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10 CFR 50.73

March 7, 2011  
BW110023

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, DC 20555-0001

Braidwood Station, Unit 2  
Facility Operating License No. NPF-77  
NRC Docket No. STN 50-457

**Subject:** Retraction of Licensee Event Report 2010-002-00 – Containment Spray Pump Suction Valve Failed to Close Resulting in an Unanalyzed Plant Condition due to Procedure Error

**Reference:** Letter from A. Shahkarami (Exelon Generation Company, LLC) to U.S. NRC, "Licensee Event Report 2010-002-00 - Containment Spray Pump Suction Valve Failed to Close Resulting in an Unanalyzed Plant Condition due to Procedure Error" dated August 16, 2010

In Reference 1, Exelon Generation Company, LLC (EGC) submitted Braidwood Station Licensee Event Report (LER) 2010-002-00, "Containment Spray Pump Suction Valve Failed to Close Resulting in an Unanalyzed Plant Condition due to Procedure Error." The LER was submitted in accordance with 10 CFR 50.73, "Licensee event report system," paragraph (a)(2)(ii)(B), as an unanalyzed condition.

The basis for reportability as described in LER 2010-002-00 was a June 17, 2010 event where a containment spray pump suction valve failed to return to its normally closed position following surveillance testing. With the containment spray pump suction valve unable to close from the main control room, an unanalyzed condition may have existed where, during a large break loss of coolant accident requiring cold leg recirculation, the refueling water storage tank (RWST) had an additional flow path to the recirculation sump. This potentially challenged the operators to complete the switchover prior to the RWST reaching the point at which pumps taking a suction from the RWST only are shutdown.

While this condition was being evaluated, an Event Notification System (ENS) notification was made per ENS 46021 under 10CFR50.72(b)(3)(ii)(B). LER 2010-002-00 was issued in accordance with 10 CFR 50.73(a)(2)(ii)(B), with the expectation that supplement to the report would be submitted following completion of the evaluation.

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The results of the evaluation concluded the operators would have performed the switchover steps within the allowed time, before reaching the RWST empty alarm setpoint. Therefore, the emergency core cooling system (ECCS) and containment spray system would have performed their design functions. The evaluation also determined the RWST outflows with 2CS009B in the open position during the ECCS switchover sequence did not affect the RWST vortex analysis. Based on no loss of design function, the plant was not in an unanalyzed condition. Therefore, the event is not reportable under 10 CFR 50.73 and LER 2010-002-00 is being retracted. The associated 10 CFR 50.72 Event Notification, i.e., ENS notification 46021 was retracted on February 1, 2011.

Should you have any questions concerning this submittal, please contact Mr. Terence Schuster, Regulatory Assurance Manager, at (815) 417-2800.

Respectfully,



Daniel J. Enright  
Site Vice President  
Braidwood Station

cc: NRR Project Manager – Braidwood Station  
Illinois Emergency Management Agency – Division of Nuclear Safety  
US NRC Regional Administrator, Region III  
US NRC Senior Resident Inspector (Braidwood Station)