Potential New Generic License Conditions

Proposed License Condition	SER Section	Description
3-2	3.7.2.5	Prior to initial fuel load, the seismic interaction review will be updated by the licensee for as-built information. This review is performed in parallel with the seismic margin evaluation. The review is based on as-procured data, as well as the as-constructed condition.
4-1	4.0	Prior to initial fuel load, the instrumentation uncertainties of the actual plant operating instrumentation will be calculated to confirm that either the design limit departure from nucleate boiling ratio (DNBR) values remain valid or that the safety analysis minimum DNBR bounds the new design limit DNBR values plus DNBR penalties, such as rod bow penalty. (PWR-specific generic condition)
5-4	5.3.3.5	Prior to initial fuel load, the licensee shall update the pressure temperature (P-T) limits using the pressure temperature limits report (PTLR) methodologies approved in the [*insert referenced certified design (e.g., AP1000 DCD)*] using the plant-specific material properties or confirm that the RV material properties meet the specifications and use the [*insert certified design supplier (e.g., Westinghouse)*] generic PTLR curves.
5-5	5.3.4.5	The licensee shall complete, prior to fuel load, verification of plant-specific belt line material properties consistent with the requirements in FSAR [*insert applicable FSAR reference(s)*]. The verification will include a pressurized thermal shock (PTS) evaluation based on as-procured RV material data and the projected neutron fluence for the life of the plant design. This evaluation report will be submitted for an NRC staff confirmatory review at least 18 months prior to initial fuel load. (PWR-specific generic condition)
9-1	9.1.2.5	The licensee shall submit to the Director of the Office of New Reactors, or the Director's designee, a schedule, no later than 12 months after issuance of the COL, that supports planning for and conduct of NRC inspections of the spent fuel rack [*neutron absorbing material*] coupon monitoring program implementation. The schedule shall be updated every 6 months until 12 months before scheduled fuel loading, and every month thereafter until either the spent fuel rack [*neutron absorbing material*] coupon monitoring program has been fully implemented or the plant has been placed in commercial service, whichever comes first. This schedule shall address spent fuel rack [*neutron absorbing material*] coupon monitoring program implementation prior to the spent fuel storage system being considered operational (i.e., filled and ready to receive spent fuel).

DRAFT

Proposed License Condition	SER Section	Description
10-1	10.1.5	Prior to initial fuel load, the licensee shall implement a flow accelerated corrosion (FAC) program. The licensee shall submit to the Director of the Office of New Reactors, or the Director's designee, a schedule, no later than 12 months after issuance of the COL, that supports planning for and conduct of NRC inspections of the FAC program implementation. The schedule shall be updated every 6 months until 12 months before scheduled fuel load, and every month thereafter until either the FAC program has been fully implemented or the plant has been placed in commercial service, whichever comes first.
13-3	13.3.5	The licensee shall submit a fully developed set of plant-specific emergency action levels (EALs) for [*specify reactor unit(s)*] in accordance with [*specify applicable EAL methodology (e.g., Nuclear Energy Institute (NEI) 07-01)*], with no deviations. The EALs shall have been discussed and agreed upon with State and local officials. These fully developed EALs shall be submitted to the NRC for confirmation at least 180 days prior to initial fuel load.
15-1	15.0.5	The licensee shall submit to the Director of the Office of New Reactors, or the Director's designee, a schedule, no later than 12 months after issuance of the COL, that supports planning for and conduct of NRC inspections. The schedule shall be updated every 6 months until 12 months before scheduled fuel load, and every month thereafter until the license condition has been fully implemented or the plant has been placed in commercial service, whichever comes first. This schedule shall address: • The availability of documented instrumentation uncertainties to calculate a power calorimetric uncertainty (prior to initial fuel load). • The availability of administrative controls to implement maintenance and contingency activities related to the power calorimetric uncertainty instrumentation (prior to initial fuel load). (AP1000-specific?) (specific to applicants seeking to go from 2% to 1% accuracy)
19-1	19.59.5	Prior to initial fuel load, the licensee shall perform a seismic walkdown to confirm design assumptions and to identify differences between the as-built plant and the licensed design. The licensee shall assess the as-built condition of each structure, system, and component (SSC) credited in the PRA-based seismic margin analysis (SMA). The licensee shall confirm that for the plant and each SSC, there is high confidence in low probability of failure (HCLPF) with adequate margin. The licensee shall evaluate differences between the as-built plant and the design

DRAFT

Proposed License Condition	SER Section	Description
		used as the basis for the SMA, updating the analysis as necessary to account for the as-built condition and to address seismically induced fires and flooding.
19-2	19.59.5	Prior to initial fuel load, the licensee shall evaluate design changes and perform a plant walkdown to determine differences between the as-built plant and the licensed design as modeled in the plant-specific probabilistic risk assessment (PRA). The licensee shall update the plant-specific PRA accordingly. The licensee shall upgrade the plant-specific PRA to incorporate all PRA-related consensus standards that have been endorsed by the NRC staff one year or more before the scheduled date for loading fuel.
19-3	19.59.5	Prior to initial fuel load, the licensee shall evaluate design changes and perform a plant walkdown to confirm design assumptions and determine differences between the as-built plant and the licensed design as modeled in the plant-specific internal fire analysis and the plant-specific internal flooding analysis. The licensee shall perform or modify the fire and flooding PRAs, as necessary, to account for the as-built condition.
19-4	19.59.5	Prior to initial fuel load, the licensee shall implement site-specific severe accident management guidelines (SAMG). To support NRC inspections of this activity, the licensee shall submit its SAMG implementation schedule to the Director of the Office of New Reactors within one year from the date the COL is issued. The licensee shall update the schedule every 6 months until 12 months prior to the scheduled date of initial fuel load. Thereafter, the licensee shall update the schedule every month until the site-specific SAMG have been fully implemented.

DRAFT