 Effluent Monitor Classification Thresholds</caption> <thead> <tr> <th>Monitor</th> <th>GE</th> <th>SAE</th> <th>ALERT</th> <th>UE</th> </tr> </thead> <tbody> <tr> <td>WRNGM (RI-5415)</td> <td>3.2E+09 µCi/sec</td> <td>3.2E+08 µCi/sec</td> <td>3.2E+07 µCi/sec</td> <td>3.2E+06 µCi/sec</td> </tr> <tr> <td>Main Steam Effluent (RI-5421, RI-5422)</td> <td>40.0 rem/hr</td> <td>4.0 rem/hr</td> <td>0.40 rem/hr</td> <td>N/A</td> </tr> <tr> <td>Main Vent (RI-5415)</td> <td>N/A</td> <td>N/A</td> <td>N/A</td> <td>2.0E+05 cpm</td> </tr> <tr> <td>Waste Processing (RI-5410)</td> <td>N/A</td> <td>N/A</td> <td>N/A</td> <td>4.0E+05 cpm</td> </tr> <tr> <td>Fuel Handling Area Vent (RI-5420)</td> <td>N/A</td> <td>N/A</td> <td>N/A</td> <td>3.4E+05 cpm</td> </tr> <tr> <td>Liquid Waste Disch* (RE-2201)</td> <td>N/A</td> <td>N/A</td> <td>off-scale hi</td> <td>8.4E+05 cpm</td> </tr> </tbody> </table> <p>*with effluent discharge not isolated</p>				Monitor	GE	SAE	ALERT	UE	WRNGM (RI-5415)	3.2E+09 µCi/sec	3.2E+08 µCi/sec	3.2E+07 µCi/sec	3.2E+06 µCi/sec	Main Steam Effluent (RI-5421, RI-5422)	40.0 rem/hr	4.0 rem/hr	0.40 rem/hr	N/A	Main Vent (RI-5415)	N/A	N/A	N/A	2.0E+05 cpm	Waste Processing (RI-5410)	N/A	N/A	N/A	4.0E+05 cpm	Fuel Handling Area Vent (RI-5420)	N/A	N/A	N/A	3.4E+05 cpm	Liquid Waste Disch* (RE-2201)	N/A	N/A	off-scale hi	8.4E+05 cpm
	Monitor	GE	SAE	ALERT	UE																																			
WRNGM (RI-5415)	3.2E+09 µCi/sec	3.2E+08 µCi/sec	3.2E+07 µCi/sec	3.2E+06 µCi/sec																																				
Main Steam Effluent (RI-5421, RI-5422)	40.0 rem/hr	4.0 rem/hr	0.40 rem/hr	N/A																																				
Main Vent (RI-5415)	N/A	N/A	N/A	2.0E+05 cpm																																				
Waste Processing (RI-5410)	N/A	N/A	N/A	4.0E+05 cpm																																				
Fuel Handling Area Vent (RI-5420)	N/A	N/A	N/A	3.4E+05 cpm																																				
Liquid Waste Disch* (RE-2201)	N/A	N/A	off-scale hi	8.4E+05 cpm																																				
3 CR/CAS/SAS Rad																																								
H Hazards & Other Conditions Affecting Plant Safety	1 Natural or Destructive Phenomena	<table border="1"> <caption>Table H-1 Safe Shutdown Areas</caption> <tbody> <tr> <td>Control Room</td> </tr> <tr> <td>Containment</td> </tr> <tr> <td>Auxiliary Building</td> </tr> <tr> <td>Diesel Generator Rooms</td> </tr> <tr> <td>Intake Structure</td> </tr> <tr> <td>140C DG Buildings</td> </tr> <tr> <td>RWT</td> </tr> <tr> <td>RWT Rooms</td> </tr> <tr> <td>CST No. 12</td> </tr> <tr> <td>FOST No. 21</td> </tr> <tr> <td>Auxiliary Feed Pump Rooms</td> </tr> </tbody> </table>				Control Room	Containment	Auxiliary Building	Diesel Generator Rooms	Intake Structure	140C DG Buildings	RWT	RWT Rooms	CST No. 12	FOST No. 21	Auxiliary Feed Pump Rooms																								
	Control Room																																							
	Containment																																							
	Auxiliary Building																																							
	Diesel Generator Rooms																																							
	Intake Structure																																							
	140C DG Buildings																																							
	RWT																																							
	RWT Rooms																																							
	CST No. 12																																							
FOST No. 21																																								
Auxiliary Feed Pump Rooms																																								
2 Fire or Explosion	<p>Notes</p> <ol style="list-style-type: none"> The ED should not wait until the applicable time has elapsed, but should declare the event as soon as it is determined that the condition will likely exceed the applicable time. The ED should not wait until the applicable time has elapsed, but should declare the event as soon as it is determined that the release duration has exceeded, or will likely exceed, the applicable time. In the absence of data to the contrary, assume that the release duration has exceeded the applicable time if an ongoing release is detected and the release start time is unknown. If loss of water level in the refueling pathway occurs while in Mode 5, 6 or D, consider classification under EALs CUS 1, CUS 2 or CUS 3. The ED should not wait until the applicable time has elapsed, but should declare the event as soon as it is determined that the condition has exceeded, or will likely exceed, the applicable time. If the equipment in the stated area was already inoperable, or out of service, before the event occurred, then EAL HA3.1 should not be declared as it will have no adverse impact on the ability of the plant to safely operate or safely shutdown beyond that already allowed by Technical Specifications at the time of the event. The lowest RVLMS indication is the 10 in. alarm, which is 10 in. above top of active fuel. Therefore, this indicator should only be used when a valid RFP/RCS level indication is not available. The NIEC can be contacted by calling (303) 273-8500. Select option #1 and inform the analyst you wish to confirm recent seismic activity in the vicinity of Calvert Cliffs Nuclear Power Plant. Provide the analyst with the following CONPP coordinates: 38° 25' 39.7" north latitude, 76° 28' 45" west longitude. High temperature in Containment may induce a current error in the Mineral Insulated (MI) cable running through Containment to the meter. The CHRMM (12)-RI-317 A&B may not detect this value (6 R/hr) under these conditions. When Containment temperature reaches 300°F, the meter will indicate approximately 40 R/hr for a few minutes then drop to approximately 10 R/hr after three hours. This information is to provide guidance on determining the validity of the readings under the specified high temperature conditions. 																																							
3 Hazardous Gas	<table border="1"> <tr> <td>HG4.1</td> <td>HG4.2</td> <td>HS4.1</td> <td>HS4.2</td> <td>HA4.1</td> <td>HA4.2</td> <td>HA4.3</td> <td>HA4.4</td> <td>HA4.5</td> <td>HA4.6</td> </tr> <tr> <td>A HOSTILE ACTION has occurred such that plant personnel are unable to operate equipment required to maintain ANY of the following safety function acceptance criteria: • Reactivity control (RC) • Vial Auxiliaries (VA) • RCS pressure and inventory control (PIC) • Core & RCS heat removal (HR)</td> <td>A HOSTILE ACTION has caused failure of Spent Fuel Cooling systems AND IMMINENT fuel damage is likely</td> <td>A HOSTILE ACTION is occurring or has occurred within the PROTECTED AREA as reported by Security Shift Supervisor</td> <td>A HOSTILE ACTION is occurring or has occurred within the Owner Controlled Area as reported by Security Shift Supervisor OR A validated notification from NRC of an AIRLINER attack threat within 30 min. of the site</td> <td>A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A credible site-specific security threat notification OR A validated notification from NRC providing information of an aircraft threat</td> <td>A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat</td> <td>A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat</td> <td>A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat</td> <td>A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat</td> <td>A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat</td> </tr> </table>				HG4.1	HG4.2	HS4.1	HS4.2	HA4.1	HA4.2	HA4.3	HA4.4	HA4.5	HA4.6	A HOSTILE ACTION has occurred such that plant personnel are unable to operate equipment required to maintain ANY of the following safety function acceptance criteria: • Reactivity control (RC) • Vial Auxiliaries (VA) • RCS pressure and inventory control (PIC) • Core & RCS heat removal (HR)	A HOSTILE ACTION has caused failure of Spent Fuel Cooling systems AND IMMINENT fuel damage is likely	A HOSTILE ACTION is occurring or has occurred within the PROTECTED AREA as reported by Security Shift Supervisor	A HOSTILE ACTION is occurring or has occurred within the Owner Controlled Area as reported by Security Shift Supervisor OR A validated notification from NRC of an AIRLINER attack threat within 30 min. of the site	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A credible site-specific security threat notification OR A validated notification from NRC providing information of an aircraft threat	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat																
HG4.1	HG4.2	HS4.1	HS4.2	HA4.1	HA4.2	HA4.3	HA4.4	HA4.5	HA4.6																															
A HOSTILE ACTION has occurred such that plant personnel are unable to operate equipment required to maintain ANY of the following safety function acceptance criteria: • Reactivity control (RC) • Vial Auxiliaries (VA) • RCS pressure and inventory control (PIC) • Core & RCS heat removal (HR)	A HOSTILE ACTION has caused failure of Spent Fuel Cooling systems AND IMMINENT fuel damage is likely	A HOSTILE ACTION is occurring or has occurred within the PROTECTED AREA as reported by Security Shift Supervisor	A HOSTILE ACTION is occurring or has occurred within the Owner Controlled Area as reported by Security Shift Supervisor OR A validated notification from NRC of an AIRLINER attack threat within 30 min. of the site	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A credible site-specific security threat notification OR A validated notification from NRC providing information of an aircraft threat	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat																															
4 Security	<table border="1"> <tr> <td>HS4.1</td> <td>HS4.2</td> <td>HA4.1</td> <td>HA4.2</td> <td>HA4.3</td> <td>HA4.4</td> <td>HA4.5</td> <td>HA4.6</td> </tr> <tr> <td>A HOSTILE ACTION has occurred such that plant personnel are unable to operate equipment required to maintain ANY of the following safety function acceptance criteria: • Reactivity control (RC) • Vial Auxiliaries (VA) • RCS pressure and inventory control (PIC) • Core & RCS heat removal (HR)</td> <td>A HOSTILE ACTION has caused failure of Spent Fuel Cooling systems AND IMMINENT fuel damage is likely</td> <td>A HOSTILE ACTION is occurring or has occurred within the PROTECTED AREA as reported by Security Shift Supervisor</td> <td>A HOSTILE ACTION is occurring or has occurred within the Owner Controlled Area as reported by Security Shift Supervisor OR A validated notification from NRC of an AIRLINER attack threat within 30 min. of the site</td> <td>A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A credible site-specific security threat notification OR A validated notification from NRC providing information of an aircraft threat</td> <td>A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat</td> <td>A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat</td> <td>A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat</td> </tr> </table>				HS4.1	HS4.2	HA4.1	HA4.2	HA4.3	HA4.4	HA4.5	HA4.6	A HOSTILE ACTION has occurred such that plant personnel are unable to operate equipment required to maintain ANY of the following safety function acceptance criteria: • Reactivity control (RC) • Vial Auxiliaries (VA) • RCS pressure and inventory control (PIC) • Core & RCS heat removal (HR)	A HOSTILE ACTION has caused failure of Spent Fuel Cooling systems AND IMMINENT fuel damage is likely	A HOSTILE ACTION is occurring or has occurred within the PROTECTED AREA as reported by Security Shift Supervisor	A HOSTILE ACTION is occurring or has occurred within the Owner Controlled Area as reported by Security Shift Supervisor OR A validated notification from NRC of an AIRLINER attack threat within 30 min. of the site	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A credible site-specific security threat notification OR A validated notification from NRC providing information of an aircraft threat	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat																				
HS4.1	HS4.2	HA4.1	HA4.2	HA4.3	HA4.4	HA4.5	HA4.6																																	
A HOSTILE ACTION has occurred such that plant personnel are unable to operate equipment required to maintain ANY of the following safety function acceptance criteria: • Reactivity control (RC) • Vial Auxiliaries (VA) • RCS pressure and inventory control (PIC) • Core & RCS heat removal (HR)	A HOSTILE ACTION has caused failure of Spent Fuel Cooling systems AND IMMINENT fuel damage is likely	A HOSTILE ACTION is occurring or has occurred within the PROTECTED AREA as reported by Security Shift Supervisor	A HOSTILE ACTION is occurring or has occurred within the Owner Controlled Area as reported by Security Shift Supervisor OR A validated notification from NRC of an AIRLINER attack threat within 30 min. of the site	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A credible site-specific security threat notification OR A validated notification from NRC providing information of an aircraft threat	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat	A SECURITY CONDITION that does not involve a HOSTILE ACTION as reported by Security Shift Supervisor OR A validated notification from NRC providing information of an aircraft threat																																	
5 Control Room Evacuation	<table border="1"> <tr> <td>HS6.1</td> <td>HS6.2</td> <td>HA6.1</td> <td>HA6.2</td> <td>HA6.3</td> <td>HA6.4</td> <td>HA6.5</td> <td>HA6.6</td> </tr> <tr> <td>None</td> <td>None</td> <td>Control Room evacuation has been initiated AND EITHER: Inability to establish Auxiliary Feedwater to at least one steam generator within 30 min. (Note 4) OR Inability to establish reactor coolant make-up (charging pump flow) within 60 min. (Note 4)</td> <td>Control Room evacuation has been initiated</td> <td>None</td> <td>None</td> <td>None</td> <td>None</td> </tr> </table>				HS6.1	HS6.2	HA6.1	HA6.2	HA6.3	HA6.4	HA6.5	HA6.6	None	None	Control Room evacuation has been initiated AND EITHER: Inability to establish Auxiliary Feedwater to at least one steam generator within 30 min. (Note 4) OR Inability to establish reactor coolant make-up (charging pump flow) within 60 min. (Note 4)	Control Room evacuation has been initiated	None	None	None	None																				
HS6.1	HS6.2	HA6.1	HA6.2	HA6.3	HA6.4	HA6.5	HA6.6																																	
None	None	Control Room evacuation has been initiated AND EITHER: Inability to establish Auxiliary Feedwater to at least one steam generator within 30 min. (Note 4) OR Inability to establish reactor coolant make-up (charging pump flow) within 60 min. (Note 4)	Control Room evacuation has been initiated	None	None	None	None																																	
6 Judgment	<table border="1"> <tr> <td>HS6.1</td> <td>HS6.2</td> <td>HA6.1</td> <td>HA6.2</td> <td>HA6.3</td> <td>HA6.4</td> <td>HA6.5</td> <td>HA6.6</td> </tr> <tr> <td>Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Actual or IMMINENT substantial core degradation or melting with potential for loss of Containment integrity OR HOSTILE ACTION that results in an actual loss of physical control of the facility Releases can be reasonably expected to exceed EPA Protective Action Guideline exposure levels (1,000 mRem TEDE and 5,000 mRem thyroid CDE) offsite for more than the immediate site area.</td> <td>Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Actual or likely major failures of plant functions needed for protection of the public OR HOSTILE ACTION that results in intentional damage or malicious acts; (1) toward site personnel or equipment that could lead to the likely failure of or; (2) that prevent effective access to equipment needed for the protection of the public ANY releases are not expected to result in exposure levels which exceed EPA Protective Action Guideline exposure levels (1,000 mRem TEDE and 5,000 mRem thyroid CDE) beyond the site boundary.</td> <td>Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: An actual or potential substantial degradation of the level of safety of the plant OR A security event that involves probable life threatening risk to site personnel or damage to site equipment because of HOSTILE ACTION ANY releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels (1,000 mRem TEDE and 5,000 mRem thyroid CDE)</td> <td>Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Indicate a potential degradation of the level of safety of the plant OR Indicate a security threat to facility protection has been initiated No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs</td> <td>Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Indicate a potential degradation of the level of safety of the plant OR Indicate a security threat to facility protection has been initiated No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs</td> <td>Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Indicate a potential degradation of the level of safety of the plant OR Indicate a security threat to facility protection has been initiated No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs</td> <td>Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Indicate a potential degradation of the level of safety of the plant OR Indicate a security threat to facility protection has been initiated No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs</td> <td>Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Indicate a potential degradation of the level of safety of the plant OR Indicate a security threat to facility protection has been initiated No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs</td> </tr> </table>				HS6.1	HS6.2	HA6.1	HA6.2	HA6.3	HA6.4	HA6.5	HA6.6	Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Actual or IMMINENT substantial core degradation or melting with potential for loss of Containment integrity OR HOSTILE ACTION that results in an actual loss of physical control of the facility Releases can be reasonably expected to exceed EPA Protective Action Guideline exposure levels (1,000 mRem TEDE and 5,000 mRem thyroid CDE) offsite for more than the immediate site area.	Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Actual or likely major failures of plant functions needed for protection of the public OR HOSTILE ACTION that results in intentional damage or malicious acts; (1) toward site personnel or equipment that could lead to the likely failure of or; (2) that prevent effective access to equipment needed for the protection of the public ANY releases are not expected to result in exposure levels which exceed EPA Protective Action Guideline exposure levels (1,000 mRem TEDE and 5,000 mRem thyroid CDE) beyond the site boundary.	Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: An actual or potential substantial degradation of the level of safety of the plant OR A security event that involves probable life threatening risk to site personnel or damage to site equipment because of HOSTILE ACTION ANY releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels (1,000 mRem TEDE and 5,000 mRem thyroid CDE)	Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Indicate a potential degradation of the level of safety of the plant OR Indicate a security threat to facility protection has been initiated No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs	Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Indicate a potential degradation of the level of safety of the plant OR Indicate a security threat to facility protection has been initiated No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs	Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Indicate a potential degradation of the level of safety of the plant OR Indicate a security threat to facility protection has been initiated No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs	Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Indicate a potential degradation of the level of safety of the plant OR Indicate a security threat to facility protection has been initiated No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs	Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Indicate a potential degradation of the level of safety of the plant OR Indicate a security threat to facility protection has been initiated No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs																				
HS6.1	HS6.2	HA6.1	HA6.2	HA6.3	HA6.4	HA6.5	HA6.6																																	
Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Actual or IMMINENT substantial core degradation or melting with potential for loss of Containment integrity OR HOSTILE ACTION that results in an actual loss of physical control of the facility Releases can be reasonably expected to exceed EPA Protective Action Guideline exposure levels (1,000 mRem TEDE and 5,000 mRem thyroid CDE) offsite for more than the immediate site area.	Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Actual or likely major failures of plant functions needed for protection of the public OR HOSTILE ACTION that results in intentional damage or malicious acts; (1) toward site personnel or equipment that could lead to the likely failure of or; (2) that prevent effective access to equipment needed for the protection of the public ANY releases are not expected to result in exposure levels which exceed EPA Protective Action Guideline exposure levels (1,000 mRem TEDE and 5,000 mRem thyroid CDE) beyond the site boundary.	Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: An actual or potential substantial degradation of the level of safety of the plant OR A security event that involves probable life threatening risk to site personnel or damage to site equipment because of HOSTILE ACTION ANY releases are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels (1,000 mRem TEDE and 5,000 mRem thyroid CDE)	Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Indicate a potential degradation of the level of safety of the plant OR Indicate a security threat to facility protection has been initiated No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs	Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Indicate a potential degradation of the level of safety of the plant OR Indicate a security threat to facility protection has been initiated No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs	Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Indicate a potential degradation of the level of safety of the plant OR Indicate a security threat to facility protection has been initiated No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs	Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Indicate a potential degradation of the level of safety of the plant OR Indicate a security threat to facility protection has been initiated No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs	Other conditions exist which in the judgment of the Emergency Director indicate that events are in progress or have occurred which involve: Indicate a potential degradation of the level of safety of the plant OR Indicate a security threat to facility protection has been initiated No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of safety systems occurs																																	
E ISFSI	<table border="1"> <tr> <td>None</td> <td>None</td> <td>None</td> <td>None</td> <td>EU1.1 Damage to a loaded cask CONFINEMENT BOUNDARY</td> </tr> </table>				None	None	None	None	EU1.1 Damage to a loaded cask CONFINEMENT BOUNDARY																															
None	None	None	None	EU1.1 Damage to a loaded cask CONFINEMENT BOUNDARY																																				

	GENERAL EMERGENCY	SITE AREA EMERGENCY	ALERT	UNUSUAL EVENT																																																
S System Malfunction	1 Loss of AC Power	SG1.1 Loss of all offsite and all onsite AC power, Table S-1, to 4kV vital buses 11(21) and 14(24) AND EITHER: Restoration of at least one 4 kV vital bus within 4 hours is not likely OR CET readings > 700°F	SA1.1 AC power capability to 4kV vital buses 11(21) and 14(24) reduced to a single power source, Table S-1, for ≥ 15 min. (Note 4) AND ANY offsite single power source failure will result in a complete loss of all 4 kV vital bus power	SU1.1 Loss of all offsite AC power, Table S-1, to 4kV vital buses 11(21) and 14(24) for ≥ 15 min. (Note 4)																																																
	2 Loss of DC Power	None	SS2.1 < 105 VDC on all 125 VDC buses (11, 12, 21 and 22) for ≥ 15 min. (Note 4)	None																																																
	3 Criticality & RPS Failure	SG3.1 An automatic reactor trip failed to shut down the reactor as indicated by reactor power > 5% AND All manual actions fail to shut down the reactor as indicated by reactor power > 5% AND ANY of the following exist or have occurred: • CET readings > 700°F • RCS pressure > PORV setpoint • RCS subcooling < 25°F	SS3.1 An automatic reactor trip failed to shut down the reactor as indicated by reactor power > 5% AND Manual actions taken at the Control Room panels do not shut down the reactor as indicated by reactor power > 5%	SA3.1 An automatic reactor trip failed to shut down the reactor AND Manual actions taken at the Control Room panels successfully shut down the reactor as indicated by reactor power ≤ 5%	SU3.1 An UNPLANNED sustained positive startup rate observed on nuclear instrumentation																																															
	4 Inability to Reach Shutdown Conditions	None	None	None	SU4.1 Plant is not brought to required operating mode within Technical Specifications LOO required action completion time																																															
	5 Inst.	Table S-2 Significant Transients • Automatic turbine rundown > 25% thermal power • Electric load rejection > 25% full electrical load • Reactor trip • Safety injection actuation	SS5.1 Loss of greater than approximately 75% of safety system annunciation or indication on Control Room panels for ≥ 15 min. (Note 4) AND A significant transient is in progress, Table S-2	SA5.1 UNPLANNED loss of greater than approximately 75% of safety system annunciation or indication on Control Room panels for ≥ 15 min. (Note 4) AND EITHER: • A significant transient is in progress, Table S-2 OR • Compensatory indications are unavailable (Plant Computer, SPDS)	SU5.1 UNPLANNED loss of greater than approximately 75% of safety system annunciation or indication on Control Room panels for ≥ 15 min. (Note 4)																																															
	6 Comm.	Table S-3 Communications Systems	None	None	SU6.1 Loss of all Table S-3 onsite (internal) communication methods affecting the ability to perform routine operations OR Loss of all Table S-3 offsite (external) communication methods affecting the ability to perform offsite notifications to any agency																																															
	7 Fuel Clad Degradation	None	None	None	SU7.1 Coolant activity > ANY of the following: • Dose equivalent 1-131 0.5 µCi/gm for 100 hrs. continuous • Dose equivalent 1-131 acceptable region of T.S. Fig. 3.4.15-1 • Dose equivalent 1-131 137.5 µCi/gm • Gross activity 100E-bar µCi/gm																																															
	8 RCS Leakage	None	None	None	SU7.2 Shutdown Monitor (RY202-1) high alarm (≥ 1E+06 cpm)																																															
F Fission Product Barrier Degradation	FG1.1 Loss of ANY two barriers AND Loss or potential loss of third barrier (Table F-1)	FS1.1 Loss or potential loss of ANY two barriers (Table F-1)	FA1.1 ANY loss or ANY potential loss of either Fuel Clad or RCS (Table F-1)	FU1.1 ANY loss or ANY potential loss of Containment (Table F-1)																																																
<table border="1"> <caption>Table F-1 Fission Product Barrier Matrix</caption> <thead> <tr> <th rowspan="2"></th> <th colspan="2">Fuel Clad Barrier</th> <th colspan="2">Reactor Coolant System Barrier</th> <th colspan="2">Containment Barrier</th> </tr> <tr> <th>Loss</th> <th>Potential Loss</th> <th>Loss</th> <th>Potential Loss</th> <th>Loss</th> <th>Potential Loss</th> </tr> </thead> <tbody> <tr> <td>A Core Cooling / Heat Removal</td> <td>1. CET readings > 1,200°F</td> <td>1. CET readings > 700°F</td> <td>None</td> <td>1. OTCC flow established 2. RCS heat removal cannot be established AND EITHER: RCS pressure > PORV setpoint OR RCS subcooling < 25°F</td> <td>None</td> <td>1. CET readings cannot be restored < 1,200°F within 15 min. 2. CET readings > 700°F AND Reactor vessel water level cannot be restored > RVLMS 10 in. alarm within 15 min.</td> </tr> <tr> <td>B Inventory</td> <td>None</td> <td>3. RVLMS level < 10 in. alarm</td> <td>1. RCS leak rate > available makeup capacity as indicated by a loss of RCS subcooling (< 25°F) 2. RUPTURED S/G results in an ECCS (SIAS) actuation</td> <td>4. RCS leak rate > 50 gpm with letdown isolated</td> <td>1. A Containment pressure rise followed by a rapid unexplained drop in Containment pressure 2. Containment pressure not consistent with LOCA conditions 3. RUPTURED S/G is also faulted outside of Containment 4. Primary-to-secondary leakage > 10 gpm AND UNSOLUBLE or prolonged steam release from affected S/G to the environment</td> <td>3. Containment pressure rise > 50 psig and rising 4. Containment hydrogen concentration > 4% 5. Containment pressure > 4.25 psig AND cannot meet ANY of the following conditions: • 2 Containment Spray Pumps Operating • 3 CACs Operating • 1 Containment Spray Pump and 2 CACs Operating</td> </tr> <tr> <td>C Radiation / Coolant Activity</td> <td>2. Containment radiation monitor (S17A/B) reading > 3,500 R/hr 3. Post-accident sample dose rate ≥ 40 mRem/hr (1 ft from sample) 4. Coolant activity > 300 µCi/cc DEQ I-131</td> <td>None</td> <td>None</td> <td>None</td> <td>None</td> <td>8. Containment radiation monitor (S17A/B) reading > 14,000 R/hr</td> </tr> <tr> <td>D Isolation Status</td> <td>None</td> <td>None</td> <td>None</td> <td>None</td> <td>None</td> <td>None</td> </tr> <tr> <td>E Judgment</td> <td>5. ANY condition in the opinion of the Emergency Director that indicates potential loss of the fuel clad barrier</td> <td>4. ANY condition in the opinion of the Emergency Director that indicates potential loss of the fuel clad barrier</td> <td>4. ANY condition in the opinion of the Emergency Director that indicates potential loss of the RCS barrier</td> <td>5. ANY condition in the opinion of the Emergency Director that indicates potential loss of the RCS barrier</td> <td>6. ANY condition in the opinion of the Emergency Director that indicates potential loss of the Containment barrier</td> <td>7. ANY condition in the opinion of the Emergency Director that indicates potential loss of the Containment barrier</td> </tr> </tbody> </table>						Fuel Clad Barrier		Reactor Coolant System Barrier		Containment Barrier		Loss	Potential Loss	Loss	Potential Loss	Loss	Potential Loss	A Core Cooling / Heat Removal	1. CET readings > 1,200°F	1. CET readings > 700°F	None	1. OTCC flow established 2. RCS heat removal cannot be established AND EITHER: RCS pressure > PORV setpoint OR RCS subcooling < 25°F	None	1. CET readings cannot be restored < 1,200°F within 15 min. 2. CET readings > 700°F AND Reactor vessel water level cannot be restored > RVLMS 10 in. alarm within 15 min.	B Inventory	None	3. RVLMS level < 10 in. alarm	1. RCS leak rate > available makeup capacity as indicated by a loss of RCS subcooling (< 25°F) 2. RUPTURED S/G results in an ECCS (SIAS) actuation	4. RCS leak rate > 50 gpm with letdown isolated	1. A Containment pressure rise followed by a rapid unexplained drop in Containment pressure 2. Containment pressure not consistent with LOCA conditions 3. RUPTURED S/G is also faulted outside of Containment 4. Primary-to-secondary leakage > 10 gpm AND UNSOLUBLE or prolonged steam release from affected S/G to the environment	3. Containment pressure rise > 50 psig and rising 4. Containment hydrogen concentration > 4% 5. Containment pressure > 4.25 psig AND cannot meet ANY of the following conditions: • 2 Containment Spray Pumps Operating • 3 CACs Operating • 1 Containment Spray Pump and 2 CACs Operating	C Radiation / Coolant Activity	2. Containment radiation monitor (S17A/B) reading > 3,500 R/hr 3. Post-accident sample dose rate ≥ 40 mRem/hr (1 ft from sample) 4. Coolant activity > 300 µCi/cc DEQ I-131	None	None	None	None	8. Containment radiation monitor (S17A/B) reading > 14,000 R/hr	D Isolation Status	None	None	None	None	None	None	E Judgment	5. ANY condition in the opinion of the Emergency Director that indicates potential loss of the fuel clad barrier	4. ANY condition in the opinion of the Emergency Director that indicates potential loss of the fuel clad barrier	4. ANY condition in the opinion of the Emergency Director that indicates potential loss of the RCS barrier	5. ANY condition in the opinion of the Emergency Director that indicates potential loss of the RCS barrier	6. ANY condition in the opinion of the Emergency Director that indicates potential loss of the Containment barrier	7. ANY condition in the opinion of the Emergency Director that indicates potential loss of the Containment barrier
	Fuel Clad Barrier		Reactor Coolant System Barrier			Containment Barrier																																														
	Loss	Potential Loss	Loss	Potential Loss	Loss	Potential Loss																																														
A Core Cooling / Heat Removal	1. CET readings > 1,200°F	1. CET readings > 700°F	None	1. OTCC flow established 2. RCS heat removal cannot be established AND EITHER: RCS pressure > PORV setpoint OR RCS subcooling < 25°F	None	1. CET readings cannot be restored < 1,200°F within 15 min. 2. CET readings > 700°F AND Reactor vessel water level cannot be restored > RVLMS 10 in. alarm within 15 min.																																														
B Inventory	None	3. RVLMS level < 10 in. alarm	1. RCS leak rate > available makeup capacity as indicated by a loss of RCS subcooling (< 25°F) 2. RUPTURED S/G results in an ECCS (SIAS) actuation	4. RCS leak rate > 50 gpm with letdown isolated	1. A Containment pressure rise followed by a rapid unexplained drop in Containment pressure 2. Containment pressure not consistent with LOCA conditions 3. RUPTURED S/G is also faulted outside of Containment 4. Primary-to-secondary leakage > 10 gpm AND UNSOLUBLE or prolonged steam release from affected S/G to the environment	3. Containment pressure rise > 50 psig and rising 4. Containment hydrogen concentration > 4% 5. Containment pressure > 4.25 psig AND cannot meet ANY of the following conditions: • 2 Containment Spray Pumps Operating • 3 CACs Operating • 1 Containment Spray Pump and 2 CACs Operating																																														
C Radiation / Coolant Activity	2. Containment radiation monitor (S17A/B) reading > 3,500 R/hr 3. Post-accident sample dose rate ≥ 40 mRem/hr (1 ft from sample) 4. Coolant activity > 300 µCi/cc DEQ I-131	None	None	None	None	8. Containment radiation monitor (S17A/B) reading > 14,000 R/hr																																														
D Isolation Status	None	None	None	None	None	None																																														
E Judgment	5. ANY condition in the opinion of the Emergency Director that indicates potential loss of the fuel clad barrier	4. ANY condition in the opinion of the Emergency Director that indicates potential loss of the fuel clad barrier	4. ANY condition in the opinion of the Emergency Director that indicates potential loss of the RCS barrier	5. ANY condition in the opinion of the Emergency Director that indicates potential loss of the RCS barrier	6. ANY condition in the opinion of the Emergency Director that indicates potential loss of the Containment barrier	7. ANY condition in the opinion of the Emergency Director that indicates potential loss of the Containment barrier																																														
<table border="1"> <tr> <td>Current Classification</td> <td>EAL#:</td> <td>Time Declared:</td> </tr> <tr> <td><input type="checkbox"/> UNUSUAL EVENT</td> <td></td> <td></td> </tr> <tr> <td><input type="checkbox"/> ALERT</td> <td></td> <td></td> </tr> <tr> <td><input type="checkbox"/> SITE AREA EMERGENCY</td> <td></td> <td></td> </tr> <tr> <td><input type="checkbox"/> GENERAL EMERGENCY</td> <td></td> <td></td> </tr> </table>					Current Classification	EAL#:	Time Declared:	<input type="checkbox"/> UNUSUAL EVENT			<input type="checkbox"/> ALERT			<input type="checkbox"/> SITE AREA EMERGENCY			<input type="checkbox"/> GENERAL EMERGENCY																																			
Current Classification	EAL#:	Time Declared:																																																		
<input type="checkbox"/> UNUSUAL EVENT																																																				
<input type="checkbox"/> ALERT																																																				
<input type="checkbox"/> SITE AREA EMERGENCY																																																				
<input type="checkbox"/> GENERAL EMERGENCY																																																				
<table border="1"> <tr> <td colspan="2">EAL Identifier</td> </tr> <tr> <td>XXX.X</td> <td></td> </tr> <tr> <td>Category (R, H, E, S, F, C)</td> <td>Sequential number within subcategory/classification</td> </tr> <tr> <td>Emergency classification (G, S, A, U)</td> <td>Subcategory number (1 if no subcategory)</td> </tr> </table>					EAL Identifier		XXX.X		Category (R, H, E, S, F, C)	Sequential number within subcategory/classification	Emergency classification (G, S, A, U)	Subcategory number (1 if no subcategory)																																								
EAL Identifier																																																				
XXX.X																																																				
Category (R, H, E, S, F, C)	Sequential number within subcategory/classification																																																			
Emergency classification (G, S, A, U)	Subcategory number (1 if no subcategory)																																																			

Modes: 1 Power Operation, 2 Startup, 3 Hot Standby, 4 Hot Shutdown, 5 Cold Shutdown, 6 Refuel, D Defueled



MODE 1, 2, 3 or 4

D-01

DOI