

FINAL SAFETY ANALYSIS REPORT

Chapter 1 Introduction and General Description of Plant

1.1 Introduction

This section of the ESBWR Design Control Document (DCD), i.e., the referenced DCD, is incorporated by reference with the following departures and/or supplements.

1.1.1 Format and Content

EF3 SUP 1.1-1

1.1.1.1 10 CFR 52 and Regulatory Guide 1.206

This FSAR was developed to comply with the content requirements of 10 CFR 52.79, and to the extent feasible, the content and format requirements contained in Regulatory Guide (RG) 1.206, “Combined License Applications for Nuclear Power Plants (LWR Edition).” See [Table 1.9-203](#), Conformance With the FSAR Content Guidance In RG 1.206. If the information requested by RG 1.206 is not needed (e.g., because it is already provided in the DCD or is located elsewhere in the FSAR), the table specifies the location of the information.

Section C.III.6 of RG 1.206 addresses referencing a design certification (DC) application rather than a certified design. The existing DC rules (10 CFR 52 appendices) require that a Combined License Application (COLA) that references a certified design include a plant-specific DCD containing the same type of information and using the same organization and numbering as the generic DCD for the ESBWR design, as modified and supplemented by the applicant’s exemptions and departures. Where necessary to present additional information, new sections were added following the logical structure of the ESBWR generic DCD.

1.1.1.2 Standard Review Plan

As required by 10 CFR 52.79(a)(41), an evaluation of the facility for conformance with the acceptance criteria contained in NUREG-0800, “Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants LWR Edition,” in effect six months prior to submittal of the COLA, was performed. This evaluation determined that this FSAR contains no unacceptable deviations from the acceptance criteria given in the applicable portions of the SRP. Where necessary, [Table 1.9-201](#), Conformance with Standard Review Plan, provides a summary of any differences from the SRP acceptance criteria, along with a justification for

an exception to a criterion or a Branch Technical Position (BTP); or the table identifies the applicable FSAR section(s) that addresses a difference.

1.1.1.3 Tables and Figures

Tabulations of data are designated “tables.” Each is identified by the section number followed by a number (for example, Table 1.9-204 would be an FSAR table in Section 1.9) The use of the “200” series for FSAR table numbers distinguishes FSAR tables from DCD tables. If a table from the DCD is referenced in the FSAR text, it is denoted as such, for example “DCD Table 4.1-1.” If a table from the DCD was revised for use in the FSAR, the original DCD table number is appended with an “R;” for example, if “DCD Table 4.2-1” was revised, it would have become “Table 4.2-1R.” Tables are located at the end of the section immediately following the text.

Drawings, pictures, sketches, curves, graphs, and engineering diagrams identified as “figures” are numbered using the section number followed by a number (for example, Figure 2.1-201 would be an FSAR figure in Section 2.1). The use of the “200” series for FSAR figure numbers distinguishes FSAR figures from DCD figures. If a figure from the DCD is referenced in the FSAR text, it is denoted as such; for example “DCD Figure 4.1-1.” If a figure from the DCD was revised for use in the FSAR, the original DCD figure number was appended with an “R;” for example, if “DCD Figure 4.2-1” was revised, it would have become “Figure 4.2-1R.” Figures are located at the end of the applicable section following the tables.

1.1.1.4 Numbering of Pages

Text pages are numbered sequentially within each chapter (for example, Page 1-4 is the fourth page of Chapter 1).

1.1.1.5 Proprietary and Security-Related Sensitive Unclassified Non-Safeguards Information (SUNSI)

Proprietary information and SUNSI¹ is withheld from public disclosure and therefore not included in the public version of the FSAR. SUNSI included in the non-public version of the FSAR is appropriately indicated.

1.1.1.6 Acronyms

In addition to the summary list of acronyms in the FSAR frontmatter, acronyms are defined at their first occurrence in FSAR text.

1.1.1.7 Incorporation by Reference

10 CFR 52.79 states in part that, “The final safety analysis report need not contain information or analyses submitted to the Commission in connection with the design certification, provided, however, that the final safety analysis report must either include or incorporate by reference the standard design certification final safety analysis report and must contain, in addition to the information and analyses otherwise required, information sufficient to demonstrate that the site characteristics fall within the site parameters specified in the design certification.” Therefore, because this COLA references the ESBWR DC application, this FSAR incorporates the ESBWR DCD by reference, with the departures presented in COLA Part 7, and with supplemental information, as appropriate (see [Subsection 1.1.1.10](#)). References in this FSAR to the DCD should be understood to mean the ESBWR DCD, Tier 2, submitted by GE-Hitachi Nuclear Energy Americas LLC (GEH), as Revision 9.

1.1.1.8 Departures from the Standard Design Certification (or Application)

A departure is a plant-specific “deviation” from design information in a standard DC rule or, consistent with Section C.III.6 of RG 1.206, from design information in a DC application.

10 CFR 52 clarifies that Tier 2 information in a standard DC rule does not include conceptual design information (CDI) and per Section C.III.6 of RG 1.206, Tier 2 information in a standard DC application does not include CDI. Therefore, replacement or revision of CDI does not constitute a departure. Additionally, information addressing combined license (COL) information/holder items and supplemental information (see [Subsection 1.1.1.10](#)) that does not change the intent or meaning of the ESBWR DCD text is not considered a departure from the ESBWR DCD.

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1. Any information which, if lost, misused, modified, or accessed without authorization, can reasonably be foreseen as causing harm to the public interest, the commercial or financial interest of the entity or individual to whom the information pertains, the conduct of NRC and Federal programs, or the personal privacy of individuals. SUNSI has been organized into the following seven groups:
 - Allegation information
 - Investigation information
 - Security-related information
 - Proprietary information
 - Privacy Act information
 - Federal, State, Foreign Government, and international agency information
 - Sensitive internal information

EF3 SUP 1.1-2

1.1.1.9 Obligations and Commitments

NEI 99-04, "Guidelines For Managing NRC Commitment Changes," provides the following definitions:

Obligation refers to any condition or action that is a legally binding requirement imposed on licensees through applicable rules, regulations, orders, and licenses (including technical specifications and license conditions). These conditions (also referred to as regulatory requirements) generally require formal NRC approval as part of the change-control process. Also included in the category of obligations are those regulations and license conditions that define change-control processes and reporting requirements for licensing basis documents such as the updated FSAR, quality assurance program, emergency plan, security plan, fire protection program, etc.

Regulatory Commitment means an explicit statement to take a specific action agreed to, or volunteered by, a licensee and submitted in writing on the docket to the NRC.

For purposes of the Fermi 3 FSAR, the information provided to meet the content requirement of 10 CFR 52.79, including implicit implementation activities, are treated as Obligations. Statements related to the future development of information that is not currently available (e.g. COL Holder Items) are treated as commitments. Commitments are identified with the designator **[START COM X.Y-00#]** at the beginning of the commitment text, and **[END COM X.Y-00#]** at the end of the text.

1.1.1.10 Supplements

Supplements fall into one of the following categories (see [Table 1.1-201](#) for definitions of categories unless noted otherwise):

- COL Item
- Conceptual Design Information
- Supplemental Information (see definition below)

Supplemental information is FSAR information that includes information not related to COL Items, departures, or conceptual design (see [Table 1.1-201](#) for definition of terms); or is information to demonstrate that the design of the facility falls within the site characteristics and design parameters specified in the DCD.

1.1.1.11 Left Margin Annotations

FSAR sections are annotated in the left margin with information that identifies: 1) the reason the information is being provided and, as applicable, 2) whether the information is standard (identical) for any ESBWR application, or specific to the COLA for a particular plant.

The annotations and their definitions are listed in [Table 1.1-201](#).

1.1.1.12 Tense

Because this FSAR is a licensing basis document that will control plant design and operations after the COL is issued, the FSAR is generally written in the present tense. Thus, plant design and configuration are described in the present tense although the plant is not yet built. Similarly, programs, procedures, and organizational matters are generally described in the present tense although such descriptions may not yet be implemented. Accordingly, the use of the present tense in this FSAR should be understood as describing the plant, programs and procedures, and organization as they will exist when in place, and not as a representation that they are already in place.

1.1.2 General Description

1.1.2.1 ESBWR Standard Plant Scope

Replace the last sentence with the following.

EF3 CDI

The orientation of the principal plant structures for Unit 3 is shown in [Figure 2.1-204](#).

Add the following at the end of this section.

EF3 SUP 1.1-2

The ESBWR standard plant scope is discussed in DCD Section 1.1.2.1. In addition to the buildings and structures within the scope of the ESBWR standard plant, the plant includes an intake structure for plant makeup water, normal power heat sink and auxiliary heat sink cooling towers, storage tanks for water and fuel oil, a switchyard, and other site support systems and structures necessary to support the operation and maintenance of the facility.

1.1.2.2 **Type of License Request**

Add the following at the end of this section.

EF3 SUP 1.1-3

This application by the Detroit Edison Company is for a combined construction permit and operating license, i.e., COL under Section 103 of the Atomic Energy Act, for the third nuclear power plant to be located on the existing Enrico Fermi Atomic Power Plant (Fermi) site in Monroe County, Michigan. This COLA references a DC application for an ESBWR (consistent with Section C.III.6 of RG 1.206) for the Fermi site. The third unit is designated Enrico Fermi Unit 3 (Fermi 3).

1.1.2.4 **Description of Location**

Add the following at the end of this section.

EF3 SUP 1.1-4

The Fermi 3 site is located on the shore of the western end of Lake Erie, at Lagoona Beach in Frenchtown Township, Monroe County, Michigan. The site is approximately 13 km (8 miles) northeast of Monroe, Michigan, 48 km (30 miles) southwest of downtown Detroit, Michigan, and 25 miles northeast of Toledo, Ohio. Reactor centerline coordinates are latitude 41°57'39"N and longitude 83°15'43"W. The site consists of approximately 1260 acres on the same site is Fermi 1 and 2. Fermi 1 was originally a fast breeder reactor, and later also a conventional oil-fired power plant. Both are decommissioned. Fermi 2 is a General Electric boiling water reactor (BWR), rated at 3430 MWt. [Figure 2.1-204](#) shows the Fermi 3 site plan. [Figure 2.1-201](#) through [Figure 2.1-203](#) show various relationships of the Fermi 3 site to the surrounding vicinity.

Transportation facilities are readily available. Interstate Highways 75 and 275 are approximately 6 km (4 miles) west of the site. More immediate access to the site is available from the Dixie Highway, which runs north and south approximately 3 km (2 miles) west of the site. From the Dixie Highway, Enrico Fermi Drive (a paved private access road) enters the site on the western boundary where it serves as the main entrance. Rail service to the site is furnished by a spur line from the main line which is approximately 6 km (4 miles) west of the site.

1.1.2.7 **Rated Core Thermal Power**

Replace the last four sentences of this section with the following.

EF3 COL 1.1-1-A

Unit 3 operates at an estimated gross electrical power output at rated power of approximately 1594 MWe (as shown in DCD Section 10.1). The estimated net electrical power output, which is dependent on site ambient conditions, the normal plant heat sink (NPHS) operation controls, and station electrical loads, is between approximately 1485 MWe and 1585 MWe.

EF3 SUP 1.1-5

1.1.2.8 **Schedule**

Key milestones associated with the estimated schedule for the earliest completion of construction and the beginning of commercial operation are as follows.

| Milestone | Estimated Earliest Schedule Date |
|---------------------------|----------------------------------|
| First Structural Concrete | December 2013 |
| Initiate Startup Testing | December 2018 |
| Fuel Load | June 2019 |
| Commercial Operation | June 2020 |

1.1.3 **COL Unit-Specific Information**

1.1-1-A **Establish Rated Electrical Output**

EF3 COL 1.1-1-A

This COL Item is addressed in [Subsection 1.1.2.7](#).

Table 1.1-201 Left Margin Annotations (Sheet 1 of 2)

[EF3 SUP 1.1-1]

| FSAR Component | Margin Annotation | Definition and Use |
|--|--|---|
| Standard Departure | STD DEP X.Y.Z -# | FSAR information that departs from the generic DCD and is common for all parallel applicants; i.e., the departure and discussion of the departure are identical for all applicants of the ESBWR technology. Each Standard Departure is numbered based on the applicable section down to the X.Y.Z level, e.g.: STD DEP 9.2-1, or STD DEP 9.2.1-1. |
| Plant-Specific Departure | (PLANT) DEP X.Y.Z-# | FSAR information that departs from the generic DCD and is plant-specific; i.e., the departure and discussion of the departure are not identical for all applicants of the ESBWR technology. Each Plant-Specific Departure is numbered based on the applicable section down to the X.Y.Z level, e.g.: EF3 DEP 9.2-1, or EF3 DEP 9.2.1-1. |
| Standard COL Item | STD COL X.Y-#-A | FSAR information that addresses a DCD COL Item that is common for all parallel applicants; i.e., the response to and discussion of the DCD COL Item are identical for all applicants of the ESBWR technology. Each Standard COL Item is numbered as identified in ESBWR DCD Table 1.10-1. The -A refers to a COL Applicant item. |
| Plant-Specific COL Item | (PLANT) COL X.Y-#-A or (PLANT) COL X.Y-#-H | FSAR information that addresses a DCD COL Item that is plant-specific; i.e., the response to the COL Item is not a Standard COL Item for parallel applicants. Each Plant-Specific COL Item is numbered as identified in the ESBWR DCD (see STD COL above). |
| Standard Conceptual Design Information | STD CDI | A Conceptual Design Information designation is used to identify FSAR information that replaces Conceptual Design Information in the DCD, in whole or in part. Replacement and supplemental Conceptual Design Information is generally plant-specific; however, for conceptual design that is generic for all applications the annotation for standard (STD) is used, STD CDI. |
| Plant Specific Conceptual Design Information | (PLANT) CDI | A Conceptual Design Information designation is used to identify FSAR information that replaces Conceptual Design Information in the DCD, in whole or in part. Plant specific replacement and supplemental Conceptual Design Information uses the annotation (PLANT) CDI, e.g., EF3 CDI. |
| Standard Supplemental Information | STD SUP X.Y-# | Supplemental FSAR information that is identical for all parallel applicants; i.e., the supplemental information is identical for all applicants of the ESBWR technology. Each Standard Supplemental Information designation is numbered based on applicable section down to the X.Y level, e.g., STD SUP 10.4-1. |

Table 1.1-201 Left Margin Annotations (Sheet 2 of 2)

[EF3 SUP 1.1-1]

| FSAR Component | Margin Annotation | Definition and Use |
|---|--------------------------|--|
| Plant-Specific Supplemental Information | (PLANT) SUP X.Y-# | Supplemental FSAR information that is plant specific (not standard). Each Plant Specific Supplemental Information designation is numbered based on applicable section down to the X.Y level, e.g., EF3 SUP 10.4-1. |

1.2 General Plant Description

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

1.2.2.10.2 Solid Waste Management System

Delete the first sentence of the seventh paragraph and replace the first sentence of the seventh paragraph with the following.

EF3 DEP 11.4-1

The Radwaste Building is configured to accommodate at least 10 years of packaged Class B and C waste and approximately three months of packaged Class A waste considering routine operations and anticipated operational occurrences.

1.2.2.11.4 Main Turbine

Delete the second sentence of the first paragraph and replace the first sentence of the first paragraph with the following.

STD CDI

The main turbine has one high-pressure (HP) turbine and three low-pressure (LP) turbines.

1.2.2.11.7 Main Condenser

Delete the second sentence of the third paragraph and replace the first sentence of the third paragraph with the following.

STD CDI

The main condenser is a multi-pressure, triple-shell unit.

1.2.2.12.13 Hydrogen Water Chemistry System

Replace this section with the following.

STD CDI

The Hydrogen Water Chemistry System (HWCS) consists of hydrogen and oxygen supply systems to inject hydrogen in the feedwater and oxygen in the offgas, plus monitoring systems to track the effectiveness of the system.

| | |
|-----------------------|--|
| | 1.2.2.12.15 Zinc Injection System |
| | Replace this section with the following. |
| STD CDI | The Zinc Injection System is not utilized. |
| | 1.2.2.12.16 Freeze Protection |
| | Replace this section with the following. |
| STD CDI | Freeze protection is incorporated at the individual system level using insulation and heat tracing for all external tanks and piping that may freeze during winter weather. |
| | 1.2.2.16.9 Radwaste Building |
| EF3 DEP 11.4-1 | Replace Figure 1.2-21 to Figure 1.2-25 with Figure 1.2-21R to Figure 1.2-25R in the parenthesis in the first sentence. |
| | 1.2.2.16.10 Other Building Structures |
| | Replace the fifth paragraph with the following. |
| EF3 CDI | Other facilities include the Service Building, Water Treatment Building, Administration Building, Training Center, Sewage Discharge System, warehouse, and hot and cold machine shop. These are all of conventional size and design, and in some cases may be shared with other units at the same site. |
| STD SUP 1.2-1 | 1.2.2.19 Modular Construction Techniques and Plans [START COM 1.2-001] To the extent practical, modular construction techniques that have been applied during ABWR construction projects will be adapted and/or modified for use during ESBWR construction. Modularization reviews will be performed to develop a plan for bringing the ABWR experience into the ESBWR. Once completed, the results of the modularization reviews will be used as guidance to develop the detailed design of the areas affected by modularization. [END COM 1.2-001] |

Figure 1.2-21R Radwaste Building Plan At Elevation –9350 [EF3 DEP 11.4-1]

{{{Security-Related Information – Withheld Under 10 CFR 2.390}}}

See Fermi 3 COLA Part 9

Figure 1.2-22R Radwaste Building Plan At Elevation –2350 [EF3 DEP 11.4-1]

{{{Security-Related Information – Withheld Under 10 CFR 2.390}}}

See Fermi 3 COLA Part 9

Figure 1.2-23R Radwaste Building Plan At Elevation 4650 [EF3 DEP 11.4-1]

{{{Security-Related Information – Withheld Under 10 CFR 2.390}}}

See Fermi 3 COLA Part 9

Figure 1.2-24R Radwaste Building Plan At Elevation 10650 [EF3 DEP 11.4-1]

{{{Security-Related Information – Withheld Under 10 CFR 2.390}}}

See Fermi 3 COLA Part 9

Figure 1.2-25R

**Radwaste Building Plan At Elevation Section A-A
And Section B-B**

[EF3 DEP 11.4-1]

{{{Security-Related Information – Withheld Under 10 CFR 2.390}}}

See Fermi 3 COLA Part 9

1.3 Comparison Tables

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

Add the following at the end of this section.

EF3 COL 1.3-1-A

There are no updates to DCD Table 1.3-1 based on unit specific information.

1.3.1 COL Information

1.3-1-A Update Table 1.3-1

This COL item is addressed in [Section 1.3](#).

1.4 Identification of Agents and Contractors

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

EF3 SUP 1.4-1

1.4.1 Detroit Edison Company

Detroit Edison is the applicant for the COL, and Detroit Edison will be the licensee authorized to construct and operate Fermi 3. Detroit Edison is therefore responsible for making each of the key project decisions, including the ultimate decision on whether to build a new nuclear power plant, and would be the plant operator.

Detroit Edison has selected GE-Hitachi Nuclear Energy Americas, LLC (GEH) as the reactor technology vendor for the design of the unit. **[START COM 1.4-001]** The primary contractor for site engineering has not been selected at the time of COLA submittal; this information will be supplied in an FSAR update following selection. **[END COM 1.4-001]** Detroit Edison has responsibility for the operation of the unit. The following sections provide information on the experience and qualifications of the aforementioned agents and contractors as well as the division of responsibility between Detroit Edison and its agents and contractors.

1.4.2 GE-Hitachi Nuclear Energy Americas, LLC (GEH)

GEH is responsible for developing the complete standard plant for the ESBWR necessary to obtain a DC from the NRC, supporting preparation

of the COL application, and activities to support deployment of the ESBWR on the Fermi site. GEH, established in June 2007, is a business alliance of GE and Hitachi's respective nuclear businesses, established to serve the global nuclear industry.

DCD Table 1.4-1 lists the commercial nuclear reactors that were completed by GE or are under construction by GEH. For 50 years, GE provided advanced technology for nuclear energy. GE developed breakthrough light water technology in the mid-1950s: the Boiling Water Reactor (BWR). Since then, GE developed nine evolutions of BWR technology, including the first operational advanced light water design in the world, the ABWR, and culminating in its latest generation of design, the ESBWR. All of GE's nuclear technology has been transferred to GEH. There are 67 plants operating worldwide utilizing GEH designs with an operating capacity of over 59 GW, including 36 BWR plants in North America. Various subcontractors are supporting GEH.

1.4.2.1 Construction of the Turbine Island and Nuclear Island

The contractors for the construction of the turbine island and the nuclear island have not yet been selected. The turbine island and the nuclear island together represent the power block. The contractor for the construction of the turbine island will be responsible for the erection and delivery of the turbine building, the electric building, and the contents of each building. The contractor for the construction of the nuclear island will be responsible for the erection and delivery of the reactor and fuel building, the control building, the hot machine shop, the radwaste building, and the contents of each building. Each contractor will be selected based on their historical work in the nuclear industry, ongoing nuclear business, ability to deliver integrated engineering and construction services, and available resources.

1.4.3 Black & Veatch

Black & Veatch served as primary contractor for development of the COL application, supplying engineering support, conceptual design, environmental impact assessments, and project management. Black & Veatch, based in Overland Park, KS, is an engineering, environmental, technical, construction services, and management services firm providing a broad range of professional services to private and government sector clients throughout the world since 1915. Black & Veatch's nuclear activities date back to the closing years of World War II with early work

including extensive service to the Atomic Energy Commission in the development of facilities at Los Alamos, New Mexico. More recent activities include the development activities for other COLAs, the Advanced Boiling Water Reactor (AWBR) Design Certification Program, and the Department of Energy's 2010 initiative for the deployment of new nuclear plants in the United States. Various subcontractors are supporting Black & Veatch, including:

1.4.3.1 Professional Service Industries, Inc. (PSI)

PSI performed laboratory testing in support of Fermi 3 site specific evaluations in [Chapter 2](#) and the Environmental Report. This effort included laboratory testing of rock and soil materials and water quality.

1.4.3.2 Boart Longyear

Boart Longyear performed geotechnical field investigations in support of [Chapter 2](#). That effort included performing standard penetration tests; obtaining core samples and rock cores; performing cone penetrometer tests; supporting down-hole seismic tests and laboratory tests of soil and rock samples; installing ground water observation wells; and preparing a data report.

1.4.3.3 Geomatrix

Geomatrix Inc. performed probabilistic seismic hazard assessments and related sensitivity analyses in support of [Chapter 2](#). These assignments included sensitivity analyses of seismic source parameters and updated ground motion attenuation relationships, development of updated Safe Shutdown Earthquake (SSE) ground motion values, and preparation of the related sections.

Other subcontractors may be added as needed.

1.4.4 Other Contractors

In addition to the major contractors listed above, contractual relationships may be established with specialized consultants to assist in developing the COLA as the need arises.

1.5 Requirements for Further Technical Information

This section of the referenced DCD is incorporated by reference with no departures or supplements.

1.6 Material Incorporated By Reference

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

Add the following paragraph at the end of this section.

EF3 SUP 1.6-1

[Table 1.6-201](#) lists topical reports not included in DCD Section 1.6 that are incorporated in whole or in part by reference in the FSAR.

Table 1.6-201 Referenced Topical Reports (Sheet 1 of 2)

[EF3 SUP 1.6-1]

| Report No. | Title | Section No. |
|--------------|--|-------------------------------|
| NEI 06-13A | Nuclear Energy Institute, "Technical Report on Template for an Industry Training Program Description," NEI 06-13A, Revision 2, March 2009 | Appendix 13BB |
| NEI 07-02A | Nuclear Energy Institute, "Generic FSAR Template Guidance for Maintenance Rule Program Description for Plants Licensed under 10 CFR Part 52," NEI 07-02A, November 2010 | 17.6 |
| NEI 07-03A | Nuclear Energy Institute, "Generic FSAR Template Guidance for Radiation Protection Program Description," NEI 07-03A, Revision 0, May 2009 | Appendix 12BB |
| NEI 07-08A | Nuclear Energy Institute, "Generic FSAR Template Guidance for Ensuring That Occupational Radiation Exposures Are As Low As Is Reasonably Achievable (ALARA)," NEI 07-08A, Revision 0, October 2009 | Appendix 12AA |
| NEI 07-09A | Nuclear Energy Institute, "Generic FSAR Template Guidance for Offsite Dose Calculation Manual (ODCM) Program Description," NEI 07-09A, Revision 0, March 2009 | 11.5 |
| NEI 07-10A | Nuclear Energy Institute, "Generic FSAR Template Guidance for Process Control Program (PCP)," NEI 07-10A, Revision 0, March 2009 | 11.4 |
| NEI 06-12 | Nuclear Energy Institute, "B.5.b. Phase 2 & 3 Submittal Guideline," NEI 06-12, Revision 3, September 2009 | 13.6 |
| NEI 08-09 | Nuclear Energy Institute, "Cyber Security Plan for Nuclear Power Reactors", NEI 08-09, Revision 6, April 2010 | 13.6 |
| NEI 06-06 | Nuclear Energy Institute, "Fitness for Duty Program Guidance for New Nuclear Power Plant Construction Sites," NEI 06-06, Revision 5, August 2009 | 13.7 |
| ST-56834/P | General Electric Company, "ESBWR Steam Turbine - Low Pressure Rotor Missile Generation Probability Analysis," ST-56834/P, Revision 2, September 14, 2010 | 10.2 |
| ST-56834/N-P | General Electric Company, "ESBWR Steam Turbine - Low Pressure Rotor Missile Generation Probability Analysis," ST-56834/N-P, Revision 2, September 14, 2010. | 10.2 |
| NEI 08-08A | Nuclear Energy Institute, "Generic FSAR Template Guidance for Life Cycle," NEI 08-08A, Revision 0, October 2009. | 12.3 |
| NEDC-33441P | GE Hitachi, "GE Hitachi Nuclear Energy Methodology for the Development of ESBWR Reactor Pressure Vessel Pressure-Temperature Curves," Revision 4, December 2010 | 5.3 |
| NEDO-33441 | GE Hitachi, "GE Hitachi Nuclear Energy Methodology for the Development of ESBWR Reactor Pressure Vessel Pressure-Temperature Curves," Revision 4, December 2010 | 5.3 |
| NEDE-33389 | General Electric Hitachi, "ESBWR Safeguards Enhancements Report, "NEDE-Security Plan 33389, Revision 1, July 2009. | Security Plan |
| NEDE-33390 | General Electric Hitachi, "ESBWR Interim Compensatory Measures Report," NEDE-33390, Revision 1, November 2008. | Security Plan |

Table 1.6-201 Referenced Topical Reports (Sheet 2 of 2)

[EF3 SUP 1.6-1]

| Report No. | Title | Section No. |
|-------------------|--|--------------------|
| NEDE-33391 | General Electric Hitachi, "ESBWR Safeguards Assessment Report," NEDE-33391, Revision 3, March 2010. | Security Plan |

1.7 Drawings and Other Detailed Information

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

1.7.1 Electrical, Instrumentation and Control Drawings

Add the following at the end of this section.

EF3 SUP 1.7-1

[Table 1.7-201](#) supplements DCD Table 1.7-2 for those portions of the electrical system configuration drawings outside the scope of the DCD.

1.7.2 Piping and Instrumentation Diagrams

Add the following at the end of the first paragraph.

EF3 SUP 1.7-1

[Table 1.7-202](#) supplements DCD Table 1.7-3 for those portions of the mechanical system configuration drawings outside the scope of the DCD.

1.7.4 COL Information

1.7-1-H Final Design Configuration Confirmation

(Deleted)

Table 1.7-201 Summary of Electrical System Configuration Drawings

[EF3 SUP 1.7.1]

[Figure 8.2-201](#), 345 kV Switchyard Single-Line Diagram

Figure 8.2-202, (Deleted)

[Figure 8.2-203](#), Transmission Line Map

Table 1.7-202 Summary of Mechanical System Configuration Drawings

[EF3 SUP 1.7-1]

[Figure 9.2-201](#), Potable Water System Simplified Diagram

[Figure 9.2-202](#), Sanitary Waste Discharge System Simplified Diagram

[Figure 9.2-203](#), Station Water System - Plant Cooling Tower Makeup System (PCTMS)

[Figure 9.2-204](#), Station Water System - Pretreated Water Supply System (PWSS)

[Figure 9.2-205](#), Plant Service Water System Simplified Diagram

[Figure 9.5-201](#), Fire Protection System Yard Main Loop

[Figure 10.4-201](#), Circ Natural Draft Cooling Tower and Pump Pit

[Figure 10.4-202](#), Condenser and Ball Cleaning System

[Figure 11.4-1R](#), Solid Waste Management System Process Diagram

[Figure 11.4-2R](#), SWMS Collection Subsystem

1.8 Interfaces with Standard Design

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

1.8.2 Identification of Balance of Plant Interfaces

Add the following paragraph after the first paragraph of this section.

STD CDI

The significant interface requirements for those systems that are beyond the scope of the DCD are identified in DCD Tier 1.

Delete the second sentence of the second paragraph of this section.

EF3 SUP 1.8-7

1.8.2.8 Independent Spent Fuel Storage Installation

Replace this section with the following.

No Fermi 3 ISFSI is currently planned. Any future Fermi 3 ISFSI will be located considering the impacts of external hazards as required by the associated 10 CFR 72 license for the Fermi 3 ISFSI.

EF3 SUP 1.8-1

1.8.3 Verification of Site Parameters

[Chapter 2.0](#) provides information demonstrating that the site characteristics fall within the ESBWR site parameters specified in the referenced certified design.

EF3 SUP 1.8-2

1.8.4 COL Information Items and Permit Conditions

[Section 1.10](#) identifies specific FSAR sections that address the COL Information items from the referenced certified design, and COL Action Items.

EF3 SUP 1.8-3

1.8.5 Generic Changes and Departures from the Referenced Certified Design

One site-specific departure has been identified from the referenced certified design, which is described in COLA Part 7. (Reference [Table 1.8-201](#))

EF3 SUP 1.8-4

1.8.6 Variances from the ESP and ESPA SSAR

This supplement is not applicable to Fermi 3.

EF3 SUP 1.8-5

1.8.7 Conceptual Design Information

The referenced DCD includes conceptual design information (CDI) for certain systems, or portions of systems, that are outside the scope of the standard plant design. [Table 1.8-202](#) identifies systems for which either the CDI in the DCD is adopted as the actual system design information, or the CDI in the DCD is replaced with site-specific design information, along with cross references to FSAR sections where the CDI is treated. Where there are differences between the conceptual design and the actual design, these differences have been evaluated. The evaluations have concluded that there are no impacts on the safety evaluations provided in the referenced certified design.

EF3 SUP 1.8-6

1.8.8 Probabilistic Risk Assessment

Site- and plant-specific information, including site meteorological data and site-specific population distribution, plant-specific design information that replaced conceptual design information described in the DCD, and the departures listed in [Subsection](#) , were reviewed with respect to the design certification PRA. The conclusion, which is documented in [Section 19.5](#), is that there is no significant change from the certified design PRA.

Table 1.8-201 Departures from the Referenced Certified Design [EF3 SUP 1.8-3]

| Number | Subject | FSAR Section |
|----------------|---|---|
| EF3 DEP 11.4-1 | Long-Term, Temporary Storage of Class B and C Low-Level Radioactive Waste | 1.2.2.10.2, 1.2.2.16.9, 9.4.3.1, 11.4, 11.4.1, 11.4.2.2.1, 11.4.2.2.2, 11.4.2.2.4, and 11.4.2.3.1 |

Table 1.8-202 Conceptual Design Information (CDI) (Sheet 1 of 2)

[EF3 SUP 1.8-5]

| Item in DCD | CDI in DCD adopted as actual design | CDI in DCD replaced with actual design | Evaluation | FSAR Section |
|--|---|--|---|--|
| 1.1.2.1 ESBWR Standard Plant Scope Figure 1.1-1 ESBWR Standard Plant General Site Plan | | X | Site plan general site plan provided | 1.1.2.1 Figure 2.1-204 |
| 1.2.2.11.4 Main Turbine | X | | Conceptual turbine type selected as site specific design | 1.2.2.11.4 |
| 1.2.2.11.7 Main Condenser | X | | Conceptual condenser type selected as site specific design | 1.2.2.11.7 |
| 1.2.2.12.13 Hydrogen Water Chemistry Table 3.2-1 P73 Note 9.3.9 Hydrogen Water Chemistry | | X | Hydrogen water chemistry option utilized | 1.2.2.12.13 9.3.9 |
| 1.2.2.12.15 Zinc Injection System Table 3.2-1 P74 Note 9.3.11 Zinc Injection System | | X | Zinc Injection system not utilized | 1.2.2.12.15 9.3.11 |
| 1.2.2.12.16 Freeze Protection | | X | Freeze protection incorporated for external tanks and piping that may freeze during winter weather | 1.2.2.12.16 |
| 1.2.2.16.10 Other Building Structures | | X | Site-specific buildings specified | 1.2.2.16.10 |
| 1.8.2 Identification of BOP Interfaces | X | | Not applicable | |
| Appendix 3A Seismic Soil-Structure Interaction Analysis | | X | Site-specific geotechnical data described in Chapter 2 | Appendix 3A Chapter 2 |
| Appendix 3A.2 ESBWR Standard Site Plan | | X | Site-specific general site plan provided | Appendix 3A Figure 2.1-201 |
| 9.2.1 Plant Service Water Table 9.2-2 Figure 9.2-1 | | X | Site-specific system description and design characteristics described | 9.2.1 Table 9.2-201 Figure 9.2-205 |
| 9.2.3 Makeup Water System Table 9.2-9 | | X | Site-specific system description and design characteristics described | 9.2.3 Table 9.2-202 |

Table 1.8-202 Conceptual Design Information (CDI) (Sheet 2 of 2)

[EF3 SUP 1.8-5]

| Item in DCD | CDI in DCD adopted as actual design | CDI in DCD replaced with actual design | Evaluation | FSAR Section |
|--|---|--|---|---|
| 9.2.4 Potable and Sanitary Water Systems | | X | Site-specific system description and design characteristics described | 9.2.4 Table 9.2-201 Table 9.2-202 |
| 9.2.10 Station Water System | | X | Site-specific system description and design characteristics described | 9.2.10 Table 9.2-204 Table 9.2-205 Figure 9.2-203 Figure 9.2-204 |
| 9.3.9 Hydrogen Water Chemistry System | | X | Site-specific system description and design characteristics described | 9.3.9 |
| 9.3.11 Zinc Injection System | | X | Zinc Injection System not utilized | 9.3.11 |
| 9A Appendix 9A Fire Hazards Analysis | | X | Site-specific building specified. Site-specific Fire Zone drawing supplied. | Figure 9A.2-33R Figure 9A.2-201 |
| 10.4.5 Circulating Water System Table 10.4-3 Figure 10.4-1 | | X | Site-specific system description and design characteristics described | 10.4.5.2.1 10.4.5.2.2 10.4.5.2.3 10.4.5.6 10.4.5.8 Table 10.4-201 Table 10.4-3R Figure 10.4-201 Figure 10.4-202 |

1.9 Conformance with Standard Review Plan and Applicability of Codes and Standards

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

1.9.1 Conformance with Standard Review Plan

Add the following paragraph at the end of this section.

EF3 COL 1.9-3-A

[Table 1.9-201](#) evaluates conformance with the SRP sections and BTPs in effect six months prior to the submittal of the COLA. [Table 1.9-201](#) does not re-address conformance with the SRP for those portions of the facility design included in the referenced certified design.

In the table, the term “Conforms” means that no exception is being taken to the guidance in the SRP section/acceptance criteria as they apply to site-specific design information, operational aspects of the facility, or siting information in the FSAR. The term “Not applicable” means that the SRP section/acceptance criteria do not apply to the ESBWR or Fermi 3. Any differences with the SRP acceptance criteria are identified and justified, with references to the applicable FSAR section(s) that address the difference, as necessary.

1.9.2 Applicability to Regulatory Criteria

Add the following paragraphs at the end of this section.

EF3 COL 1.9-3-A

Division 1, 4, 5, and 8 Regulatory Guides

[Table 1.9-202](#) evaluates conformance with Division 1, 4, 5, and 8 RGs in effect six months prior to the submittal of the COLA. Each issued Division 1 RG is evaluated. Issued Division 4, 5, and 8 RGs identified in the SRP, RG 1.206, or DCD Table 1.9-21 as COL responsibility are also evaluated. [Table 1.9-202](#) does not re-address conformance with RGs for those portions of the facility design included in the referenced certified design.

In the table, the term “Conforms” means that no exception is being taken to the guidance in the regulatory positions as they apply to site-specific design information, operational aspects of the facility, or siting information in the FSAR. The term “Not applicable” means that the regulatory positions do not apply to the ESBWR or Fermi 3.

Regulatory Guide 1.206

[Table 1.9-203](#) evaluates conformance with the FSAR content guidance in RG 1.206. Where necessary, the table identifies the FSAR section where the required information is provided. In the table, the term “Conforms” means that the information called for in RG 1.206 is either: 1) already addressed in the DCD; or 2) addressed by adding new information beyond that contained in the DCD. The term “Not applicable” means that the information called for in RG 1.206 does not apply to the ESBWR or Fermi 3.

[Table 1.9-203](#) evaluates conformance with RG 1.206, Section C.III.1 , “Information Needed for a Combined License Application Referencing a Certified Design.” Section C.I, “Standard Format and Content of Combined License Applications for Nuclear Power Plants-Light-Water Reactor Edition,” were also evaluated, as applicable, if portions of these sections were referenced or identified in RG 1.206, Section C.III.1 .

EF3 SUP 1.9-1

Industrial Codes and Standards

[Table 1.9-204](#) identifies the Industrial Codes and Standards that are applicable to those portions of the Fermi 3 design that are beyond the scope of the DCD, and to the operational aspects of the facility.

1.9.3 Applicability of Experience Information

Add the following after the first sentence of the section.

EF3 SUP 1.9-2

[Table 1.9-205](#) lists NUREG and NUREG/CR reports cited in the FSAR.

Add the following paragraph at the end of this section.

[Table 1.9-205](#) addresses operational experience information, as described in applicable NUREG reports, for those portions of the Fermi 3 design and operation that are beyond the scope of the DCD. The comment column of [Table 1.9-205](#) includes a reference to the applicable FSAR section that provides further discussion of the operational experience.

COL Information

1.9-3-A SRP and Regulatory Guide Applicability

EF3 COL 1.9-3-A

This COL Item is addressed in [Subsection 1.9.1](#) and [Subsection 1.9.2](#).

Table 1.9-201 Conformance with Standard Review Plan (Sheet 1 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|---------------|---|------------------|--------|--|--|
| 1 | Introduction and Interfaces | Initial Issuance | Mar-07 | No Specific Acceptance Criteria | Conforms |
| 2.0 | Site Characteristics and Site Parameters | Initial Issuance | Mar-07 | II.1, II.2, II.3, II.5 | Not applicable |
| | | | | II.4 | Conforms |
| 2.1.1 | Site Location and Description | Rev. 3 | Mar-07 | II.1, II.2 | Conforms |
| 2.1.2 | Exclusion Area Authority and Control | Rev. 3 | Mar-07 | II.1, II.2, II.3 | Conforms |
| 2.1.3 | Population Distribution | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5 | Conforms |
| 2.2.1 - 2.2.2 | Identification of Potential Hazards in Site Vicinity | Rev. 3 | Mar-07 | II.1, II.2, II.3 | Conforms |
| 2.2.3 | Evaluation of Potential Accidents | Rev. 3 | Mar-07 | II.1, II.2 | Conforms |
| 2.3.1 | Regional Climatology | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9 | Conforms |
| 2.3.2 | Local Meteorology | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms |
| 2.3.3 | Onsite Meteorological Measurements Programs | Rev. 3 | Mar-07 | II.1, II.2, II.3 | Conforms with exception of proximity of trees to meteorological tower. Impacts from trees is addressed in Subsection 2.3.3.1.6 . |
| 2.3.4 | Short Term Atmospheric Dispersion Estimates for Accident Releases | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6 | Conforms |
| 2.3.5 | Long-Term Atmospheric Dispersion Estimates for Routine Releases | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6 | Conforms |
| 2.4.1 | Hydrologic Description | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6 | Conforms |

Table 1.9-201 Conformance with Standard Review Plan (Sheet 2 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|--------------------|--|------------|-------------|---|---|
| 2.4.2 | Floods | Rev. 4 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10 | Conforms |
| 2.4.3 | Probable Maximum Flood (PMF) on Streams and Rivers | Rev. 4 | Mar-07 | II.1, II.2, II.3 | Conforms |
| 2.4.4 | Potential Dam Failures | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7 | Conforms |
| 2.4.5 | Probable Maximum Surge and Seiche Flooding | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6 | Conforms |
| 2.4.6 | Probable Maximum Tsunami Hazards | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8 | Conforms |
| 2.4.7 | Ice Effects | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5 | Conforms |
| 2.4.8 | Cooling Water Canals and Reservoirs | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms |
| 2.4.9 | Channel Diversions | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7 | Conforms |
| 2.4.10 | Flooding Protection Requirements | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms |
| 2.4.11 | Low Water Considerations | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5 | Conforms |
| 2.4.12 | Groundwater | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5 | Conforms |
| 2.4.13 | Accidental Releases of Radioactive Liquid Effluents in Ground and Surface Waters | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5 | Conforms. A tank rupture analysis was not performed since special design features were incorporated to mitigate the consequences of failures. |
| 2.4.14 | Technical Specifications and Emergency Operation Requirements | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5 | Conforms |
| 2.5.1 | Basic Geologic and Seismic Information | Rev. 4 | Mar-07 | II.1, II.2 | Conforms |
| 2.5.2 | Vibratory Ground Motion | Rev. 4 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6 | Conforms |

Table 1.9-201 Conformance with Standard Review Plan (Sheet 3 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|--------------------|--|------------|-------------|--|-------------------|
| 2.5.3 | Surface Faulting | Rev. 4 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8 | Conforms |
| 2.5.4 | Stability of Subsurface Materials and Foundations | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5 II.6, II.7, II.8, II.9, II.10, II.11, II.12 | Conforms |
| 2.5.5 | Stability of Slopes | Rev. 3 | Mar-07 | II.1, II.2, II.3 | Conforms |
| 3.2.1 | Seismic Classification | Rev. 2 | Mar-07 | II.1 | Conforms |
| 3.2.2 | System Quality Group Classification | Rev. 2 | Mar-07 | II.1 | Conforms |
| 3.3.1 | Wind Loadings | Rev. 3 | Mar-07 | II.1, II.2, II.3 | Conforms |
| 3.3.2 | Tornado Loadings | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms |
| 3.4.1 | Internal Flood Protection for Onsite Equipment Failures | Rev. 3 | Mar-07 | II.1, II.2 | Conforms |
| 3.4.2 | Analysis Procedures | Rev. 3 | Mar-07 | II.1, II.2, II.3 | Conforms |
| 3.5.1.1 | Internally Generated Missiles (Outside Containment) | Rev. 3 | Mar-07 | II.1, II.2 | Conforms |
| 3.5.1.2 | Internally-Generated Missiles (Inside Containment) | Rev. 3 | Mar-07 | II.1, II.2 | Conforms |
| 3.5.1.3 | Turbine Missiles | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6 | Conforms |
| 3.5.1.4 | Missiles Generated by Tornadoes and Extreme Winds | Rev. 3 | Mar-07 | II.1, II.2 | Conforms |
| 3.5.1.5 | Site Proximity Missiles (Except Aircraft) | Rev. 4 | Mar-07 | II.1, II.2 | Conforms |
| 3.5.1.6 | Aircraft Hazards | Rev. 3 | Mar-07 | II.1, II.2 | Conforms |
| 3.5.2 | Structures, Systems, and Components to be Protected from Externally-Generated Missiles | Rev. 3 | Mar-07 | | Conforms |

Table 1.9-201 Conformance with Standard Review Plan (Sheet 4 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|--------------------|---|------------|-------------|---|--|
| 3.5.3 | Barrier Design Procedures | Rev. 3 | Mar-07 | II.1, II.2 | Conforms |
| 3.6.1 | Plant Design for Protection Against Postulated Piping Failures in Fluid Systems Outside Containment | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5 | Conforms |
| 3.6.2 | Determination of Rupture Locations and Dynamic Effects Associated with the Postulated Rupture of Piping | Rev. 2 | Mar-07 | II.1, II.2, II.3 | Conforms |
| 3.6.3 | Leak-Before-Break Evaluation Procedures | Rev. 1 | Mar-07 | II.1, II.2 | Not applicable. ESBWR design does not rely on a Leak Before Break Evaluation. |
| 3.7.1 | Seismic Design Parameters | Rev. 3 | Mar-07 | All except II.4 | Conforms except that the ESBWR is based on a single earthquake (SSE) design. |
| | | | | II.4 | Conforms - supplemented by "Interim Staff Guidance on Seismic Issues Associated with High Frequency Ground Motion in Design Certification and Combined License Applications," COL/DCISG-1, May 2008. |
| 3.7.2 | Seismic System Analysis | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12, II.13, II.14 | Conforms |
| 3.7.3 | Seismic Subsystem Analysis | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12, II.13, II.14 | Conforms except that the ESBWR is based on a single earthquake (SSE) design. |
| 3.7.4 | Seismic Instrumentation | Rev. 2 | Mar-07 | II.1, II.2 | Conforms |
| 3.8.1 | Concrete Containment | Rev. 2 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7 | Conforms |
| 3.8.2 | Steel Containment | Rev. 2 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7 | Conforms |

Table 1.9-201 Conformance with Standard Review Plan (Sheet 5 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|--------------------|--|------------|-------------|--|---|
| 3.8.3 | Concrete and Steel Internal Structures of Steel or Concrete Containments | Rev. 2 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7 | Conforms |
| 3.8.4 | Other Seismic Category I Structures | Rev. 2 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8 | Conforms |
| 3.8.5 | Foundations | Rev. 2 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7 | Conforms |
| 3.9.1 | Special Topics for Mechanical Components | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms |
| 3.9.2 | Dynamic Testing and Analysis of Systems, Structures, and Components | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7 | Conforms |
| 3.9.3 | ASME Code Class 1, 2, and 3 Components, and Component Supports, and Core Support Structures | Rev. 2 | Mar-07 | II.1, II.2, II.3 | Conforms |
| 3.9.4 | Control Rod Drive Systems | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms |
| 3.9.5 | Reactor Pressure Vessel Internals | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6 | Conforms |
| 3.9.6 | Functional Design, Qualification, and Inservice Testing Programs for Pumps, Valves, and Dynamic Restraints | Rev. 3 | Mar-07 | II.1, II.3, II.4, II.5, II.6 | Conforms |
| | | | | II.2 | Not applicable. There are no safety related pumps. |
| 3.9.7 | Risk-Informed Inservice Testing | Rev. 0 | Aug-98 | II.A, II.B | Not applicable. Risk-informed inservice testing is not being used. |
| 3.9.8 | Risk-Informed Inservice Inspection of Piping | Rev. 0 | Sep-03 | II.1, II.2, II.3 | Not applicable. Risk-informed inservice inspection of piping is not being used. |

Table 1.9-201 Conformance with Standard Review Plan (Sheet 6 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|--------------------|--|------------------|-------------|---|-------------------|
| 3.10 | Seismic and Dynamic Qualification of Mechanical and Electrical Equipment | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.5 | Conforms |
| | | | | II.4, II.6 | Conforms |
| 3.11 | Environmental Qualification of Mechanical and Electrical Equipment | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12, II.13, II.14, II.15, II.16 | Conforms |
| 3.12 | ASME Code Class 1, 2, and 3 Piping Systems, Piping Components and their Associated Supports | Initial Issuance | Mar-07 | II.A, II.B, II.C, II.D | Conforms |
| 3.13 | Threaded Fasteners - ASME Code Class 1, 2, and 3 | Initial Issuance | Mar-07 | II.1, II.2 | Conforms |
| BTP 3-1 | Classification of Main Steam Components Other than the Reactor Coolant Pressure Boundary for BWR Plants | Rev. 2 | Mar-07 | | Conforms |
| BTP 3-2 | Classification of BWR/6 Main Steam and Feedwater Components Other than the Reactor Coolant Pressure Boundary | Rev. 2 | Mar-07 | | Conforms |
| BTP 3-3 | Protection Against Postulated Piping Failures in Fluid Systems Outside Containment | Rev. 3 | Mar-07 | | Conforms |
| BTP 3-4 | Postulated Rupture Locations in Fluid System Piping Inside and Outside Containment | Rev. 2 | Mar-07 | | Conforms |
| 4.2 | Fuel System Design | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms |

Table 1.9-201 Conformance with Standard Review Plan (Sheet 7 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|--|--------|--------|--|---|
| 4.3 | Nuclear Design | Rev. 3 | Mar-07 | II.1, II.2, II.4 | Conforms |
| | | | | II.3 | Conforms |
| 4.4 | Thermal and Hydraulic Design | Rev. 2 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.8, II.9, II.10 | Conforms |
| | | | | II.7 | Not applicable |
| 4.5.1 | Control Rod Drive Structural Materials | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms |
| 4.5.2 | Reactor Internal and Core Support Structure Materials | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5 | Conforms |
| 4.6 | Functional Design of Control Rod Drive System | Rev. 2 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8 | Conforms |
| BTP 4-1 | Westinghouse Constant Axial Offset Control (CAOC) | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| 5.2.1.1 | Compliance with the Codes and Standards Rule, 10 CFR 50.55a | Rev. 3 | Mar-07 | RG 1.26 | Conforms |
| 5.2.1.2 | Applicable Code Cases | Rev. 3 | Mar-07 | RG 1.84, RG 1.147, RG 1.192 | Conforms |
| 5.2.2 | Overpressure Protection | Rev. 3 | Mar-07 | II.1, II.2, II.5, II.6, II.7 | Conforms |
| | | | | II.3, & II.4 | Not applicable to the ESBWR |
| 5.2.3 | Reactor Coolant Pressure Boundary Materials | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms. Acceptance Criterion II.3 is addressed in DCD Section 5.2.3.3 |
| 5.2.4 | Reactor Coolant Pressure Boundary Inservice Inspection and Testing | Rev. 2 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11 | Conforms |
| 5.2.5 | Reactor Coolant Pressure Boundary Leakage Detection | Rev. 2 | Mar-07 | II.1, II.2 | Conforms |
| 5.3.1 | Reactor Vessel Materials | Rev. 2 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7 | Conforms |

Table 1.9-201 Conformance with Standard Review Plan (Sheet 8 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|--------------------|--|------------|-------------|---|---|
| 5.3.2 | Pressure-Temperature Limits, Upper-Shelf Energy, and Pressurized Thermal Shock | Rev. 2 | Mar-07 | II.1, II.2, II.3 | Conforms |
| 5.3.3 | Reactor Vessel Integrity | Rev. 2 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8 | Conforms |
| 5.4 | Reactor Coolant System Component and Subsystem Design | Rev. 2 | Mar-07 | | Conforms |
| 5.4.1.1 | Pump Flywheel Integrity (PWR) | Rev. 2 | Mar-07 | | Not applicable to the ESBWR |
| 5.4.2.1 | Steam Generator Materials | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| 5.4.2.2 | Steam Generator Program | Rev. 2 | Mar-07 | | Not applicable to the ESBWR |
| 5.4.6 | Reactor Core Isolation Cooling System (BWR) | Rev. 4 | Mar-07 | II.1, II.2, II.4, II.5, II.6, II.7a, b, c & f, II.8, II.9, II.10 | As applicable to a passive-design isolation condenser system (ICS) in place of an active-design reactor core isolation cooling (RCIC) system. Criterion II.2 addressed in DCD Table 1.9-5 |
| | | | | II.3 and II.7d & e | Not applicable to the ESBWR |
| 5.4.7 | Residual Heat Removal (RHR) System | Rev. 4 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms |
| 5.4.8 | Reactor Water Cleanup System (BWR) | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms |
| 5.4.11 | Pressurizer Relief Tank | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| 5.4.12 | Reactor Coolant System High Point Vents | Rev. 1 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12, II.13, II.14 | Conforms |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|---|------------------|--------|---|---|
| 5.4.13 | Isolation Condenser System (BWR) | Initial Issuance | Mar-07 | II.1, II.2, II.3, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12 | Conforms |
| | | | | II.4 | Conforms with the following exception: The ESBWR is designed to shut down safely without reliance on offsite or diesel-generator-derived AC power, therefore, RG 1.93 is only applicable to onsite safety-related DC power systems. |
| BTP 5-1 | Monitoring of Secondary Side Water Chemistry in PWR Steam Generators | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| BTP 5-2 | Overpressurization Protection of Pressurized-Water Reactors While Operating at Low Temperatures | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| BTP 5-3 | Fracture Toughness Requirements | Rev. 3 | Mar-07 | | Conforms |
| BTP 5-4 | Design Requirements of the Residual Heat Removal System | Rev. 3 | Mar-07 | | Not applicable to ESBWR |
| 6.1.1 | Engineered Safety Features Materials | Rev. 2 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms |
| 6.1.2 | Protective Coating Systems (Paints) - Organic Materials | Rev. 3 | Mar-07 | II.1 | Conforms |
| 6.2.1 | Containment Functional Design | Rev. 3 | Mar-07 | | Conforms |
| 6.2.1.1.A | PWR Dry Containments, Including Subatmospheric Containments | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|--------------------|--|--------------|-------------|---|---|
| 6.2.1.1.B | Ice Condenser Containments | Draft Rev. 3 | Jun-96 | | Not applicable to the ESBWR |
| 6.2.1.1.C | Pressure-Suppression Type BWR Containments | Rev. 7 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11 | Conforms |
| 6.2.1.2 | Subcompartment Analysis | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms |
| 6.2.1.3 | Mass and Energy Release Analysis for Postulated Loss-of-Coolant Accidents (LOCAs) | Rev. 3 | Mar-07 | II.1, II.2, II.3 | Conforms |
| 6.2.1.4 | Mass and Energy Release Analysis for Postulated Secondary System Pipe Ruptures | Rev. 2 | Mar-07 | | Not applicable to the ESBWR |
| 6.2.1.5 | Minimum Containment Pressure Analysis for Emergency Core Cooling System Performance Capability Studies | Rev. 3 | Mar-07 | | Conforms. See DCD Table 1.9-6, and DCD Table 1.9-20, and Appendix 6C. |
| 6.2.2 | Containment Heat Removal Systems | Rev. 5 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8 | Conforms |
| 6.2.3 | Secondary Containment Functional Design | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms. See DCD Table 1.9-20. |
| 6.2.4 | Containment Isolation System | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12, II.13, II.14, II.15, II.16, II.17, II.18, II.19, II.20, II.21, II.22 | Conforms |
| 6.2.5 | Combustible Gas Control in Containment | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9 | Conforms |
| 6.2.6 | Containment Leakage Testing | Rev. 3 | Mar-07 | | Conforms |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|--|--------|--------|---|---|
| 6.2.7 | Fracture Prevention of Containment Pressure Boundary | Rev. 1 | Mar-07 | II.1, II.2 | Conforms |
| 6.3 | Emergency Core Cooling System | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.6, II.7, II.8, II.10 | Conforms |
| | | | | II.5, II.9 | Not applicable |
| 6.4 | Control Room Habitability System | Rev. 3 | Mar-07 | II.1, II.2, II.4, II.5, II.6 | Conforms |
| | | | | II.3 | Exception: For differential pressure testing of the control room, the periodic verification interval of every 18 months in Acceptance Criteria II.3.a through II.3.c is increased to every 24 months to accommodate the ESBWR's two year operating cycle. The frequencies for testing the CR HVAC system are defined by Technical Specifications 3.7.2 and 5.5.12 of the referenced certified design. |
| | | | | II.7 | Exception: SRP states that self-contained breathing apparatus for the control room personnel should be on hand. DCD 6.4.1.1 states that CRHA habitability requirements are satisfied without the need for individual breathing apparatus and/or special clothing. |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|---|------------------|--------|--|--|
| 6.5.1 | ESF Atmosphere Cleanup Systems | Rev. 3 | Mar-07 | | Conforms. Surveillances, testing, and maintenance guidelines for the CRHAVS are addressed in Technical Specifications 3.7.2, 5.5.12, and 5.5.13, Maintenance Rule requirements in Section 17.6, and procedure requirements in Section 13.5 |
| 6.5.2 | Containment Spray as a Fission Product Cleanup System | Rev. 4 | Mar-07 | | Not applicable. See DCD Table 1.9-20. |
| 6.5.3 | Fission Product Control Systems and Structures | Rev. 3 | Mar-07 | II.1, II.2, (there is no II.3) | Conforms |
| | | | | II.4 | Not applicable. Drywell spray function is not credited in DCD Chapter 15 dose analysis. |
| 6.5.4 | Ice Condenser as a Fission Product Cleanup System | Draft Rev. 4 | Jun-96 | | Not applicable to the ESBWR |
| 6.5.5 | Pressure Suppression Pool as a Fission Product Cleanup System | Rev. 1 | Mar-07 | II.1, II.2 | Conforms. Refer to DCD Table 1.9-20. |
| | | | | II.3 | Not applicable. |
| 6.6 | Inservice Inspection and Testing of Class 2 and 3 Components | Rev. 2 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11 | Conforms |
| 6.7 | Main Steam Isolation Valve Leakage Control System (BWR) | Draft Rev. 3 | Jun-96 | | Not applicable |
| BTP 6-1 | pH For Emergency Coolant Water for Pressurized Water Reactors | Initial Issuance | Mar-07 | | Not applicable to the ESBWR |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|--------------------|--|------------|-------------|-------------------------------------|---|
| BTP 6-2 | Minimum Containment Pressure Model for PWR ECCS Performance Evaluation | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| BTP 6-3 | Determination of Bypass Leakage Paths in Dual Containment Plants | Rev. 3 | Mar-07 | | Conforms. Refer to DCD Table 1.9-20. |
| BTP 6-4 | Containment Purging During Normal Plant Operations | Rev. 3 | Mar-07 | | Conforms. Refer to TS SR 3.6.1.3. |
| BTP 6-5 | Currently the Responsibility of Reactor Systems Piping From the RWST (or BWST) and Containment Sump(s) to the Safety Injection Pumps | Rev. 3 | Mar-07 | | Not applicable |
| 7.0 | Instrumentation and Controls - Overview of Review Process | Rev. 5 | Mar-07 | | Conforms |
| Appendix 7.0-A | Review Process for Digital Instrumentation and Control Systems | Rev. 5 | Mar-07 | | Conforms |
| 7.1 | Instrumentation and Controls - Introduction | Rev. 5 | Mar-07 | II.1, II.2, II.3 | Conforms. Procedures addressed in Section 13.5 . ITAAC addressed in COLA Part 10. |
| 7.1-T | Table 7-1 Regulatory Requirements, Acceptance Criteria, and Guidelines for Instrumentation and Control Systems Important to Safety | Rev. 5 | Mar-07 | | Conforms |

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| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|----------------|---|------------------|--------|---|--|
| Appendix 7.1-A | Acceptance Criteria and Guidelines for Instrumentation and Controls Systems Important to Safety | Rev. 5 | Mar-07 | 1, 2, 3, 4, 5 | Conforms |
| Appendix 7.1-B | Guidance for Evaluation of Conformance to IEEE Std 279 | Rev. 5 | Mar-07 | | Conforms |
| Appendix 7.1-C | Guidance for Evaluation of Conformance to IEEE Std 603 | Rev. 5 | Mar-07 | | Conforms |
| Appendix 7.1-D | Guidance for Evaluation of the Application of IEEE Std 7-4.3.2 | Initial Issuance | Mar-07 | SRM to SECY 93-087 II.Q | Conforms |
| 7.2 | Reactor Trip System | Rev. 5 | Mar-07 | II.1, II.2, II.3, II.4, SRM to SECY 93-087 II.Q | Conforms. Procedures addressed in Section 13.5 . Technical Specifications addressed in Chapter 16 . ITAAC addressed in COLA Part 10. |
| 7.3 | Engineered Safety Features Systems | Rev. 5 | Mar-07 | II.1, II.2, II.3, II.4, SRM to SECY 93-087 II.Q | Conforms. Procedures addressed in Section 13.5 . Technical Specifications addressed in Chapter 16 . ITAAC addressed in COLA Part 10. |
| 7.4 | Safe Shutdown Systems | Rev. 5 | Mar-07 | II.1, II.2, II.3 | Conforms. Procedures addressed in Section 13.5 . Technical Specifications addressed in Chapter 16 . ITAAC addressed in COLA Part 10. |
| 7.5 | Information Systems Important to Safety | Rev. 5 | Mar-07 | II.1, II.2, II.3, II.4, II.5, SRM to SECY 93-087 II.Q | Conforms. Procedures addressed in Section 13.5 . Technical Specifications addressed in Chapter 16 . ITAAC addressed in COLA Part 10. |
| 7.6 | Interlock Systems Important to Safety | Rev. 5 | Mar-07 | II.1, II.2, II.3 | Conforms. Procedures addressed in Section 13.5 . Technical Specifications addressed in Chapter 16 . ITAAC addressed in COLA Part 10. |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|--------------|--|--------|--------|---|--|
| 7.7 | Control Systems | Rev. 5 | Mar-07 | II.1, II.2, II.3, II.4, SRM to SECY 93-087 II.Q | Conforms. Procedures addressed in Section 13.5 . Technical Specifications addressed in Chapter 16 . ITAAC addressed in COLA Part 10. |
| 7.8 | Diverse Instrumentation and Control Systems | Rev. 5 | Mar-07 | II.1, II.2, II.3, II.4, SRM to SECY 93-087 II.Q | Conforms. Procedures addressed in Section 13.5 . Technical Specifications addressed in Chapter 16 . ITAAC addressed in COLA Part 10. |
| 7.9 | Data Communication Systems | Rev. 5 | Mar-07 | II.1, II.2, II.3 | Conforms. Addressed in DCD Section 7.1. Procedures addressed in Section 13.5 . Technical Specifications addressed in Chapter 16 . ITAAC addressed in COLA Part 10. |
| Appendix 7-A | General Agenda, Station Site Visits (formerly Appendix 7-B) | Rev. 5 | Mar-07 | | Not applicable. Provides guidance to the NRC to conduct site visits. |
| Appendix 7-B | Acronyms, Abbreviations, and Glossary (formerly Appendix 7-C) | Rev. 5 | Mar-07 | | Conforms |
| BTP 7-1 | Guidance on Isolation of Low-Pressure Systems from the High-Pressure Reactor Coolant System | Rev. 5 | Mar-07 | | Conforms |
| BTP 7-2 | Guidance on Requirements of Motor-Operated Valves in the Emergency Core Cooling System Accumulator Lines | Rev. 5 | Mar-07 | | Not applicable to the ESBWR |
| BTP 7-3 | Guidance on Protection System Trip Point Changes for Operation with Reactor Coolant Pumps Out of Service | Rev. 5 | Mar-07 | | Not applicable to the ESBWR |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|---|--------|--------|------------------------------|--|
| BTP 7-4 | Guidance on Design Criteria for Auxiliary Feedwater Systems | Rev. 5 | Mar-07 | | Not applicable to the ESBWR |
| BTP 7-5 | Guidance on Spurious Withdrawals of Single Control Rods in Pressurized Water Reactors | Rev. 5 | Mar-07 | | Not applicable to the ESBWR |
| BTP 7-6 | Guidance on Design of Instrumentation and Controls Provided to Accomplish Changeover from Injection to Recirculation Mode | Rev. 5 | Mar-07 | | Not applicable. ESBWR does not use recirculation pumps or active ECCS pumps. |
| HICB-7 | Not Used | | | | Not used |
| BTP 7-8 | Guidance for Application of Regulatory Guide 1.22 | Rev. 5 | Mar-07 | | Conforms. Chapter 16 addresses Technical Specifications. |
| BTP 7-9 | Guidance on Requirements for Reactor Protection System Anticipatory Trips | Rev. 5 | Mar-07 | | Conforms |
| BTP 7-10 | Guidance on Application of Regulatory Guide 1.97 | Rev. 5 | Mar-07 | | Conforms. Section 13.5 addresses procedures. |
| BTP 7-11 | Guidance on Application and Qualification of Isolation Devices | Rev. 5 | Mar-07 | | Conforms. |
| BTP 7-12 | Guidance on Establishing and Maintaining Instrument Setpoints | Rev. 5 | Mar-07 | | Conforms. Section 13.5 addresses procedures. |
| BTP 7-13 | Guidance on Cross-Calibration of Protection System Resistance Temperature Detectors | Rev. 5 | Mar-07 | | Not applicable. RTDs are not used in the ESBWR protection systems. |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|---|--------|--------|------------------------------|---|
| BTP 7-14 | Guidance on Software Reviews for Digital Computer-Based Instrumentation and Control Systems | Rev. 5 | Mar-07 | | Conforms |
| HCIB-15 | Not Used | | | | Not used |
| BTP 7-16 | Withdrawn | | | | Withdrawn |
| BTP 7-17 | Guidance on Self-Test and Surveillance Test Provisions | Rev 5 | Mar-07 | | Conforms. Section 13.5 addresses procedures. Chapter 16 addresses Technical Specifications. |
| BTP 7-18 | Guidance on the Use of Programmable Logic Controllers in Digital Computer-Based Instrumentation and Control Systems | Rev. 5 | Mar-07 | | Conforms. Section 13.5 addresses procedures. |
| BTP 7-19 | Guidance for Evaluation of Diversity and Defense-in-Depth in Digital Computer-Based Instrumentation and Control Systems | Rev. 5 | Mar-07 | | Conforms |
| HCIB-20 | Not Used | | | | Not used |
| BTP 7-21 | Guidance on Digital Computer Real-Time Performance | Rev. 5 | Mar-07 | | Conforms |
| 8.1 | Electric Power - Introduction | Rev. 3 | Mar-07 | | Conforms |

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| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|----------------------------|--------|--------|--|--|
| 8.2 | Offsite Power System | Rev. 4 | Mar-07 | II.4, II.5, II.6, II.8 | Conforms |
| | | | | II.1, II.2, II.3, II.7 | Not applicable. ESBWR is a passive design and does not rely on offsite power. |
| 8.3.1 | A-C Power Systems (Onsite) | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4.A through II.4.H, II.4.J, II.5, II.6, II.7, II.10 | Conforms |
| | | | | II.4.I | Not applicable. The ESBWR diesel generators are not safety-related. |
| | | | | II.8 | Not applicable. The ESBWR diesel generators are not safety-related, nor is AC power needed to achieve safe shutdown. |
| | | | | II.9 | Conforms. Addressed in DCD 17.4 and Section 17.6 . |
| 8.3.2 | D-C Power Systems (Onsite) | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.7, II.8, II.9, II.10 | Conforms |
| | | | | II.5, II.6 | Not applicable. Addressed in DCD Sections 8.3.2.1.1 and 8.3.2.2.2. |
| | | | | II.11 | Not applicable. The ESBWR is designed to shutdown safely without reliance on offsite or diesel-generator-derived AC power for 72 hours, which exceeds station blackout requirements. |
| | | | | II.12 | Conforms. Addressed in Section 17.6 . |
| | | | | II.13 | Conforms. Addressed in Section 17.6 . |

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| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|--------------|---|------------------|--------|------------------------------|---|
| 8.4 | Station Blackout | Initial Issuance | Mar-07 | II.1, II.2 | Conforms. Addressed in DCD Section 15.5.5. |
| | | | | II.3 | Not applicable. Onsite Class 1E Emergency AC power sources are not required for ESBWR safe shutdown. |
| | | | | II.4, II.5 | Conforms. Addressed in Section 17.6 . |
| Appendix 8-A | General Agenda, Station Site Visits | Rev. 1 | Mar-07 | | Not applicable. Provides guidance to NRC to conduct site visits. |
| BTP 8-1 | Requirements on Motor-Operated Valves in the ECCS Accumulator Lines | Rev. 3 | Mar-07 | | Not applicable. The ESBWR does not have any safety-related motor-operated valves. |
| BTP 8-2 | Use of Diesel-Generator Sets for Peaking | Rev. 3 | Mar-07 | | Not applicable. The ESBWR can achieve safe shutdown without AC power and the diesel-generator sets are not safety-related. Therefore, this BTP is not applicable. |
| BTP 8-3 | Stability of Offsite Power Systems | Rev. 3 | Mar-07 | | Conforms. Stability studies were performed to investigate the loss of off-site generation. |
| BTP 8-4 | Application of the Single Failure Criterion to Manually Controlled Electrically Operated Valves | Rev. 3 | Mar-07 | | Not applicable. The ESBWR does not use any manually-operated valves to mitigate an accident. |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|--|--------|--------|--|---|
| BTP 8-5 | Supplemental Guidance for Bypass and Inoperable Status Indication for Engineered Safety Features Systems | Rev. 3 | Mar-07 | | Not applicable. The ESBWR is designed in accordance with ICSB 21, the predecessor to BTP 8-5, as stated in DCD Table 8.1-1 and DCD Section 8.3.2.2.2. Also, refer to DCD Table 7.1-1 for conformance to RG 1.47 and Bypass and Inoperable Status Indicator (BISI) for all safety-related systems. |
| BTP 8-6 | Adequacy of Station Electric Distribution System Voltages | Rev. 3 | Mar-07 | | Not applicable. The ESBWR is designed in accordance with PSB 1, the predecessor to BTP 8-6, as stated in DCD Table 8.1-1 and DCD Section 8.3.1.1.2. |
| BTP 8-7 | Criteria for Alarms and Indications Associated with Diesel-Generator Unit Bypassed and Inoperable Status | Rev. 3 | Mar-07 | | Not applicable. The ESBWR does not use safety-related diesel generators. |
| 9.1.1 | Criticality Safety of Fresh and Spent Fuel Storage and Handling | Rev. 3 | Mar-07 | II.1 | Conforms |
| 9.1.2 | New and Spent Fuel Storage | Rev. 4 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9 | Conforms |
| 9.1.3 | Spent Fuel Pool Cooling and Cleanup System | Rev. 2 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7 II.8 | Conforms Conforms. EP-ITAAC are addressed in COLA Part 10. |
| 9.1.4 | Light Load Handling System (Related to Refueling) | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms |
| 9.1.5 | Overhead Heavy Load Handling Systems | Rev. 1 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms |

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| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|--|--------|--------|--|---|
| 9.2.1 | Station Service Water System | Rev. 5 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6 | Conforms |
| 9.2.2 | Reactor Auxiliary Cooling Water Systems | Rev. 4 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6 | Conforms |
| 9.2.3 | Demineralized Water Makeup System | | | | SRP withdrawn |
| 9.2.4 | Potable and Sanitary Water Systems | Rev. 3 | Mar-07 | II.1.A, II.1.B, II.1.C | Conforms |
| 9.2.5 | Ultimate Heat Sink | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5 | Conforms |
| 9.2.6 | Condensate Storage Facilities | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9 | Conforms |
| 9.3.1 | Compressed Air System | Rev. 2 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6 | Conforms. Instrument Air is addressed in DCD Section 9.3.6, Service Air is addressed in DCD Section 9.3.7, and High Pressure Nitrogen Supply System is addressed in DCD Section 9.3.8. |
| 9.3.2 | Process and Post-accident Sampling Systems | Rev. 3 | Mar-07 | II.1, II.3, II.4 II.2 | Conforms Exception. Technical Specifications do not require analyses. Subsection 9.3.2 addresses actions required to qualify process sampling for taking radioactive samples without having a specific post-accident sampling system. Analyses and frequencies of process systems are addressed in plant operating procedures. |
| 9.3.3 | Equipment and Floor Drainage System | Rev. 3 | Mar-07 | II.1, II.2, II.3 | Conforms |
| 9.3.4 | Chemical and Volume Control System (PWR) (Including Boron Recovery System) | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|--|--------|--------|---|--|
| 9.3.5 | Standby Liquid Control System (BWR) | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5 | Conforms |
| 9.4.1 | Control Room Area Ventilation System | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6 | Conforms. Section 9.4 was evaluated against these criteria. |
| 9.4.2 | Spent Fuel Pool Area Ventilation System | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms |
| 9.4.3 | Auxiliary and Radwaste Area Ventilation System | Rev. 3 | Mar-07 | II.1, II.2, II.3 | Conforms. Section 9.4 was evaluated against these criteria. |
| 9.4.4 | Turbine Area Ventilation System | Rev. 3 | Mar-07 | II.1, II.2, II.3 | Conforms |
| 9.4.5 | Engineered Safety Feature Ventilation System | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6 | Conforms |
| 9.5.1 | Fire Protection Program | Rev. 5 | Mar-07 | II.1, II.2, II.4 | Not applicable. See DCD Table 1.9-21. |
| | | | | II.3, II.5, II.6 | Conforms |
| | | | | II.7 | Exception: The elements of the Fire Protection Program required to be operational prior to receipt of new fuel are those elements necessary to protect buildings storing new fuel and adjacent fire areas that could affect the fuel storage area. Other required elements of the Fire Protection Program will be fully operational prior to initial fuel loading. Refer to Section 13.4 . |
| 9.5.2 | Communications Systems | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12, II.13, II.14 | Conforms |
| 9.5.3 | Lighting Systems | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms |

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| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|--------------------|--|------------|-------------|-------------------------------------|-----------------------------|
| 9.5.4 | Emergency Diesel Engine Fuel Oil Storage and Transfer System | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| 9.5.5 | Emergency Diesel Engine Cooling Water System | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| 9.5.6 | Emergency Diesel Engine Starting System | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| 9.5.7 | Emergency Diesel Engine Lubrication System | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| 9.5.8 | Emergency Diesel Engine Combustion Air Intake and Exhaust System | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| 10.2 | Turbine Generator | Rev. 3 | Mar-07 | II.1.A, II.1.B | Conforms |

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| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|-------------------------------|-----|------|------------------------------|---|
| 10.2 | Turbine Generator (continued) | | | II.1.C | <p>Exception—The Turbine Generator Set (TGS) has the capability to permit periodic testing of all components important to safety while the unit is at or above rated speed. In DCD Section 10.2.2.7, a list of components that may be tested with the unit at load is provided. However, some load reduction may be necessary before testing main stop and control valves, and intermediate stop and intercept valves (see DCD Section 10.2.3.7). Overspeed trip testing is performed at speed levels greater than or equal to rated speed with no electrical load. Thus, not all components are capable of being tested at rated load as required in the corresponding Acceptance Criterion.</p> <p>Load reduction for turbine valve testing is common in the existing fleet of power reactors and is considered acceptable. Testing at turbine loads below the rated load condition is considered an acceptable means of confirming that equipment relied on to prevent turbine overspeed related failures is available and capable of providing required functions. Further, component redundancies, as described in DCD Section 10.2.2.4, ensure that a single failure of any of the above valves important to safety will not disable the function of the overspeed protection system.</p> |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|-------------------------------|-----|------|------------------------------|---|
| 10.2 | Turbine Generator (continued) | | | II.2.A | Exception—Inservice inspection of main steam and reheat valves is discussed in DCD Sections 10.2.2.7 and 10.2.3.7. The first disassembly and visual inspection of all main stop valves, main control valves, intermediate stop, and intercept valves are performed within the first three refueling shutdowns. However, the interval for subsequent inspections may be extended beyond the SRP interval of 3-1/3 years to an interval consistent with applicable industry guidance, subject to the requirements of the turbine missile probability analysis. The inspection interval may not exceed the requirements or assumptions in the turbine missile probability analysis. Further, inspection intervals are only extended if there are no significant findings in the initial (baseline) inspections. Thus, with the above provisions, extending the inspection interval beyond the SRP interval is considered acceptable. |
| | | | | II.2.B, II.3 | Conforms |

Table 1.9-201 Conformance with Standard Review Plan (Sheet 26 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|--------------------------------------|--------|--------|---|---|
| 10.2.3 | Turbine Rotor Integrity | Rev. 2 | Mar-07 | II.1, II.2 | Conforms |
| | | | | II.3.A | Exception - DCD Section 10.2.3.5 states that, "Forgings are rough-machined with minimum stock allowance prior to heat treatment." This statement meets the intent of the corresponding SRP Acceptance Criterion. The exception to the Acceptance Criterion is introduced with the reference to welded rotors. The GE N3R-6F52 steam turbine selected for this site utilizes integral forgings in the rotor design and fabrication. Although other manufacturers produce welded rotors, the GE N3R-6F52 rotor is not a welded rotor design and does not utilize welding to construct the base rotor. Flaws in the forging may be repaired by welding and other means, but only after heat treatment. Thus, the intent of this Acceptance Criterion is met. |
| | | | | II.3.B, II.3.C, II.3.D, II.4, II.5 | Conforms |
| 10.3 | Main Steam Supply System | Rev. 4 | Mar-07 | II.1, II.2, II.3,II.5, II.6, II.7, II.8 | Conforms |
| | | | | II.4 | Not applicable to the ESBWR |
| 10.3.6 | Steam and Feedwater System Materials | Rev. 3 | Mar-07 | II.1, II.2 | Conforms |
| 10.4.1 | Main Condensers | Rev. 3 | Mar-07 | II.1 | Conforms |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|----------------------------------|--------|--------|--------------------------------------|---|
| 10.4.2 | Main Condenser Evacuation System | Rev. 3 | Mar-07 | II.1 | Exception: ESBWR design is based on "Standards for Steam Surface Condensers," 10 th Ed., Heat Exchanger Institute (2006) |
| 10.4.3 | Turbine Gland Sealing System | Rev. 3 | Mar-07 | | Conforms |
| 10.4.4 | Turbine Bypass System | Rev. 3 | Mar-07 | II.1, II.2, II.3 | Conforms |
| 10.4.5 | Circulating Water System | Rev. 3 | Mar-07 | II.1 | Conforms |
| 10.4.6 | Condensate Cleanup System | Rev. 3 | Mar-07 | II.1 | Conforms |
| | | | | II.2 | Not applicable to the ESBWR |
| 10.4.7 | Condensate and Feedwater System | Rev. 4 | Mar-07 | II.1, II.2.B, II.3, II.4, II.5, II.6 | Conforms |
| | | | | II.2.A | Not applicable to the ESBWR |
| | | | | II.7 | Exception: This SRP acceptance criterion states that guidance for acceptable FAC inspection programs "is found in (NRC) Generic Letter 89-08 and in EPRI NP-3944." EPRI document NSAC-202L, Rev. 2, supersedes EPRI NP-3944 and is therefore referenced in place of EPRI NP-3944 in DCD Section 6.6.7, for guidance regarding FAC (erosion corrosion) monitoring and related inspection programs. The more recent document, EPRI NSAC-202L, utilizes more extensive industry experience and improved inspection methods and modeling. The substitution of EPRI NSAC-202L, Rev. 2, in place of EPRI NP-3944 is therefore acceptable. |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|---|--------|--------|--|---|
| 10.4.7 | Condensate and Feedwater System (Continued) | | | II.8 | Conforms. Addressed in DCD Sections 3.9.3, 5.2.4, and 10.4.7, and DCD Tables 1.9-22 and 1.11-1. |
| 10.4.8 | Steam Generator Blowdown System (PWR) | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| 10.4.9 | Auxiliary Feedwater System (PWR) | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| BTP 10-1 | Design Guidelines for Auxiliary Feedwater System Pump Drive and Power Supply Diversity for Pressurized Water Reactor Plants | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| BTP 10-2 | Design Guidelines for Avoiding Water Hammers in Steam Generators | Rev. 4 | Mar-07 | | Not applicable to the ESBWR |
| 11.1 | Source Terms | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.6, II.7, II.8, II.9 | Conforms. For acceptance criteria II.9, an alternate computer code to the BWR GALE code is used as described in the DCD. Addressed in DCD Section 12.2 and in FSAR Section 12.2 . |
| | | | | II.5 | Conforms. Addressed in Section 11.2 and 11.3 |
| 11.2 | Liquid Waste Management System | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5 | Conforms. Addressed in DCD Sections 11.2 and 12.2, and in Section 11.2 and 12.2 . |
| | | | | II.6 | Not applicable. Applies to ESP applications. |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|---|--------|--------|--|--|
| 11.3 | Gaseous Waste Management System | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7 | Conforms. Addressed in DCD Sections 11.3 and 12.2, and in Section 11.2 and 12.2 . |
| | | | | II.8 | Not applicable. Applies to ESP applications. |
| 11.4 | Solid Waste Management System | Rev. 3 | Mar-07 | II.1, II.2, II.5, II.7, II.8, II.9, II.10, II.14 | Conforms. |
| | | | | II.3, II.4, II.6, II.11, II.12, II.13 | Conforms (addressed in DCD Section 11.4 and in Section 11.4 ; for Acceptance Criterion II.13, this is also addressed in Section 11.5) with the following exception: RG 1.206, Section 13.4 includes the PCP as an operational program, and only requires a program description in the COLA and a milestone for full program implementation. The FSAR provides a description of the PCP, along with the implementation milestone. Procedures for handling waste will be developed once the PCP is implemented. |
| 11.5 | Process and Effluent Radiological Monitoring Instrumentation and Sampling Systems | Rev. 4 | Mar-07 | II.1, II.2 | Conforms (addressed in DCD Section 11.5.2) with the following exception: Procedural controls are based on NQA-1, rather than RG 1.33, as described in Section 13.5 . Quality Assurance Program requirements are addressed in Section 17.5 . |

Table 1.9-201 Conformance with Standard Review Plan (Sheet 30 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|--|-----|------|------------------------------|--|
| 11.5 | Process and Effluent Radiological Monitoring Instrumentation and Sampling Systems (Continued) | | | II.3, II.4, II.5 | Conforms (addressed in DCD Sections 11.5.2 and 11.5.3, and in Section 11.5) with the following exceptions: 1) RG 1.206, Section 13.4 includes the ODCM (including the SREC) and PCP as operational programs, and only requires program descriptions in the COLA and milestones for full program implementation. The FSAR provides descriptions of the PCP and ODCM along with implementation milestones. 2) Procedural controls are based on NQA-1, rather than RG 1.33, as described in Section 13.5 . Quality Assurance Program requirements are addressed in Section 17.5 . Conformance with NUREG-0718 is addressed in DCD Table 1.9-8. |
| | | | | II.6 | Conforms |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|---|--------|--------|------------------------------|--|
| BTP 11-3 | Design Guidance for Solid Radioactive Waste Management Systems Installed in Light-Water-Cooled Nuclear Power Reactor Plants | Rev. 3 | Mar-07 | B.1,B.3, B.5 B.2, B.4 | Conforms Conforms (addressed in DCD Section 11.4 and in Section 11.4 ; for Acceptance Criterion II.13, this is also addressed in Section 11.5) with the following exception: RG 1.206, Section 13.4 includes the PCP as an operational program, and only requires a program description in the COLA and a milestone for full program implementation. The FSAR provides a description of the PCP, along with the implementation milestone. Procedures for handling waste will be developed once the PCP is implemented. |
| BTP 11-5 | Postulated Radioactive Releases Due to a Waste Gas System Leak or Failure | Rev. 3 | Mar-07 | | Conforms. Addressed in DCD Section 11.3. |
| BTP 11-6 | Postulated Radioactive Releases Due to Liquid-containing Tank Failures | Rev. 3 | Mar-07 | | Conforms. Addressed in DCD Section 15.3.16 and in Subsection 2.4.13 . |
| 12.1 | Assuring that Occupational Radiation Exposures Are As Low As Is Reasonably Achievable | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4 | Conforms. Addressed in Section 13.2 , and Appendix 12AA and Appendix 12BB . |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|--------------------------------------|--------|--------|------------------------------|--|
| 12.2 | Radiation Sources | Rev. 3 | Mar-07 | II.1 | Not applicable. Acceptance criterion cites RG 1.3. SRP states RG 1.3 is applicable to license holders issued prior to January 10, 1997. COL applicant is not a license holder. |
| | | | | II.2 | Not applicable to the ESBWR |
| | | | | II.3 | Conforms. Addressed in DCD Sections 12.3 and 15.4 and in Section 6.4 . |
| | | | | II.4 | Conforms. Addressed in DCD Sections 12.3. |
| | | | | II.5 | Conforms |
| | | | | II.6 | Conforms. Addressed in DCD Appendix 1A and DCD Section 12.2. |
| | | | | II.7 | Conforms. Addressed in DCD Section 12.2. |
| 12.3–12.4 | Radiation Protection Design Features | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5 | Conforms |

Table 1.9-201 Conformance with Standard Review Plan (Sheet 33 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|--|--------|--------|--|---|
| 12.5 | Operational Radiation Protection Program | Rev. 3 | Mar-07 | II.1 | Conforms with the following exceptions: 1) NUREG-0731 is not active, and is not utilized; 2) RG 8.8 specifies the use of RG 1.16. Reporting per C.1.b(2) and C.1.b(3) of RG 1.16 is no longer required. |
| | | | | II.2.A, II.2.B, II.2.C, II.2.D, II.2.E.i, II.2.E.ii, II.2.E.iii, II.2.E.iv, II.2.F, II.2.G, II.2.H, II.4 | Conforms |
| | | | | II.2.E.v | Conforms with the following exception: NUREG-1736 states that RGs 8.20, 8.26, and 8.32 are outdated and recommends use of the methods in RG 8.9, Rev. 1. Therefore, the methods identified in RG 8.9, Rev. 1 will be used in place of those in RGs 8.20, 8.26, and 8.32. |
| | | | | II.3 | Conforms with the following exceptions: 1) RG 8.25 is not applicable to power stations; 2) NUREG-1736 states that RGs 8.20, 8.26, and 8.32 are outdated and recommends use of the methods in RG 8.9, Rev. 1; and 3) RP program and procedures are established, implemented, maintained, and reviewed under the QA Program described in Section 17.5 . |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|---------------|---|--------|--------|---|---|
| 13.1.1 | Management and Technical Support Organization | Rev. 5 | Mar-07 | II.1.A, B, D, II.2.A.i through II.2.A.v | Conforms. |
| | | | | II.1.C | Exception: The experience requirements of corporate staff are set by corporate policy and not provided in detail; however, the experience level of Detroit Edison, as discussed in Section 13.1 and Appendix 13AA , in the area of nuclear plant development, construction, and management establishes that Detroit Edison has the necessary capability and staff to ensure that design and construction of the facility will be performed in an acceptable manner. |
| | | | | II.2.A.vi, II.2.A.vii | Conforms. Addressed in Section 13.1 and 14.2 . |
| | | | | II.2.A.viii | Not applicable. Only applies to applicants whose applications were pending as of February 16, 1982. |
| 13.1.2–13.1.3 | Operating Organization | Rev. 6 | Mar-07 | General 1 | Conforms. |
| 13.1.2–13.1.3 | Operating Organization (Continued) | | | General 2, General 3 | Conforms |
| | | | | General 4 | Not applicable. There are no requests for exemptions from the requirements of 10 CFR 50.54(m). |

Table 1.9-201 Conformance with Standard Review Plan (Sheet 35 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|---|--------|--------|---|--|
| | | | | II.1.A, II.1.B | Conforms with the following exception: Section 17.5 states, "The operational phase quality assurance program requirements will be established through the Company's commitment to ANSI/ASME NQA-1-1994 as described within this QAPD. This edition of NQA-1 contains overall quality assurance requirements equivalent to those of ANSI N18.7-1976, and the Company has included within this QAPD the required administrative controls from ANSI N18.7-1976. Therefore, the Company does not commit to compliance with the requirements of ANSI N18.7-1976/ANS-3.2." |
| | | | | II.1.A.i through II.1.A.v, II.1.C, II.1.E, II.1.F, II.1.G | Conforms |
| | | | | II.1.D | Not applicable |
| | | | | II.1.H | Conforms. Addressed in Section 13.2 . |
| 13.2.1 | Reactor Operator Requalification Program: Reactor Operator Training | Rev. 3 | Mar-07 | II.1.A.i | Conforms. Addressed in Section 13.1 . |
| | | | | II.1.A.ii, II.1.A.iii, II.1.A.v, II.1.B, II.1.D, II.1.E | Conforms |

Table 1.9-201 Conformance with Standard Review Plan (Sheet 36 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|--|--------|--------|--|--|
| 13.2.1 | Reactor Operator Requalification Program: Reactor Operator Training (Continued) | | | II.1.A.iv | Conforms. Addressed in Section 13.1 , 13.2 , and 17.5 . |
| | | | | II.1.A.vi | Conforms. Addressed in DCD Chapter 18. |
| | | | | II.1.A.vii | Exception: The COLA incorporates by reference approved industry template NEI 06-13, which does not address compliance with NUREG-1021. |
| | | | | II.1.C | Exception: This item states that “formal segments of the initial licensed operator training program should be substantially complete when the pre-operational program test begins.” Appendix 13BB commits to a similar state of readiness: “Before initial fuel loading, the number of persons trained in preparation for RO and SRO licensing examinations will be sufficient to meet regulatory requirements, with allowances for examination contingencies and without the need for planned overtime.” |
| 13.2.2 | Non-Licensed Plant Staff Training | Rev. 3 | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.7, II.8, II.9 | Conforms. |

Table 1.9-201 Conformance with Standard Review Plan (Sheet 37 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|---|--------|--------|--|---|
| 13.2.2 | Non-Licensed Plant Staff Training (Continued) | | | II.6 | Exception. This item states that “formal segments of the initial training program should be substantially complete when the pre-operational test program begins.” Appendix 13BB commits to a similar state of readiness: “Before initial fuel loading, sufficient plant staff will be trained to provide for safe plant operations.” |
| | | | | II.10 | Conforms. Addressed in DCD Section 9.5.1. |
| | | | | II.11 | Conforms. Addressed in Sections 13.2 and 13.4 |
| 13.3 | Emergency Planning | Rev. 3 | Mar-07 | II.1, II.2, | Conforms. Addressed in Section 13.4 , COLA Part 5, and COLA Part 10. |
| | | | | II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II.11, II.12, II.13, II.17, II.18, II.27, II.28, II.29, II.30 | Conforms. Addressed in COLA Part 5. |
| | | | | II.14 | Not applicable. Allows NRC to issue a license when applicant asserts that noncompliance with offsite EP requirements is because state or local government has declined to participate in emergency planning. |
| | | | | II.15, II.16, II.19, II.20, II.21 | Not applicable. Only applies to ESP applications. |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|--|------------------|--------|---|--|
| 13.3 | Emergency Planning (Continued) | | | II.22 | Not applicable. Only applies to design certification applications. |
| | | | | II.23, II.24 | Conforms. Addressed in COLA Part 10. |
| | | | | II.25 | Conforms. Addressed in DCD Section 13.3 and COLA Part 5. The Fermi 2 EOF will be used for Fermi 3. |
| | | | | II.26 | Conforms. Reviewed under SRPs 7.5 and 18.2. |
| | | | | II.31 | Conforms. Addressed in Section 13.4 |
| 13.4 | Operational Programs | Rev. 3 | Mar-07 | | Conforms |
| 13.5.1.1 | Administrative Procedures - General | Initial Issuance | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7 | Conforms |
| | | | | II.8 | Section 13.5 and DCD Section 18.9 discuss conformance with NUREG-0711 |
| | | | | II.9, II.10, II.11, II.12, II.13, II.14, II.15, II.16, II.17, II.18, II.19, II.20 | Conforms |
| 13.5.2.1 | Operating and Emergency Operating Procedures | Rev. 2 | Mar-07 | II.1 | Conforms |
| | | | | II.2.A, II.2.B | Conforms |
| | | | | II.2.C | Section 13.5 and DCD Section 18.9 discuss conformance with NUREG-0711 |
| | | | | II.2.D, II.2.E, II.2.F, II.2.G, II.2.H, II.2.I | Conforms |
| 13.6 | Physical Security | Rev. 3 | Mar-07 | | Addressed in COLA Part 8. |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|--------------------|--|------------------|-------------|---|--|
| 13.6.1 | Physical Security - Combined License Review Responsibilities | Initial Issuance | Mar-07 | | Addressed in COLA Part 8. |
| 13.6.2 | Physical Security - Design Certification | Initial Issuance | Mar-07 | | Not applicable. Applies to design certification applications. |
| 13.6.3 | Physical Security - Early Site Permit | Initial Issuance | Mar-07 | | Not applicable. Applies to ESP applications. |
| 14.2 | Initial Plant Test Program - Design Certification and New License Applicants | Rev. 3 | Mar-07 | 1A, 1B, 1C, 2A, COL/OL Applicants: 3A, 3B, 3C, 3D, 3E, 3F, 3G, 3H, 4A, 4B, 5A, 5B, 5C, 5D, 6A, 6B, 6C | Conforms with the following exception: Refer to Table 1.9-202 for exceptions to RG 1.68. |
| | | | | DC Applicants: 3A, 3B, 3C, 3D, 4A, 6A, 6B, 6C | Not applicable. Applies to DC applicants. |
| 14.2.1 | Generic Guidelines for Extended Power Uprate Testing Programs | Initial Issuance | Aug-06 | | Not applicable. Applies to power uprates. |
| 14.3 | Inspections, Tests, Analyses, and Acceptance Criteria | Initial Issuance | Mar-07 | II.1, II.2 | Conforms |
| 14.3.1 | [Reserved] | [Reserved] | Mar-07 | | Not used |
| 14.3.2 | Structural and Systems Engineering - Inspections, Tests, Analyses, and Acceptance Criteria | Initial Issuance | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II.9, II.10, II. 11 | Conforms |
| 14.3.3 | Piping Systems and Components - Inspections, Tests, Analyses, and Acceptance Criteria | Initial Issuance | Mar-07 | II.1, II.2.A, II.2.B, II.2.C, II.2.D, II.2.E | Conforms |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|--------------------|--|------------------|-------------|--|-------------------|
| 14.3.4 | Reactor Systems - Inspections, Tests, Analyses, and Acceptance Criteria | Initial Issuance | Mar-07 | II.1, II.2, II.3, II.4, II.5 | Conforms |
| 14.3.5 | Instrumentation and Controls - Inspections, Tests, Analyses, and Acceptance Criteria | Initial Issuance | Mar-07 | II.1, II.2, II.3, II.4, II.5 | Conforms |
| 14.3.6 | Electrical Systems - Inspections, Tests, Analyses, and Acceptance Criteria | Initial Issuance | Mar-07 | Class 1E Equipment: II.1, II.2, II.3, II.4, II.5 Other Electrical Equipment Important to Safety: II.1, II.2, II.3, II.4, II.5 | Conforms |
| 14.3.7 | Plant Systems - Inspections, Tests, Analyses, and Acceptance Criteria | Initial Issuance | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7, II.8, II. 9 | Conforms |
| 14.3.8 | Radiation Protection - Inspections, Tests, Analyses, and Acceptance Criteria | Initial Issuance | Mar-07 | II.1, II.2, II.3 | Conforms |
| 14.3.9 | Human Factors Engineering - Inspections, Tests, Analyses, and Acceptance Criteria | Initial Issuance | Mar-07 | II.1, II.2, II.3, II.4, II.5, II.6 | Conforms |
| 14.3.10 | Emergency Planning - Inspections, Tests, Analyses, and Acceptance Criteria | Initial Issuance | Mar-07 | II.1, II.2 | Conforms |
| 14.3.11 | Containment Systems - Inspections, Tests, Analyses, and Acceptance Criteria | Initial Issuance | Mar-07 | II.1, II.2, II.3, II.4, II.5 | Conforms |
| 14.3.12 | Physical Security Hardware - Inspections, Tests, Analyses, and Acceptance Criteria | Initial Issuance | Mar-07 | II.1 | Conforms |
| 15 | Introduction - Transient and Accident Analyses | Rev. 3 | Mar-07 | I.1, I.2, 1.3, I.4, I.5, I.6 | Conforms |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|--------------------|--|------------------|-------------|--|---|
| 15.0.1 | Radiological Consequence Analyses Using Alternative Source Terms | Rev. 0 | Jul-00 | V | Conforms |
| 15.0.2 | Review of Transient and Accident Analysis Method | Rev. 0 | Dec-05 | II.1, II.2, II.3, II.4, II.5, II.6 | Conforms |
| 15.0.3 | Design Basis Accident Radiological Consequences of Analyses for Advanced Light Water Reactors | Initial Issuance | Mar-07 | | Conforms |
| 15.1.1– 15.1.4 | Decrease in Feedwater Temperature, Increase in Feedwater Flow, Increase in Steam Flow, and Inadvertent Opening of a Steam Generator Relief or Safety Valve | Rev. 2 | Mar-07 | II.1, II.2, II.3, II.4, II.5, 1, 2, 3, 4 | Conforms |
| 15.1.5 | Steam System Piping Failures Inside and Outside of Containment (PWR) | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| 15.1.5.A | Radiological Consequences of Main Steam Line Failures Outside Containment of a PWR | | | | Not applicable to the ESBWR |
| 15.2.1– 15.2.5 | Loss of External Load; Turbine Trip; Loss of Condenser Vacuum; Closure of Main Steam Isolation Valve (BWR); and Steam Pressure Regulator Failure (Closed) | Rev. 2 | Mar-07 | 1A, 1B, 1C, 1D, 2A, 2B, 2D, 2E, 2F, 3A, 3B, 3C, 3D 2C | Conforms Not applicable. This is not an event of moderate frequency. |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------------|---|--------|--------|--|---|
| 15.2.6 | Loss of Nonemergency AC Power to the Station Auxiliaries | Rev. 2 | Mar-07 | II.1, II.2, II.4, II.5, II.5B, II.5C, II.5D | Conforms |
| | | | | II.3 | Not applicable. This is not an event of moderate frequency. |
| | | | | II.5A | Not applicable. There are no RCS loops in the ESBWR. |
| 15.2.7 | Loss of Normal Feedwater Flow | Rev. 2 | Mar-07 | 1A, 1B, 1C, 1D, 2A, 2B, 2D, 2E, 2F, 3A, 3B, 3C, 3D | Conforms |
| | | | | 2C | Not applicable. This is not an event of moderate frequency. |
| 15.2.8 | Feedwater System Pipe Breaks Inside and Outside Containment (PWR) | Rev. 2 | Mar-07 | | Not applicable to the ESBWR |
| 15.3.1– 15.3.2 | Loss of Forced Reactor Coolant Flow Including Trip of Pump Motor and Flow Controller Malfunctions | Rev. 2 | Mar-07 | | Not applicable to the ESBWR |
| 15.3.3– 15.3.4 | Reactor Coolant Pump Rotor Seizure and Reactor Coolant Pump Shaft Break | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| 15.4.1 | Uncontrolled Control Rod Assembly Withdrawal from a Subcritical or Low Power Startup Condition | Rev. 3 | Mar-07 | 1A, 1C | Conforms |
| | | | | 1B | Not applicable to the ESBWR |
| 15.4.2 | Uncontrolled Control Rod Assembly Withdrawal at Power | Rev. 3 | Mar-07 | 1A, 1C | Conforms |
| | | | | 1B | Not applicable to the ESBWR |
| 15.4.3 | Control Rod Misoperation (System Malfunction or Operator Error) | Rev. 3 | Mar-07 | 1, 2, 3 | Conforms |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|--------------------|--|------------|-------------|-------------------------------------|---|
| 15.4.4 - 15.4.5 | Startup of an Inactive Loop or Recirculation Loop at an Incorrect Temperature, and Flow Controller Malfunction Causing an Increase in BWR Core Flow Rate | Rev. 2 | Mar-07 | 1A, 1B, 1D, 1E, 1F, 1, 2, 3, 4 | Conforms |
| | | | | 1C | Not applicable. This is not an event of moderate frequency. |
| 15.4.6 | Inadvertent Decrease in Boron Concentration in the Reactor Coolant System (PWR) | Rev. 2 | Mar-07 | | Not applicable to the ESBWR |
| 15.4.7 | Inadvertent Loading and Operation of a Fuel Assembly in an Improper Position | Rev. 2 | Mar-07 | 1, 2 | Conforms |
| 15.4.8 | Spectrum of Rod Ejection Accidents (PWR) | Rev. 3 | Mar-07 | | Not applicable to the ESBWR |
| 15.4.8.A | Radiological Consequences of a Control Rod Ejection Accident (PWR) | | | | Not applicable to the ESBWR |
| 15.4.9 | Spectrum of Rod Drop Accidents (BWR) | Rev. 3 | Mar-07 | 1, 2, 3 | Conforms. Postulated events are not applicable to the ESBWR. |
| 15.4.9.A | Radiological Consequences of Control Rod Drop Accident (BWR) | Rev 2 | July 81 | | Conforms. Postulated control rod drop events are not applicable to the ESBWR. |
| 15.5.1– 15.5.2 | Inadvertent Operation of ECCS and Chemical and Volume Control System Malfunction that Increases Reactor Coolant Inventory | Rev. 2 | Mar-07 | 1, 2, 3 | Conforms |

Table 1.9-201 Conformance with Standard Review Plan (Sheet 44 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|---|--------|---------|---|---|
| 15.6.1 | Inadvertent Opening of a PWR Pressurizer Pressure Relief Valve or a BWR Pressure Relief Valve | Rev. 2 | Mar-07 | 1, 2, 3, A, B, C, D | Conforms |
| 15.6.2 | Radiological Consequences of the Failure of Small Lines Carrying Primary Coolant Outside Containment | Rev. 2 | Jul-81 | II.1, II.2 | Conforms |
| 15.6.3 | Radiological Consequences of Steam Generator Tube Failure | | | | Not applicable to the ESBWR |
| 15.6.4 | Radiological Consequences of Main Steam Line Failure Outside Containment (BWR) | Rev. 2 | Jul-81 | II.1, II.2, II.3 | Conforms |
| | | | | II.4 | Conforms. Addressed in TS 3.4.3. |
| 15.6.5 | Loss-of-Coolant Accidents Resulting From Spectrum of Postulated Piping Breaks Within the Reactor Coolant Pressure Boundary | Rev. 3 | Mar-07 | II.1A, II.1B, II.1C, II.1D, II.1E, II.2, II.3 | Conforms. |
| 15.6.5.A | Radiological Consequences of a Design Basis Loss-of-Coolant Accident Including Containment Leakage Contribution | Rev 1 | July 81 | | Not Applicable. Reference DCD Table 1.9-20. |
| 15.6.5.B | Radiological Consequences of a Design Basis Loss-of-Coolant Accident: Leakage From Engineered Safety Feature Components Outside Containment | Rev 1 | July 81 | | Not Applicable. Reference DCD Table 1.9-20. |

Table 1.9-201 Conformance with Standard Review Plan (Sheet 45 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|--|------------------|---------|--|--|
| 15.6.5.D | Radiological Consequences of a Design Basis Loss-of-Coolant Accident: Leakage From Main Steam Isolation Valve Leakage Control System (BWR) | Rev 1 | July 81 | | Not Applicable. Reference DCD Table 1.9-20. |
| 15.7.3 | Postulated Radioactive Releases Due to Liquid-Containing Tank Failures | | | 1, 2 | Conforms |
| 15.7.4 | Radiological Consequences of Fuel Handling Accidents | Rev. 2 | Jul-81 | II.1, II.2, II.3, II.4, II.5 | Conforms. Radiological assumptions superseded by SRP 15.0.1. |
| 15.7.5 | Spent Fuel Cask Drop Accidents | Rev. 2 | July 81 | II.1, II.2, II.3, II.4, II.5 | Conforms. Because a spent fuel cask drop exceeding 9.2 m (30 ft) is not postulated (DCD Section 15.4.10.1), per SRP 15.7.5 a design basis radiological analysis is not required. Therefore, the acceptance criteria do not apply even though the SRP does. |
| 15.8 | Anticipated Transients Without Scram | Rev. 2 | Mar-07 | 1A | Not applicable. ESBWR does not have recirculation pumps. |
| | | | | 1B, 1C, 1D, 1E | Conforms |
| | | | | 1F | Conforms |
| 15.9 | Boiling Water Reactor Stability | Initial Issuance | Mar-07 | 1, 2, 3, 4A, 4B, 5, 6, 7, 9A, 9B, 9C, 10, 11 | Conforms |
| | | | | 8, 9D | Conforms |
| 16 | Technical Specifications | Rev. 2 | Mar-07 | | Conforms |

Table 1.9-201 Conformance with Standard Review Plan (Sheet 46 of 47)

[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|--|------------------|--------|--|---|
| 16.1 | Risk-informed Decision Making: Technical Specifications | Rev. 1 | Mar-07 | | Not applicable |
| 17.1 | Quality Assurance During the Design and Construction Phases | Rev. 2 | Jul-81 | | Not applicable. RG 1.206 refers the COL applicant to Section 17.5 for the format and content of a QA Program for design and construction of new plants. |
| 17.2 | Quality Assurance During the Operations Phase | Rev. 2 | Jul-81 | | Not applicable. RG 1.206 refers the COL applicant to Section 17.5 for the format and content of a QA Program for design and construction of new plants. |
| 17.3 | Quality Assurance Program Description | Rev. 0 | Aug-90 | | Not applicable. RG 1.206 refers the COL applicant to Section 17.5 for the format and content of a QA Program for design and construction of new plants. |
| 17.4 | Reliability Assurance Program (RAP) | Initial Issuance | Mar-07 | II.B.1, II.B.2, II.B.3, II.B.4, II.B.5, II.B.6, II.B.7, II.B.8, II.B.9 | Conforms. Addressed in DCD Section 17.4 and in Section 17.6 . |
| 17.5 | Quality Assurance Program Description - Design Certification, Early Site Permit and New License Applicants | Initial Issuance | Mar-07 | II.A, II.B, II.C, II.D., II.E, II.F, II.G, II.H, II.I, II.J, II.K, II.L, II.M, II.N, II.O, II.P, II.Q, II.R, II.S, II.T, II.U, II.V, II.W Option 1 | Conforms |
| | | | | II.W Option II | Not applicable for Fermi 3. Option I chosen. |
| 17.6 | Maintenance Rule | Initial Issuance | Mar-07 | II.1, II.2 | Conforms |

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[EF3 COL 1.9-3-A]

| SRP Section | Title | Rev | Date | Specific Acceptance Criteria | Evaluation |
|-------------|--|--------|--------|--|--|
| 18 | Human Factors Engineering | Rev. 2 | Mar-07 | II.A | Conforms |
| | | | | II.B, II.C | Not applicable. These acceptance criteria apply to changes to existing plants. |
| 19.0 | Probabilistic Risk Assessment and Severe Accident Evaluation for New Reactors | Rev. 2 | Jun-07 | II.1, II.2, II.3, II.4, II.5, II.6, II.7 | Conforms |
| | | | | II.8, II.9 | Not applicable. Only applies to Westinghouse AP 600 design. |
| 19.1 | Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities | Rev. 2 | Jun-07 | | Not applicable. There are no plans for risk-informed activities. |
| 19.2 | Review of Risk Information Used to Support Permanent Plant Specific Changes to the Licensing Basis: General Guidelines | Rev. 0 | Jun-07 | | Not applicable. There are no plans for risk-informed applications. |

Table 1.9-202 Conformance with Regulatory Guides (Sheet 1 of 26)

[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|--|----------|--------|-------------|---|
| 1.1 | Net Positive Suction Head for Emergency Core Cooling and Containment Heat Removal System Pumps | Rev. 0 | Nov-70 | General | Not applicable |
| 1.3 | Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss of Coolant Accident for Boiling Water Reactors | Rev. 2 | Jun-74 | General | Not applicable. RG 1.183 is used. |
| 1.4 | Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss of Coolant Accident for Pressurized Water Reactors | Rev. 2 | Jun-74 | General | Not applicable |
| 1.5 | Assumptions Used for Evaluating the Potential Radiological Consequences of a Steam Line Break Accident for Boiling Water Reactors | Rev. 0 | Mar-71 | General | Not applicable. RG 1.183 is used. |
| 1.6 | Independence Between Redundant Standby (Onsite) Power Sources and Between Their Distribution Systems | Rev. 0 | Mar-71 | General | The ESBWR Standard Plant does not need or have any safety-related standby AC power sources. RG 1.6 is only applicable as applied to the safety-related DC power sources and to the nonsafety-related onsite and offsite power sources as related to independence and redundancy between their sources and distribution systems. |
| 1.7 | Control of Combustible Gas Concentrations in Containment | Rev. 3 | Mar-07 | General | Conforms |
| 1.8 | Qualification and Training of Personnel for Nuclear Power Plants | Rev. 3 | May-00 | | See App 17AA, QAPD, Part IV |

Table 1.9-202 Conformance with Regulatory Guides (Sheet 2 of 26)

[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|---|----------|--------|--------------------------|---|
| 1.9 | Application and Testing of Safety-Related Diesel Generators in Nuclear Power Plants | Rev. 4 | Mar-07 | General | Not applicable |
| 1.11 | Instrument Lines Penetrating Primary Reactor Containment (Safety Guide 11) Supplement to Safety Guide 11, Backfitting Considerations | Rev. 0 | Feb-72 | C.1, C.2, E | Conforms |
| 1.12 | Nuclear Power Plant Instrumentation for Earthquakes | Rev. 2 | Mar-97 | C.1, C.4 – C.7, C.3, C.8 | Conforms Conforms. The seismic monitoring program, including the necessary test and operating procedures, will be implemented prior to receipt of fuel on site. |
| 1.13 | Spent Fuel Storage Facility Design Basis | Rev. 2 | Mar-07 | General | Conforms |
| 1.14 | Reactor Coolant Pump Flywheel Integrity | Rev. 1 | Aug-75 | General | Not applicable |
| 1.16 | Reporting of Operating Information—Appendix A Technical Specifications | Rev. 4 | Aug-75 | General | Conforms with the following exceptions: Reporting per C.1.b(2) and C.1.b(3) is no longer required. |
| 1.20 | Comprehensive Vibration Assessment Program for Reactor Internals During Preoperational and Initial Startup Testing | Rev. 3 | Mar-07 | C.1 C.2 C.3 | Conforms. Conforms. Conforms. |
| 1.21 | Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants | Rev. 1 | Jun-74 | General | Conforms. Subsection 11.4.2.3 (NEI 07-10A) and Subsection 11.5.4.5 (NEI 07-09A) provide descriptions of the PCP and ODCM, respectively. Implementation milestones are provided in Section 13.4 . |

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[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|--------------|--|----------|--------|----------------|---|
| 1.22 | Periodic Testing of Protection System Actuation Functions | Rev. 0 | Feb-72 | General | Conforms. Operational program implementation is described in Section 13.4 . |

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[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|---|----------|--------|-------------|---|
| 1.23 | Meteorological Monitoring Programs For Nuclear Power Plants | Rev. 1 | Mar-07 | General | <p>Exception: The meteorological monitoring program for pre-operational and operational phases complies with RG 1.23, Rev. 1. The meteorological monitoring program used for pre-application complies for the most part, with RG 1.23, Revision 0 and Draft Revision 1 (Sept. 1980). Specific areas where the pre-application monitoring program do not comply with RG 1.23 Rev. 1, are discussed in Subsection 2.3.3.1. Specific areas where the pre-application monitoring program do not comply with RG 1.23, Rev. 0, and Draft Ref. 1 (1980), and also do not comply with RG 1.23, Rev. 1 (2007), are as follows: The RG in part requires that sensors should be located at a distance of at least 10 times the height of any nearby obstruction if the height of the obstruction exceeds one-half the height of the wind measurement. This criterion is not met for the existing meteorological tower at Fermi 2 and relocation of the tower would be required for construction of Fermi 3 (Refer to Subsection 2.3.3.1.1). In addition, the proximity of trees to the existing meteorological tower does not meet this criterion. This is addressed in Subsection 2.3.3.1.6. Calibration of wind direction sensor does not include test for starting threshold. Refer to Subsection 2.3.3.1.3 for discussion.</p> |

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[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|--|----------|--------|-------------|--|
| 1.24 | Assumptions Used for Evaluating the Potential Radiological Consequences of a Pressurized Water Reactor Radioactive Gas Storage Tank Failure | Rev. 0 | Mar-72 | All | Not applicable |
| 1.25 | Assumptions Used for Evaluating the Potential Radiological Consequences of a Fuel Handling Accident in the Fuel Handling and Storage Facility for Boiling and Pressurized Water Reactors | Rev. 0 | Mar-72 | General | Not applicable. RG 1.183 is used. |
| 1.26 | Quality Group Classifications and Standards for Water-, Steam-, and Radioactive-Waste-Containing Components of Nuclear Power Plants | Rev. 4 | Mar-07 | | See App 17AA, QAPD, Part IV |
| 1.27 | Ultimate Heat Sink for Nuclear Power Plants | Rev. 2 | Jan-76 | General | The UHS is within the scope of the referenced certified design and is addressed in DCD Section 9.2.5. |
| 1.28 | Quality Assurance Program Requirements (Design and Construction) | Rev. 3 | Aug-85 | | See App 17AA, QAPD, Part IV |
| 1.29 | Seismic Design Classification | Rev. 4 | Mar-07 | | See App 17AA, QAPD, Part IV |
| 1.30 | Quality Assurance Requirements for the Installation, Inspection, and Testing of Instrumentation and Electric Equipment | Rev. 0 | Aug-72 | General | Exception: The QAPD identified in Section 17.5 addresses a QA program based on a newer NQA-1-1994, as discussed in SRP 17.5. |
| 1.31 | Control of Ferrite Content in Stainless Steel Weld Metal | Rev. 3 | Apr-78 | General | Conforms. Operational program implementation is described in Section 13.4 . |

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[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|---|----------|--------|-------------|--|
| 1.32 | Criteria for Power Systems for Nuclear Power Plants | Rev. 3 | Mar-04 | General | Conforms. |
| 1.33 | Quality Assurance Program Requirements (Operation) | Rev. 2 | Feb-78 | | See App 17AA, QAPD, Part IV |
| 1.34 | Control of Electroslag Weld Properties | Rev. 0 | Dec-72 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 1.35 | Inservice Inspection of Ungrouted Tendons in Prestressed Concrete Containments | Rev. 3 | Jul-90 | General | Not applicable |
| 1.35.1 | Determining Prestressing for Inspection of Prestressed Concrete Containments | Rev. 0 | Jul-90 | General | Not applicable |
| 1.36 | Nonmetallic Thermal Insulation for Austenitic Stainless Steel | Rev. 0 | Feb-73 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 1.37 | Quality Assurance Requirements for Cleaning of Fluid Systems and Associated Components of Water-Cooled Nuclear Power Plants | Rev. 1 | Mar-07 | | See App 17AA, QAPD, Part IV |
| 1.38 | Quality Assurance Requirements for Packaging, Shipping, Receiving, Storage, and Handling of Items for Water-Cooled Nuclear Power Plants | Rev. 2 | May-77 | General | Exception. Section 17.5 identifies equivalent quality assurance standards. |
| 1.39 | Housekeeping Requirements for Water-Cooled Nuclear Power Plants | Rev. 2 | Sep-77 | General | Exception. Section 17.5 identifies equivalent quality assurance standards. |
| 1.40 | Qualification Tests of Continuous-Duty Motors Installed Inside the Containment of Water-Cooled Nuclear Power Plants | Rev. 0 | Mar-73 | General | Conforms |

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[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|---|----------|--------|-------------|---|
| 1.41 | Preoperational Testing of Redundant On-Site Electric Power Systems to Verify Proper Load Group Assignments | Rev. 0 | Mar-73 | General | Conforms with the following exception: There are no safety-related DGs for ESBWR. |
| 1.43 | Control of Stainless Steel Weld Cladding of Low-Alloy Steel Components | Rev. 0 | May-73 | General | Conforms |
| 1.44 | Control of the Use of Sensitized Stainless Steel | Rev. 0 | May-73 | General | Conforms. Operational program implementation is described in Section 13.4 . |
| 1.45 | Reactor Coolant Pressure Boundary Leakage Detection Systems | Rev. 0 | May-73 | General | Conforms. Operational program implementation is described in Section 13.4 . |
| 1.47 | Bypassed and Inoperable Status Indication for Nuclear Power Plant Safety Systems | Rev. 0 | May-73 | General | Conforms. Operational program implementation is described in Section 13.4 . |
| 1.50 | Control of Preheat Temperature for Welding of Low-Alloy Steel | Rev. 0 | May-73 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 1.52 | Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Post-Accident Engineered-Safety-Feature Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants | Rev. 3 | Jun-01 | General | Conforms |
| 1.53 | Application of the Single-Failure Criterion to Safety Systems | Rev. 2 | Nov-03 | General | Conforms |
| 1.54 | Service Level I, II, and III Protective Coatings Applied to Nuclear Power Plants | Rev. 1 | Jul-00 | | See App 17AA, QAPD, Part IV |
| 1.56 | Maintenance of Water Purity in Boiling Water Reactors | Rev. 1 | Jul-78 | General | Conforms. |

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[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|----------------------|---|-----------------|-------------|------------------------|--|
| 1.57 | Design Limits and Loading Combinations for Metal Primary Reactor Containment System Components | Rev. 1 | Mar-07 | General | Conforms |
| 1.59 | Design Basis Floods for Nuclear Power Plants (Errata Published 7/30/80) | Rev. 2 | Aug-77 | General | Conforms |
| 1.60 | Design Response Spectra for Seismic Design of Nuclear Power Plants | Rev. 1 | Dec-73 | General | Conforms |
| 1.61 | Damping Values for Seismic Design of Nuclear Power Plants | Rev. 1 | Mar-07 | General | Conforms |
| 1.62 | Manual Initiation of Protective Actions | Rev. 0 | Oct-73 | General | Conforms |
| 1.63 | Electric Penetration Assemblies in Containment Structures for Nuclear Power Plants | Rev. 3 | Feb-87 | General | Conforms |
| 1.65 | Materials and Inspections for Reactor Vessel Closure Studs | Rev. 0 | Oct-73 | General | Conforms |
| 1.68 | Initial Test Programs for Water-Cooled Nuclear Power Plants | Rev. 2 | Aug-78 | General | Conforms with the following exception: Equipment listed in Appendix A, Items 1.k(2) and 1.k(3) not included in the initial test program. |
| 1.68.1 | Preoperational and Initial Startup Testing of Feedwater and Condensate Systems for Boiling Water Reactor Power Plants | Rev. 1 | Jan-77 | General | Conforms |
| 1.68.2 | Initial Startup Test Program to Demonstrate Remote Shutdown Capability for Water-Cooled Nuclear Power Plants | Rev. 1 | Jul-78 | General | Conforms |
| 1.68.3 | Preoperational Testing of Instrument and Control Air Systems | Rev. 0 | Apr-82 | General | Conforms |

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[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|--|----------|--------|-------------|---|
| 1.69 | Concrete Radiation Shields for Nuclear Power Plants | Rev. 0 | Dec-73 | General | Conforms |
| 1.70 | Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants LWR Edition | Rev. 3 | Nov-78 | -- | Not applicable. RG 1.206 is used. Table 1.9-203 . |
| 1.71 | Welder Qualification for Areas of Limited Accessibility | Rev. 1 | Mar-07 | General | Conforms. Operational program implementation is described in Section 13.4 . |
| 1.72 | Spray Pond Piping Made from Fiberglass-Reinforced Thermosetting Resin | Rev. 2 | Nov-78 | General | Not applicable |
| 1.73 | Qualification Tests of Electric Valve Operators Installed Inside the Containment of Nuclear Power Plants | Rev. 0 | Jan-74 | General | Conforms |
| 1.75 | Criteria for Independence of Electrical Safety Systems | Rev. 3 | Feb-05 | General | Conforms |
| 1.76 | Design Basis Tornado and Tornado Missiles for Nuclear Power Plants | Rev. 1 | Mar-07 | General | Conforms |
| 1.77 | Assumptions Used for Evaluating a Control Rod Ejection Accident for Pressurized Water Reactors | Rev. 0 | May-74 | General | Not applicable |
| 1.78 | Evaluating the Habitability of a Nuclear Power Plant Control Room During a Postulated Hazardous Chemical Release | Rev. 1 | Dec-01 | General | Conforms |
| 1.79 | Preoperational Testing of Emergency Core Cooling Systems for Pressurized Water Reactors | Rev. 1 | Sep-75 | General | Not applicable |
| 1.81 | Shared Emergency and Shutdown Electric Systems for Multi-Unit Nuclear Power Plants | Rev. 1 | Jan-75 | General | Not applicable |

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[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|------------------|---|-----------------|-------------|--------------------|---|
| 1.82 | Water Sources for Long-Term Recirculation Cooling Following a Loss-of-Coolant Accident | Rev. 3 | Nov-03 | General | Not applicable |
| 1.83 | Inservice Inspection of Pressurized Water Reactor Steam Generator Tubes | Rev. 1 | Jul-75 | General | Not applicable |
| 1.84 | Design, Fabrication, and Materials Code Case Acceptability, ASME Section III | Rev. 34 | Oct 07 | General | Conforms. Code Case N-782,s also applied as described in the Comments Section for RG 1.84 in Table 1.9-21 of the DCD. |
| 1.86 | Termination of Operating Licenses for Nuclear Reactors | Rev. 0 | Jun-74 | General | This RG is outside the scope of the FSAR. |
| 1.87 | Guidance for Construction of Class 1 Components in Elevated-Temperature Reactors (Supplement to ASME Section III Code Cases 1592, 1593, 1594, 1595, and 1596) | Rev. 1 | Jun-75 | General | Not applicable |
| 1.89 | Environmental Qualification of Certain Electric Equipment Important to Safety for Nuclear Power Plants | Rev. 1 | Jun-84 | General | Conforms. Source terms from RG 1.183 used. |
| 1.90 | Inservice Inspection of Prestressed Concrete Containment Structures with Grouted Tendons | Rev. 1 | Aug-77 | General | Not applicable |
| 1.91 | Evaluations of Explosions Postulated to Occur on Transportation Routes Near Nuclear Power Plants | Rev. 1 | Feb-78 | General | Conforms |
| 1.92 | Combining Modal Responses and Spatial Components in Seismic Response Analysis | Rev. 2 | Jul-06 | General | Conforms |

Table 1.9-202 Conformance with Regulatory Guides (Sheet 11 of 26)

[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|--|----------|--------|-------------|--|
| 1.93 | Availability of Electric Power Sources | Rev. 0 | Dec-74 | General | Conforms with the following exception: The ESBWR is designed to shut down safely without reliance on offsite or diesel-generator-derived AC power, therefore, the regulatory guide is only applicable to onsite safety-related DC power systems. |
| 1.94 | Quality Assurance Requirements for Installation, Inspection, and Testing of Structural Concrete and Structural Steel During the Construction Phase of Nuclear Power Plants | Rev. 1 | Apr-76 | General | Exception. Section 17.5 identifies equivalent QA standards in NQA-1, Subpart 2.5. |
| 1.96 | Design of Main Steam Isolation Valve Leakage Control Systems for Boiling Water Reactor Nuclear Power Plants | Rev. 1 | Jun-76 | General | Not applicable |
| 1.97 | Criteria for Accident Monitoring Instrumentation for Nuclear Power Plants | Rev. 4 | Jun-06 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 1.98 | Assumptions Used for Evaluating the Potential Radiological Consequences of a Radioactive Offgas System Failure in a Boiling Water Reactor | Rev. 0 | Mar-76 | General | Not applicable. Superseded by BTP 11-5. |
| 1.99 | Radiation Embrittlement of Reactor Vessel Materials | Rev. 2 | May-88 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 1.100 | Seismic Qualification of Electric and Mechanical Equipment for Nuclear Power Plants | Rev. 2 | Jun-88 | General | Conforms |

Table 1.9-202 Conformance with Regulatory Guides (Sheet 12 of 26)

[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|--|----------|--------|-------------|---|
| 1.101 | Emergency Response Planning and Preparedness for Nuclear Power Reactors | Rev. 3 | Aug-05 | General | Conforms except Fermi 3 Emergency Plan utilizes NEI 07-01, Rev. 0 for EALs instead of Appendix 1 of NUREG-0654/FEMA-REP-1 |
| 1.102 | Flood Protection for Nuclear Power Plants | Rev. 1 | Sep-76 | General | Conforms |
| 1.105 | Setpoints For Safety-Related Instrumentation | Rev. 3 | Dec-99 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 1.106 | Thermal Overload Protection for Electric Motors on Motor-Operated Valves | Rev. 1 | Feb-77 | General | Not applicable |
| 1.107 | Qualifications for Cement Grouting for Prestressing Tendons in Containment Structures | Rev. 1 | Feb-77 | General | Not applicable |
| 1.109 | Calculation of Annual Doses to Man from Routine Releases of Reactor Effluents for the Purpose of Evaluating Compliance with 10 CFR Part 50, Appendix I | Rev. 1 | Oct-77 | General | Conforms |
| 1.110 | Cost-Benefit Analysis for Radwaste Systems for Light-Water-Cooled Nuclear Power Reactors | Rev. 0 | Mar-76 | General | Conforms |
| 1.111 | Methods for Estimating Atmospheric Transport and Dispersion of Gaseous Effluents in Routine Releases from Light-Water-Cooled Reactors | Rev. 1 | Jul-77 | General | Conforms |
| 1.112 | Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Light-Water-Cooled Nuclear Power Reactors | Rev. 1 | Mar-07 | General | Conforms except the suggested breakdown identified in Appendix A to the RG is not used because it is not consistent with the DCD presentation of information. |

Table 1.9-202 Conformance with Regulatory Guides (Sheet 13 of 26)

[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|--|----------|--------|-------------|---|
| 1.113 | Estimating Aquatic Dispersion of Effluents from Accidental and Routine Reactor Releases for the Purpose of Implementing Appendix I | Rev. 1 | Apr-77 | General | Conforms |
| 1.114 | Guidance to Operators at the Controls and to Senior Operators in the Control Room of a Nuclear Power Unit | Rev. 2 | May-89 | General | Conforms |
| 1.115 | Protection Against Low-Trajectory Turbine Missiles | Rev. 1 | Jul-77 | General | Conforms |
| 1.116 | Quality Assurance Requirements for Installation, Inspection, and Testing of Mechanical Equipment and Systems | Rev. 0 | May-77 | General | Exception: Section 17.5 identifies equivalent QA standards in NQA-1, Subpart 2.8. |
| 1.117 | Tornado Design Classification | Rev. 1 | Apr-78 | General | Conforms |
| 1.118 | Periodic Testing of Electric Power and Protection Systems | Rev. 3 | Apr-95 | General | Conforms. Operational program implementation is described in Section 13.4 . |
| 1.121 | Bases for Plugging Degraded PWR Steam Generator Tubes | Rev. 0 | Aug-76 | General | Not applicable |
| 1.122 | Development of Floor Design Response Spectra for Seismic Design of Floor-Supported Equipment or Components | Rev. 1 | Feb-78 | General | Conforms |
| 1.124 | Service Limits and Loading Combinations for Class 1 Linear-Type Supports | Rev. 2 | Feb-07 | General | Conforms |
| 1.125 | Physical Models for Design and Operation of Hydraulic Structures and Systems for Nuclear Power Plants | Rev. 1 | Oct-78 | General | Conforms |
| 1.126 | An Acceptable Model and Related Statistical Methods for the Analysis of Fuel Densification | Rev. 1 | Mar-78 | General | Conforms |

Table 1.9-202 Conformance with Regulatory Guides (Sheet 14 of 26)

[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|------------------|--|-----------------|-------------|--|---|
| 1.127 | Inspection of Water-Control Structures Associated with Nuclear Power Plants | Rev. 1 | Mar-78 | General | Conforms |
| 1.128 | Installation Design and Installation of Vented Lead-Acid Storage Batteries for Nuclear Power Plants | Rev. 2 | Feb-07 | General | Conforms |
| 1.129 | Maintenance, Testing, and Replacement of Vented Lead-Acid Storage Batteries for Nuclear Power Plants | Rev. 2 | Feb-07 | General | Conforms |
| 1.130 | Service Limits and Loading Combinations for Class 1 Plate-and-Shell-Type Supports | Rev. 2 | Mar-07 | General | Conforms |
| 1.131 | Qualification Tests of Electric Cables, Field Splices, and Connections for Light-Water-Cooled Nuclear Power Plants | Rev. 0 | Aug-77 | General | Conforms |
| 1.132 | Site Investigations for Foundations of Nuclear Power Plants | Rev. 2 | Oct-03 | C.1, C.2, C.3, C.4.1 - C.4.2, C.4.4, C.5 - C.7 | Conforms. |
| | | | | C4.3.2.5 | Properties of borrow materials not investigated. Cat I structures are on bedrock. |
| | | | | C.4.5 | Some borehole logging required reconciliation of final results |
| | | | | C.6 | Not applicable |
| 1.133 | Loose-Part Detection Program for the Primary System of Light Water Cooled Reactors | Rev. 1 | May-81 | General | Not applicable |

Table 1.9-202 Conformance with Regulatory Guides (Sheet 15 of 26)

[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|--|----------|--------|-------------|--|
| 1.134 | Medical Evaluation of Licensed Personnel for Nuclear Power Plants | Rev. 3 | Mar-98 | General | Conforms. Although RG 1.134 is not specifically identified in the FSAR, equivalent requirements for medical evaluations for licensed personnel are embedded in policies and procedures of operations and training departments. |
| 1.135 | Normal Water Level and Discharge at Nuclear Power Plants | Rev. 0 | Sep-77 | General | Not applicable. |
| 1.136 | Design Limits, Loading Combinations, Materials, Construction, and Testing of Concrete Containments | Rev. 3 | Mar-07 | General | Conforms |
| 1.137 | Fuel-Oil Systems for Standby Diesel Generators | Rev. 1 | Oct-79 | General | Not applicable |
| 1.138 | Laboratory Investigations of Soils and Rocks for Engineering Analysis and Design of Nuclear Power Plants | Rev. 2 | Dec-03 | General | Conforms |
| 1.139 | Guidance for Residual Heat Removal | Rev. 0 | May-78 | General | Conforms |
| 1.140 | Design, Inspection, and Testing Criteria for Air Filtration and Adsorption Units of Normal Atmosphere Cleanup Systems in Light-Water-Cooled Nuclear Power Plants | Rev. 2 | Jun-01 | General | Conforms. Operational program implementation is described in Section 13.4 . |
| 1.141 | Containment Isolation Provisions for Fluid Systems | Rev. 0 | Apr-78 | General | Conforms |
| 1.142 | Safety-Related Concrete Structures for Nuclear Power Plants (Other Than Reactor Vessels and Containments) | Rev. 2 | Nov-01 | General | Conforms |

Table 1.9-202 Conformance with Regulatory Guides (Sheet 16 of 26)

[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|---|----------|--------|-------------|---|
| 1.143 | Design Guidance for Radioactive Waste Management Systems, Structures, and Components Installed in Light-Water-Cooled Nuclear Power Plants | Rev. 2 | Nov-01 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 1.145 | Atmospheric Dispersion Models for Potential Accident Consequence Assessments at Nuclear Power Plants | Rev. 1 | Nov-82 | General | Conforms |
| 1.147 | Inservice Inspection Code Case Acceptability, ASME Section XI, Division 1 | Rev. 15 | Oct 07 | General | Conforms. Operational program implementation is described in Section 13.4 . |
| 1.148 | Functional Specification for Active Valve Assemblies in Systems Important to Safety in Nuclear Power Plants | Rev. 0 | Mar-81 | General | Conforms |
| 1.149 | Nuclear Power Plant Simulation Facilities for Use in Operator Training and License Examinations | Rev. 3 | Oct-01 | General | Conforms |
| 1.150 | Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice Examinations | Rev. 1 | Feb-83 | General | Conforms. Operational program implementation is described in Section 13.4 . |
| 1.151 | Instrument Sensing Lines | Rev. 0 | Jul-83 | General | Conforms. Operational program implementation is described in Section 13.4 . |
| 1.152 | Criteria for Use of Computers in Safety Systems of Nuclear Power Plants | Rev. 2 | Jan-06 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 1.153 | Criteria for Safety Systems | Rev. 1 | Jun-96 | General | Conforms |
| 1.154 | Format and Content of Plant-Specific Pressurized Thermal Shock Safety Analysis Reports for Pressurized Water Reactors | Rev. 0 | Jan-87 | General | Not applicable |

Table 1.9-202 Conformance with Regulatory Guides (Sheet 17 of 26)

[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|--|----------|--------|-------------|---|
| 1.155 | Station Blackout | Rev. 0 | Aug-88 | General | Conforms, except no emergency AC power is required for the ESBWR. Only the coping analysis is applicable. Operational program implementation is described in Section 13.4 |
| 1.156 | Environmental Qualification of Connection Assemblies for Nuclear Power Plants | Rev. 0 | Nov-87 | General | Conforms |
| 1.157 | Best-Estimate Calculations of Emergency Core Cooling System Performance | Rev. 0 | May-89 | General | Conforms |
| 1.158 | Qualification of Safety-Related Lead Storage Batteries for Nuclear Power Plants | Rev. 0 | Feb-89 | General | Conforms |
| 1.159 | Assuring the Availability of Funds for Decommissioning Nuclear Reactors | Rev. 1 | Oct-03 | General | Conforms. The amount of funds for decommissioning and the method of financial assurance is described in COLA Part 1. |
| 1.160 | Monitoring the Effectiveness of Maintenance at Nuclear Power Plants | Rev. 2 | Mar-97 | General | Conforms. Operational program implementation is described in Section 13.4 . |
| 1.161 | Evaluation of Reactor Pressure Vessels with Charpy Upper-Shelf Energy Less Than 50 Ft.-Lb. | Rev. 0 | Jun-95 | General | Not applicable. |
| 1.162 | Format and Content of Report for Thermal Annealing of Reactor Pressure Vessels | Rev. 0 | Feb-96 | General | This RG is outside the scope of the FSAR. |
| 1.163 | Performance-Based Containment Leak-Test Program | Rev. 0 | Sep-95 | General | Conforms |
| 1.165 | Identification and Characterization of Seismic Sources and Determination of Safe Shutdown Earthquake Ground Motion | Rev. 0 | Mar-97 | General | Conforms. |

Table 1.9-202 Conformance with Regulatory Guides (Sheet 18 of 26)

[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|--|----------|--------|-------------|--|
| 1.166 | Pre-Earthquake Planning and Immediate Nuclear Power Plant Operator Postearthquake Actions | Rev. 0 | Mar-97 | General | Conforms. The seismic monitoring program, including the necessary test and operating procedures, will be implemented prior to receipt of fuel on site. |
| 1.167 | Restart of a Nuclear Power Plant Shut Down by a Seismic Event | Rev. 0 | Mar-97 | General | Not applicable. |
| 1.168 | Verification, Validation, Reviews, and Audits for Digital Computer Software Used in Safety Systems of Nuclear Power Plants | Rev. 1 | Feb-04 | General | Conforms. Procedures addressed in Section 13.5 ITAAC addressed in COLA Part 10. |
| 1.169 | Configuration Management Plans for Digital Computer Software Used in Safety Systems of Nuclear Power Plants | Rev. 0 | Sep-87 | General | Conforms. Procedures addressed in Section 13.5 ITAAC addressed in COLA Part 10. |
| 1.170 | Software Test Documentation for Digital Computer Software Used in Safety Systems of Nuclear Power Plants | Rev. 0 | Sep-97 | General | Conforms. Procedures addressed in Section 13.5 ITAAC addressed in COLA Part 10. |
| 1.171 | Software Unit Testing for Digital Computer Software Used in Safety Systems of Nuclear Power Plants | Rev. 0 | Sep-97 | General | Conforms. Procedures addressed in Section 13.5 ITAAC addressed in COLA Part 10. |
| 1.172 | Software Requirements Specifications for Digital Computer Software Used in Safety Systems of Nuclear Power Plants | Rev. 0 | Sep-97 | General | Conforms. Procedures addressed in Section 13.5 ITAAC addressed in COLA Part 10. |
| 1.173 | Developing Software Life Cycle Processes for Digital Computer Software Used in Safety Systems of Nuclear Power Plants | Rev. 0 | Sep-97 | General | Conforms. Procedures addressed in Section 13.5 ITAAC addressed in COLA Part 10. |

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[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|--|----------|--------|-------------|---|
| 1.174 | An Approach for Using Probabilistic Risk Assessment in Risk-Informed Decisions on Plant-Specific Changes to the Licensing Basis | Rev. 1 | Nov-02 | General | Not applicable. The approach described in this RG is not being used. |
| 1.175 | An Approach for Plant-Specific, Risk-Informed Decisionmaking: Inservice Testing | Rev. 0 | Aug-98 | General | Not applicable. Risk informed inservice testing is not being used. |
| 1.177 | An Approach for Plant-Specific, Risk-Informed Decisionmaking: Technical Specifications | Rev. 0 | Aug-98 | General | Not applicable. Risk informed Technical Specifications are not being used. |
| 1.178 | An Approach For Plant-Specific Risk-informed Decisionmaking Inservice Inspection of Piping | Rev. 0 | Sep-98 | General | Not applicable. Risk informed inservice inspection is not being used. |
| 1.179 | Standard Format and Content of License Termination Plans for Nuclear Power Reactors | Rev. 0 | Jan-99 | General | This RG is outside the scope of the FSAR. |
| 1.180 | Guidelines for Evaluating Electromagnetic and Radio-Frequency Interference in Safety-Related Instrumentation and Control Systems | Rev. 1 | Oct-03 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 1.181 | Content of the Updated Final Safety Analysis Report in Accordance with 10 CFR 50.71(e) | Rev. 0 | Sep-99 | General | Conforms |
| 1.182 | Assessing and Managing Risk Before Maintenance Activities at Nuclear Power Plants | Rev. 0 | May-00 | General | Conforms |
| 1.183 | Alternative Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Reactors | Rev. 0 | Jul-00 | General | Conforms |

Table 1.9-202 Conformance with Regulatory Guides (Sheet 20 of 26)

[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|--|----------|--------|-------------|--|
| 1.184 | Decommissioning of Nuclear Power Reactors | Rev. 0 | Jul-00 | General | Not applicable. The RG provides guidance on how to conduct decommissioning activities. |
| 1.185 | Standard Format and Content for Post-Shutdown Decommissioning Activities Report | Rev. 0 | Jul-00 | General | This RG is outside the scope of the FSAR. |
| 1.186 | Guidance and Examples for Identifying 10 CFR 50.2 Design Bases | Rev. 0 | Oct-00 | General | This RG is outside the scope of the FSAR. |
| 1.187 | Guidance for Implementation of 10 CFR 50.59, Changes, Tests, and Experiments | Rev. 0 | Nov-00 | General | Conforms. |
| 1.188 | Standard Format and Content for Applications to Renew Nuclear Power Plant Operating Licenses | Rev. 1 | Sep-05 | General | This RG is outside the scope of the FSAR. |
| 1.189 | Fire Protection for Nuclear Power Plants | Rev. 1 | Mar-07 | General | Conforms with the following exception. Section C.1.1.c of the RG states that during construction, on sites with an operating unit, the superintendent of the operating plant should have overall responsibility for fire protection. However, due to physical and administrative separation of Fermi 3 from the operating unit, the onsite executive in charge of construction will have overall responsibility for Fermi 3 fire protection during construction. |
| 1.190 | Calculational and Dosimetry Methods for Determining Pressure Vessel Neutron Fluence | Rev. 0 | Mar-01 | General | Conforms. The reactor vessel material surveillance program is described in Subsection 5.3.1.8 . Implementation of the program is described in Section 13.4 |

Table 1.9-202 Conformance with Regulatory Guides (Sheet 21 of 26)

[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|--|----------|--------|-------------|---|
| 1.191 | Fire Protection Program for Nuclear Power Plants During Decommissioning and Permanent Shutdown | Rev. 0 | May-01 | General | This RG is outside the scope of the FSAR. |
| 1.192 | Operation and Maintenance Code Case Acceptability, ASME OM Code | Rev. 0 | Jun-03 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 1.193 | ASME Code Cases Not Approved for Use | Rev. 1 | Aug-05 | General | Conforms |
| 1.194 | Atmospheric Relative Concentrations for Control Room Radiological Habitability Assessments at Nuclear Power Plants | Rev. 0 | Jun-03 | General | Conforms |
| 1.195 | Methods and Assumptions for Evaluating Radiological Consequences of Design Basis Accidents at Light-Water Nuclear Power Reactors | Rev. 0 | May-03 | General | Not applicable. RG 1.183 is used. |
| 1.196 | Control Room Habitability at Light-Water Nuclear Power Reactors | Rev. 1 | Jan-07 | General | Conforms |
| 1.197 | Demonstrating Control Room Envelope Integrity at Nuclear Power Plant Reactors | Rev. 0 | May-03 | General | Conforms |
| 1.198 | Procedures and Criteria for Assessing Seismic Soil Liquefaction At Nuclear Power Plant Sites | Rev. 0 | Nov-03 | General | Conforms |
| 1.199 | Anchoring Components and Structural Supports in Concrete | Rev. 0 | Nov-03 | General | Conforms |
| 1.200 | An Approach for Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities | Rev. 1 | Jan-07 | General | Not applicable |

Table 1.9-202 Conformance with Regulatory Guides (Sheet 22 of 26)

[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|--|----------|--------|-------------|---|
| 1.201 | Guidelines for Categorizing Structures, Systems, and Components in Nuclear Power Plants According to Their Safety Significance | Rev. 1 | May-06 | General | Not applicable |
| 1.202 | Standard Format and Content of Decommissioning Cost Estimates for Nuclear Power Reactors | Rev. 0 | Feb-05 | General | Not applicable. The RG provides guidance for submitting decommissioning cost estimates to NRC prior to license termination. |
| 1.203 | Transient and Accident Analysis Methods | Rev. 0 | Dec-05 | General | Conforms |
| 1.204 | Guidelines for Lightning Protection of Nuclear Power Plants | Rev. 0 | Nov-05 | General | Conforms. Operational program implementation is described in Section 13.4 . |
| 1.205 | Risk-Informed, Performance-Based Fire Protection for Existing Light-Water Nuclear Power Plants | Rev. 0 | May-06 | General | Not applicable. Risk-informed, performance-based fire protection is not used. |
| 1.206 | Combined License Applications for Nuclear Power Plants (LWR Edition) | Rev. 0 | Jun-07 | General | See Table 1.9-203 . |
| 1.207 | Guidelines for Evaluating Fatigue Analyses Incorporating the Life Reduction of Metal Components Due to the Effects of the Light-Water Reactor Environment for New Reactors | Rev. 0 | Mar-07 | General | Conforms |
| 1.208 | A Performance-Based Approach to Define the Site-Specific Earthquake Ground Motion | Rev. 0 | Mar-07 | All | Conforms |

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[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|--|------------------|--------|-------------|---|
| 1.209 | Guidelines for Environmental Qualification of Safety-Related Computer-Based Instrumentation and Control Systems in Nuclear Power Plants | Rev. 0 | Mar-07 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 4.7 | General Site Suitability Criteria for Nuclear Power Stations | Rev. 2 | Apr-98 | General | Conforms. |
| 4.15 | Quality Assurance for Radiological Monitoring Programs (Inception Through Normal Operations to License Termination) – Effluent Streams and the Environment | Rev. 2 (Interim) | Mar-07 | General | Conforms. Subsection 11.5.4.5 (NEI 07-09A) provides a description of the ODCM. The implementation milestone is provided in Section 13.4 |
| 4.21 | Minimization of Contamination and Radioactive Waste Generation: Life-Cycle Planning | Rev 0 | Jun-08 | General | Conforms through implementation of NEI 08-08A. |
| 5.44 | Perimeter Intrusion Alarm Systems | Rev. 3 | Oct-97 | General | Conforms to one test option as discussed in the RG defined by a plant station procedure. |
| 5.62 | Reporting of Safeguards Events | Rev. 1 | Nov-87 | General | Not applicable. Reportability of Safeguards Events is in accordance with 10 CFR 73 Appendix G. |
| 5.66 | Access Authorization Program for Nuclear Power Plants | Rev. 1 | Jul-09 | General | Conforms |
| 5.69 | Guidance for the Application of the Radiological Sabotage Design-Basis Threat in the Design, Development, and Implementation of a Physical Security Program that meets 10 CFR 73.55 Requirements | Rev 0 | Aug-07 | General | Conforms |
| 8.1 | Radiation Symbol | Rev. 0 | Feb-73 | General | Conforms. The facility utilizes standard radiation symbols. |

Table 1.9-202 Conformance with Regulatory Guides (Sheet 24 of 26)

[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|---|----------|--------|-------------|--|
| 8.2 | Guide for Administrative Practices in Radiation Monitoring | Rev. 0 | Feb-73 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 8.4 | Direct-Reading and Indirect-Reading Pocket Dosimeters | Rev. 0 | Feb-73 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 8.5 | Criticality and Other Interior Evacuation Signals | Rev. 1 | Mar-81 | General | Conforms. Operational program implementation is described in Section 13.4 . |
| 8.6 | Standard Test Procedure for Geiger-Muller Counters | Rev. 0 | May-73 | General | Exception: Instrument calibration programs is based upon criteria in ANSI N323A-1997 (with 2004 correction Sheet) "Radiation Protection Instrumentation Test and Calibration, Portable Survey Instruments." The ANSI 42.3-1969 Standard is no longer recognized as sufficient for calibration of modern Instruments. Operational program implementation is described in Section 13.4 |
| 8.7 | Instructions for Recording and Reporting Occupational Radiation Dose Data | Rev. 2 | Nov-05 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 8.8 | Information Relevant to Ensuring that Occupational Radiation Exposures at Nuclear Power Stations Will Be As Low As Is Reasonably Achievable | Rev. 3 | Jun-78 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 8.9 | Acceptable Concepts, Models, Equations, and Assumptions for a Bioassay Program | Rev. 1 | Jul-93 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 8.10 | Operating Philosophy for Maintaining Occupational Radiation Exposures As Low As Is Reasonably Achievable | Rev. 1-R | May-77 | General | Conforms. Operational program implementation is described in Section 13.4 |

Table 1.9-202 Conformance with Regulatory Guides (Sheet 25 of 26)

[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|---|----------|--------|-------------|---|
| 8.11 | Applications of Bioassay for Uranium | Rev. 0 | Jun-74 | General | Not applicable. RG 8.11 has been superseded by RG 8.9, Rev 1. |
| 8.13 | Instruction Concerning Prenatal Radiation Exposure | Rev. 3 | Jun-99 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 8.15 | Acceptable Programs for Respiratory Protection | Rev. 1 | Oct-99 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 8.19 | Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants – Design Stage Man-Rem Estimates | Rev. 1 | Jun-79 | General | Conforms |
| 8.20 | Applications of Bioassay for I-125 and I-131 | Rev. 1 | Sep-79 | General | Exception. Per NUREG-1736, RG 8.20 is outdated. RG 8.9 is used. Operational program implementation is described in Section 13.4 |
| 8.25 | Air Sampling in the Workplace | Rev. 1 | Jun-92 | General | Not applicable |
| 8.26 | Applications of Bioassay for Fission and Activation Products | Rev. 0 | Sep-80 | General | Exception. Per NUREG-1736, RG 8.26 is outdated. RG 8.9 is used. Operational program implementation is described in Section 13.4 |
| 8.27 | Radiation Protection Training for Personnel at Light-Water-Cooled Nuclear Power Plants | Rev. 0 | Mar-81 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 8.28 | Audible-Alarm Dosimeters | Rev. 0 | Jul-81 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 8.29 | Instruction Concerning Risks from Occupational Radiation Exposure | Rev. 1 | Feb-96 | General | Conforms. Operational program implementation is described in Section 13.4 |

Table 1.9-202 Conformance with Regulatory Guides (Sheet 26 of 26)

[EF3 COL 1.9-3-A]

| RG Number | Title | Revision | Date | RG Position | Evaluation |
|-----------|---|----------|--------|-------------|---|
| 8.32 | Criteria for Establishing a Tritium Bioassay Program | Rev. 0 | Jul-88 | General | Exception. Per NUREG-1736, RG 8.32 is outdated. RG 8.9 is used. Operational program implementation is described in Section 13.4 |
| 8.33 | Quality Management Program | Rev. 0 | Oct-91 | General | Not applicable to nuclear power plants. RG 8.33 applies to nuclear medicine. |
| 8.34 | Monitoring Criteria and Methods To Calculate Occupational Radiation Doses | Rev. 0 | Jul-92 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 8.35 | Planned Special Exposures | Rev. 0 | Jun-92 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 8.36 | Radiation Dose to the Embryo/Fetus | Rev. 0 | Jul-92 | General | Conforms. Operational program implementation is described in Section 13.4 |
| 8.38 | Control of Access to High and Very High Radiation Areas of Nuclear Plants | Rev. 1 | May-06 | General | Conforms. Operational program implementation is described in Section 13.4 |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
(Sheet 1 of 39) [EF3 COL 1.9-3-A]

| Section | Section Title | Conformance Evaluation |
|--------------------|--|--|
| C.III.2 1 | Introduction and General Description of the Plant | Conforms |
| C.III.2 1.1 | Introduction | Conforms |
| C.III.2 1.2 | General Plant Description | Conforms. Addressed in Subsection 1.2.2.19 and Section 2.0 , and DCD Figures 1.2-1 through 1.2-33. |
| C.III.2 1.3 | Comparisons with Other Facilities | Conforms |
| C.III.2 1.4 | Identification of Agents and Contractors | Conforms |
| C.III.2 1.5 | Requirements for Further Technical Information | Conforms |
| C.III.2 1.6 | Material Referenced | Conforms |
| C.III.2 1.7 | Drawings and Other Detailed Information | Conforms |
| C.III.2 1.8 | Site and Plant Design Interfaces and Conceptual Design Information | Conforms |
| C. III.2 1.9 | Conformance with Regulatory Criteria | Conforms |
| C.III.2 2.1.1 | Site Location and Description | Conforms |
| C.III.2 2.1.2.1 | Authority | Conforms |
| C.III.2 2.1.2.2 | Control of Activities Unrelated to Plant Operation | Conforms. There are no known significant changes regarding activities unrelated to plant operation within the exclusion area. |
| C.III.2 2.1.2.3 | Arrangements for Traffic Control | Conforms. There are no known significant changes regarding highways, railroads, or waterways that traverse the exclusion area. |
| C.III.2 2.1.2.4 | Abandonment or Relocation of Roads | Conforms. There are no known significant changes regarding any public roads traversing the exclusion area. |
| C.III.2 2.1.3 | Population Distribution | Conforms |
| C.III.2 2.2 | Nearby Industrial, Transportation, and Military Facilities | Conforms |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
(Sheet 2 of 39) [EF3 COL 1.9-3-A]

| Section | Section Title | Conformance Evaluation |
|-------------------|--|--|
| C.III.2 2.3.1 | Regional Climatology | Conforms |
| C.III.2 2.3.2 | Local Meteorology | Conforms |
| C.III.2 2.3.3 | Onsite Meteorological Measurements Program | Conforms. |
| C.III.2 2.3.4 | Short-Term Atmospheric Dispersion Estimates for Accident Releases | Conforms |
| C.III.2 2.3.5 | Long-Term Atmospheric Dispersion Estimates for Routine Releases | Conforms |
| C.III.2 2.4.1 | Hydrologic Description | Conforms |
| C.III.2 2.4.2 | Floods | Conforms |
| C.III.2 2.4.3 | Probable Maximum Flood (PMF) on Streams and Rivers | Conforms |
| C.III.2 2.4.4 | Potential Dam Failures | Conforms |
| C.III.2 2.4.5 | Probable Maximum Surge and Seiche Flooding | Conforms |
| C.III.2 2.4.6 | Probable Maximum Tsunami Hazards | Conforms |
| C.III.2 2.4.7 | Ice Effects | Conforms. |
| C.III.2 2.4.8 | Cooling Water Canals and Reservoirs | Conforms |
| C.III.2 2.4.9 | Channel Diversions | Conforms |
| C.III.2 2.4.10 | Flooding Protection Requirements | Conforms. There are no safety-related SSCs that are not part of the DC facility. |
| C.III.2 2.4.11 | Low Water Considerations | Conforms |
| C.III.2 2.4.12 | Groundwater | Not applicable. A permanent dewatering system is not required. |
| C.III.1 2.4.13 | Accidental Release of Radioactive Liquid Effluent in Ground and Surface Waters | Conforms |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
(Sheet 3 of 39) [EF3 COL 1.9-3-A]

| Section | Section Title | Conformance Evaluation |
|---------------------|---|--|
| C.III.2 2.4.14 | Technical Specifications and Emergency Operation Requirements | Conforms |
| C.III.2 2.5.1 | Basic Geologic and Seismic Information | Conforms |
| C.III.1 2.5.2 | Vibratory Ground Motion | Conforms |
| C.III.2 2.5.3 | Surface Faulting | Conforms |
| C.III.2 2.5.4 | Stability of Subsurface Materials and Foundations | Conforms |
| C.III.1 2.5.4.1 | Geologic Features | Conforms |
| C.III.1 2.5.4.2 | Properties of Subsurface Materials | Conforms |
| C.III.1 2.5.4.3 | Foundation Interfaces | Conforms |
| C.III.1 2.5.4.4 | Geophysical Surveys | Conforms |
| C.III.1 2.5.4.5 | Excavations and Backfill | Conforms with the following exception: Sources of backfill have not been identified. Backfill properties will be verified prior to construction. |
| C.III.1 2.5.4.6 | Ground Water Conditions | Conforms |
| C.III.1 2.5.4.7 | Response of Soil and Rock to Dynamic Loading | Conforms |
| C.III.1 2.5.4.8 | Liquefaction Potential | Conforms |
| C.III.1 2.5.4.9 | Earthquake Site Characteristics | Conforms |
| C.III.1 2.5.4.10 | Static Stability | Conforms |
| C.III.1 2.5.4.11 | Design Criteria | Conforms |
| C.III.1 2.5.4.12 | Techniques to Improve Subsurface Conditions | Conforms |
| C.III.2 2.5.5 | Stability of Slopes | Conforms |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
(Sheet 4 of 39) [EF3 COL 1.9-3-A]

| Section | Section Title | Conformance Evaluation |
|-------------------|---|--|
| C.III.1 3.1 | Conformance with NRC General Design Criteria | Conforms. Conformance with the NRC's criteria in 10 CFR 50, Appendix A, is described in DCD Section 3.1 and the applicable DCD system sections. |
| C.III.1 3.2.1 | Seismic Classification | Conforms. There are no additional safety-related or RTNSS SSCs subject to seismic classification beyond those addressed in the DCD. There are no SSCs outside the referenced certified design that are required to be designed for an OBE. |
| C.III.1 3.2.2 | System Quality Group Classification | Conforms. There are no additional safety-related or RTNSS SSCs subject to system quality group classification beyond those addressed in the DCD. |
| C.III.1 3.3.1 (1) | Wind Loadings | Conforms. There are no safety-related SSCs outside the scope of the certified design. Nonsafety-related facility SSCs that are not included in the referenced certified design meet the requirements of DCD Sections 3.3.1.3 and 3.3.2.3. |
| C.III.1 3.3.1 (2) | Wind Loadings | Conforms |
| C.III.1 3.3.2 | Tornado Loadings | Conforms |
| C.III.1 3.4 | Internal Flood Protection | Conforms. There are no SSCs outside the scope of the referenced certified design that require internal flood protection whose failure could prevent a safe shutdown of the plant or result in the uncontrolled release of significant radioactivity. |
| C.III.1 3.4.2 | Analysis Procedures | Conforms. There are no Seismic Category I structures outside the scope of the referenced certified design. |
| C.III.1 3.5.1.1 | Internally Generated Missiles (Outside Containment) | Conforms. There are no SSCs outside the scope of the referenced certified design that are required to be protected against damage from internally generated missiles. |
| C.III.1 3.5.1.2 | Internally Generated Missiles (Inside Containment) | Conforms. |
| C.III.1 3.5.1.3 | Turbine Missiles | Conforms. Addressed in DCD Section 10.2.3.8. |
| C.III.1 3.5.1.4 | Missiles Generated by Tornadoes and Extreme Winds | Conforms. Table 2.0-201 demonstrates that the site-specific tornado characteristics are bounded by the parameters assumed in the DCD. DCD Section 3.5.1.4 indicates that resistance to missiles is independent of site topography. |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
(Sheet 5 of 39) [EF3 COL 1.9-3-A]

| Section | Section Title | Conformance Evaluation |
|--------------------|---|--|
| C.III.1 3.5.1.5 | Site Proximity Missiles (Except Aircraft) | Conforms |
| C.III.2 3.5.1.6 | Aircraft Hazards | Conforms |
| C.III.1 3.5.2 | Structures, Systems, and Components To Be Protected from Externally Generated Missiles | Conforms. There are no SSCs outside the scope of the referenced certified design that are required to be protected from externally generated missiles. |
| C.III.1 3.5.3 | Barrier Design Procedures | Conforms. There are no SSCs that require reanalysis for tornado, extreme wind, or site proximity missile impact or for aircraft impact. |
| C.III.1 3.6 | Protection against Dynamic Effects Associated with the Postulated Rupture of Piping | Conforms |
| C.III.1 3.6.1 | Plant Design for Protection against Postulated Piping Failures in Fluid systems Outside of Containment | Conforms |
| C.III.1 3.6.2 | Determination of Rupture Locations and Dynamic Effects Associated with the Postulated Rupture of Piping | Conforms |
| C.III.1 3.6.3 | Leak-Before-Break Evaluation Procedures | Not Applicable. ESBWR design does not rely on a Leak Before Break Evaluation. |
| C.III.1 3.7.1 | Seismic Design Parameters | Conforms. Addressed in DCD Sections 3.7 and 3.7.1. |
| C.III.1 3.7.1.1 | Design Ground Motion | Conforms. Addressed in DCD Section 3.7.1.1 and Subsection 2.5.2 . |
| C.III.1 3.7.1.2 | Percentage of Critical Damping Values | Conforms |
| C.III.1 3.7.1.3 | Supporting Media for Seismic Category I Structures | Conforms |
| C.III.1 3.7.2 | Seismic System Analysis | Conforms. Addressed in DCD Section 3.7.2. |
| C.III.1 3.7.2.1 | Seismic Analysis Methods | Conforms |
| C.III.1 3.7.2.2 | Natural Frequencies and Responses | Conforms. Addressed in DCD Section 3.7.2.2. |
| C.III.1 3.7.2.3 | Procedures Used for Analytical Modeling | Conforms |
| C.III.1 3.7.2.4 | Soil/Structure Interaction | Conforms |
| C.III.1 3.7.2.5 | Development of Floor Response Spectra | Conforms. Addressed in DCD Section 3.7.2.5. |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
(Sheet 6 of 39) [EF3 COL 1.9-3-A]

| Section | Section Title | Conformance Evaluation |
|------------------|--|---|
| C.III.1 3.7.2.6 | Three Components of Earthquake Motion | Conforms |
| C.III.1 3.7.2.7 | Combination of Modal Responses | Conforms |
| C.III.1 3.7.2.8 | Interaction of Nonseismic Category I Structures with Seismic Category I Structures | Conforms. There are no Seismic Category I structures outside the scope of the referenced certified design. In lieu of providing the plant-specific distances between structures and the heights of structures, the distance and height requirements for Non-Seismic Category I structures are addressed in DCD Section 3.7.2.8. |
| C.III.1 3.7.2.9 | Effects of Parameter Variations on Floor Response Spectra | Conforms. Addressed in DCD Section 3.7.2.9. |
| C.III.1 3.7.2.10 | Use of Constant Vertical Static Factors | Conforms |
| C.III.1 3.7.2.11 | Method Used to Account for Torsional Effects | Conforms |
| C.III.1 3.7.2.12 | Comparison of Responses | Conforms. Addressed in DCD Section 3.7.2.12. |
| C.III.1 3.7.2.13 | Methods for Seismic Analysis of Dams | Not applicable. There are no Seismic Category I dams in the ESBWR design per DCD Section 3.7.3.14. |
| C.III.1 3.7.2.14 | Determination of Dynamic Stability of Seismic Category I Structures | Conforms. Addressed in DCD Sections 3.7.2.14 and 3.8.5.5. |
| C.III.1 3.7.2.15 | Analysis Procedure for Damping | Conforms |
| C.III.1 3.7.3.1 | Seismic Analysis Methods | Conforms |
| C.III.1 3.7.3.2 | Procedures Used for Analytical Modeling | Conforms |
| C.III.1 3.7.3.3 | Analysis Procedure for Damping | Conforms |
| C.III.1 3.7.3.4 | Three Components of Earthquake Motion | Conforms |
| C.III.1 3.7.3.5 | Combination of Modal Responses | Conforms. Addressed in DCD Section 3.7.3.7. |
| C.III.1 3.7.3.6 | Use of Constant Vertical Static Factors | Conforms |
| C.III.1 3.7.3.7 | Buried Seismic Category I Piping, Conduits, and Tunnels | Conforms. Addressed in DCD Section 3.7.3.13. |
| C.III.1 3.7.3.8 | Methods for Seismic Analysis of Seismic Category I Concrete Dams | Not applicable. There are no Seismic Category I dams for Fermi 3. |
| C.III.1 3.7.3.9 | Methods for Seismic Analysis of Above-Ground Tanks | Conforms. Addressed in DCD Section 3.7.3.15. |
| C.III.1 3.7.4 | Seismic Instrumentation | Conforms |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
(Sheet 7 of 39) [EF3 COL 1.9-3-A]

| Section | Section Title | Conformance Evaluation |
|------------------|---|--|
| C.III.1 3.8.1 | Concrete Containment | Conforms |
| C.III.1 3.8.2 | Steel Containment | Conforms |
| C.III.1 3.8.3 | Concrete and Steel Internal Structures of Steel or Concrete Containments | Conforms |
| C.III.1 3.8.4 | Other Seismic Category I Structures | Conforms. There are no Seismic Category I structures that are outside the scope of the DCD. |
| C.III.1 3.8.5 | Foundations | Conforms |
| C.III.1 3.9.1 | Special Topics for Mechanical Components | Conforms. There are no Seismic Category I components or supports beyond those evaluated in the reference certified design. |
| C.III.1 3.9.1.1 | Design Transients | Conforms. There are no Seismic Category I components or supports beyond those evaluated in the reference certified design. |
| C.III.1 3.9.1.2 | Computer Programs Used in Analysis | Conforms. There are no Seismic Category I components or supports beyond those evaluated in the reference certified design. |
| C.III.1 3.9.1.3 | Experimental Stress Analysis | Conforms. There are no Seismic Category I components or supports beyond those evaluated in the reference certified design. |
| C.III.1 3.9.1.4 | Considerations for the Evaluation of the Faulted Condition | Conforms. There are no Seismic Category I components or supports beyond those evaluated in the reference certified design. |
| C.III.1 3.9.2 | Dynamic Testing and Analysis of Systems, Components, and Equipment | Conforms. There are no systems outside the scope of the referenced certified design that require dynamic testing and analysis. |
| C.III.1 3.9.2.1 | Piping Vibration, Thermal Expansion, and Dynamic Effects | Conforms. There are no ASME Code Class 1, 2, and 3 systems; other high-energy piping systems inside seismic Category I structures; high-energy portions of systems for which failure could reduce the functioning of any seismic Category I plant feature to an unacceptable level; or seismic Category I portions of moderate-energy piping systems located outside containment outside the scope of the referenced certified design. |
| C.III.1 3.9.2.2 | Seismic Analysis and Qualification of Seismic Category I Mechanical Equipment | Conforms |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
(Sheet 8 of 39) [EF3 COL 1.9-3-A]

| Section | Section Title | Conformance Evaluation |
|-----------------|--|--|
| C.III.1 3.9.2.3 | Dynamic Response Analysis of Reactor Internals Under Operational Flow Transients and Steady-State Conditions | Conforms. There are no ESBWR pressure vessel internals that the referenced certified design does not cover. |
| C.III.1 3.9.2.4 | Pre-Operational Flow-Induced Vibration Testing of Reactor Internals | Conforms. There are no BWR pressure vessel internals that the referenced certified design does not cover. DCD Sections 3.9.2.3 and 3.9.2.4 adequately cover the analysis of potential adverse flow effects that could impact BWR vessel internals. |
| C.III.1 3.9.2.5 | Dynamic System Analysis of the Reactor Internals Under Faulted Condition | Conforms. Addressed in DCD Section 3.9.3.1 and DCD Table 3.9-2. |
| C.III.1 3.9.2.6 | Correlations of Reactor Internals Vibration Tests with the Analytical Results | Conforms. Addressed in DCD Section 3.9.2.6. |
| C.III.1 3.9.3 | ASME Code Class 1, 2, and 3 Components and Component Supports, and Core Support Structures | Conforms. There are no pressure-retaining components or component supports designed or constructed in accordance with ASME Code Class 1, 2, or 3, or GDC 1,2,4,14, or 15, beyond those evaluated in the referenced certified design. |
| C.III.1 3.9.4 | Control Rod Drive Systems | Conforms |
| C.III.1 3.9.5.1 | Design Arrangements | Conforms |
| C.III.1 3.9.5.2 | Loading Conditions | Conforms |
| C.III.1 3.9.5.3 | Design Bases | Conforms |
| C.III.1 3.9.5.4 | BWR Reactor Pressure Vessel Internals Including Steam Dryer | Conforms. There are no reactor pressure vessel internals (including the steam dryer) or other main steam system components that are not covered by the referenced certified design. Classification of the reactor internals is described in Section 3.9.2.4. |
| C.III.1 3.9.6.1 | Functional Design and Qualification of Pumps, Valves, and Dynamic Restraints | Conforms. There is no safety-related equipment beyond the scope of the referenced certified design. |
| C.III.1 3.9.6.2 | Inservice Testing Program for Pumps | Not applicable. There are no safety-related pumps. |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
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| Section | Section Title | Conformance Evaluation |
|-------------------|---|---|
| C.III.1 3.9.6.3 | Inservice Testing Program for Valves | Conforms. Addressed in DCD Section 3.9.6; the list of valves included in the IST program is provided in DCD Table 3.9-8. IST Program test procedures and schedules are addressed in Technical Specifications 5.5.5. Justification for cold shutdown and refueling outage test schedules is addressed in DCD Section 3.9.6 and DCD Table 3.9-8. The implementation milestones for the IST and MOV Programs are addressed in Section 13.4 |
| C.III.1 3.9.6.3.1 | Inservice Testing Program for Motor-Operated Valves (MOVs) | Conforms. Addressed in DCD Section 3.9.6. |
| C.III.1 3.9.6.3.2 | Inservice Testing Program for Power-Operated Valves (POVs) Other Than MOVs | Conforms. Addressed in DCD Section 3.9.6. |
| C.III.1 3.9.6.3.3 | Inservice Testing Program for Check Valves | Conforms. Addressed in DCD Section 3.9.6. |
| C.III.1 3.9.6.3.4 | Pressure Isolation Valve (PIV) Leak Testing | Not applicable. The ESBWR plant does not have any PIVs. |
| C.III.1 3.9.6.3.5 | Containment Isolation Valve (CIV) Leak Testing | Conforms |
| C.III.1 3.9.6.3.6 | Inservice Testing Program for Safety and Relief Valves | Conforms. Addressed in DCD Table 3.9-8. |
| C.III.1 3.9.6.3.7 | Inservice Testing Program for Manually Operated Valves | Conforms. Addressed in DCD Table 3.9-8. |
| C.III.1 3.9.6.3.8 | Inservice Testing Program for Explosively Activated Valves | Conforms. Addressed in DCD Table 3.9-8. |
| C.III.1 3.9.6.4 | Inservice Testing Program for Dynamic Restraints | Conforms with the following exception: A plant specific snubber table will be prepared in conjunction with closure of ITAAC Table 3.1-1. |
| C.III.1 3.9.6.5 | Relief Requests and Alternative Authorizations to ASME OM Code | Conforms |
| C.III.1 3.10.1 | Seismic Qualification Criteria | Conforms. There is no seismic or dynamic qualification required for equipment that is outside the scope of the referenced certified design. |
| C.III.1 3.10.2 | Methods and Procedures for Qualifying Mechanical and Electrical Equipment and Instrumentation | Conforms |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
(Sheet 10 of 39) [EF3 COL 1.9-3-A]

| Section | Section Title | Conformance Evaluation |
|------------------|--|---|
| C.III.1 3.10.3 | Methods and Procedures of Analysis or Testing of Supports of Mechanical and Electrical Equipment and Instrumentation | Conforms |
| C.III.1 3.10.4 | Test and Analyses Results and Experience Database | Conforms |
| C.III.1 3.11 | Environmental Qualification of Mechanical and Electrical Equipment | Conforms. There is no other equipment beyond that which has been evaluated in the referenced certified design. |
| C.III.1 3.11.1 | Equipment Location and Environmental Conditions | Conforms |
| C.III.1 3.11.2 | Qualification Tests and Analysis | Conforms |
| C.III.1 3.11.3 | Qualification Test Results | Conforms |
| C.III.1 3.11.4 | Loss of Ventilation | Conforms |
| C.III.1 3.11.5 | Estimated Chemical and Radiation Environment | Conforms |
| C.III.1 3.11.6 | Qualification of Mechanical Equipment | Conforms |
| C.III.1 3.12.1 | Introduction | Conforms |
| C.III.1 3.12.2 | Codes and Standards | Conforms. Addressed in DCD Sections 3.2, 3.6, 3.7, and Chapter 5 and Chapter 14 . |
| C.III.1 3.12.3 | Piping Analysis Methods | Conforms. Addressed in DCD Sections 3.7.2.2 and 3.7.3.9. |
| C.III.1 3.12.3.1 | Experimental Stress Analyses | Conforms. Addressed in DCD Section 3.9.1.3. |
| C.III.1 3.12.3.2 | Modal Response Spectrum Method | Conforms. Addressed in DCD Section 3.7.2.1. |
| C.III.1 3.12.3.3 | Response Spectra Method (or Independent Support Motion Method) | Conforms. Addressed in DCD Section 3.7.2.1.2. |
| C.III.1 3.12.3.4 | Time History Method | Conforms. Addressed in DCD Section 3.7.2.1.1. |
| C.III.1 3.12.3.5 | Inelastic Analyses Method | Not Applicable. Per DCD Section 3.9.1.4 (Inelastic Analyses Methods), except for pipe whip restraints, inelastic analyses methods are not used in the ESBWR piping design and analysis. |
| C.III.1 3.12.3.6 | Small-Bore Piping Method | Conforms. Addressed in DCD Section 3.7.3.16. |
| C.III.1 3.12.3.7 | Nonseismic/Seismic Interaction (II/I) | Conforms with the following exception: The location and distance between piping systems will be established as part of the completion of ITAAC Table 3.1-1. |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
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| Section | Section Title | Conformance Evaluation |
|-------------------|--|---|
| C.III.1 3.12.3.8 | Seismic Category I Buried Piping | Not Applicable. Per DCD Section 3.7.3.13, there is no buried Seismic Category I piping. |
| C.III.1 3.12.4 | Piping Modeling Technique | Conforms. Addressed in DCD Section 3.7.3.3.1 and Appendix 3D for the PISYS computer code. |
| C.III.1 3.12.4.1 | Computer Codes | Conforms. Addressed in DCD Appendix 3D. |
| C.III.1 3.12.4.2 | Dynamic Piping Model | Conforms. Addressed in DCD Section 3.7.3.3.1. |
| C.III.1 3.12.4.3 | Piping Benchmark Program | Conforms. Addressed in DCD Appendix 3D. |
| C.III.1 3.12.4.4 | Decoupling Criteria | Conforms. Addressed in DCD Sections 3.7.2.3 and 3.7.3.16. |
| C.III.1 3.12.5.1 | Seismic Input Envelope vs. Site-Specific Spectra | Conforms. Addressed in DCD Section 3.7.1. |
| C.III.1 3.12.5.2 | Design Transients | Conforms. Addressed in DCD Section 3.9.1.1 and DCD Table 3.9-1. |
| C.III.1 3.12.5.3 | Loadings and Load Combination | Conforms. Addressed in DCD Section 3.9.1.1 and DCD Table 3.9-8. |
| C.III.1 3.12.5.4 | Damping Values | Conforms. Addressed in DCD Section 3.7.1.2 and DCD Table 3.7-1. |
| C.III.1 3.12.5.5 | Combination of Modal Responses | Conforms. Addressed in DCD Section 3.7.3.7. |
| C.III.1 3.12.5.6 | High-Frequency Modes | Conforms. Addressed in DCD Sections 3.7.1.1 and 3.7.1.2. |
| C.III.1 3.12.5.7 | Fatigue Evaluation of ASME Code Class 1 Piping | Conforms. Addressed in DCD Section 3.9.3.4 and DCD Table 3.9-8. |
| C.III.1 3.12.5.8 | Fatigue Evaluation of ASME Code Class 2 and 3 Piping | Conforms. Addressed in DCD Section 3.9. |
| C.III.1 3.12.5.9 | Thermal Oscillations in Piping Connected to the Reactor Coolant System | Conforms |
| C.III.1 3.12.5.10 | Thermal Stratification | Conforms. Addressed in DCD Section 3.9.2.1.2. |
| C.III.1 3.12.5.11 | Safety Relief Valve Design, Installation, and Testing | Conforms. Addressed in DCD Figures 5.2-3 and 5.4-3, and DCD Table 3.9-8. |
| C.III.1 3.12.5.12 | Functional Capability | Conforms. Addressed in DCD Table 3.9-2, Note 13, and DCD Chapters 5 and 6. |
| C.III.1 3.12.5.13 | Combination of Inertial and Seismic Anchor Motion Effects | Conforms. Addressed in DCD Section 3.7.3.9. |
| C.III.1 3.12.5.14 | Operating-Basis Earthquake as a Design Load | Not applicable. The SSE establishes the design load for the ESBWR. |
| C.III.1 3.12.5.15 | Welded Attachments | Conforms. Addressed in DCD Section 3.9.3.7.1. |

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| Section | Section Title | Conformance Evaluation |
|-------------------|---|---|
| C.III.1 3.12.5.16 | Modal Damping for Composite Structures | Conforms. Addressed in DCD Section 3.7.2.13. |
| C.III.1 3.12.5.17 | Minimum Temperature for Thermal Analyses | Conforms. Addressed in DCD Sections 3.9.1.1 and 3.9.3.1. |
| C.III.1 3.12.5.18 | Intersystem Loss-of-Coolant Accident | Conforms. Addressed in DCD Appendix 3K. |
| C.III.1 3.12.5.19 | Effects of Environment on Fatigue Design | Conforms. Addressed in DCD Section 3.9.3.4. The reference in RG 1.206 to 1.76 appears to be in error, and should have referenced 1.207. |
| C.III.1 3.12.6.1 | Applicable Codes | Conforms. Addressed in DCD Section 3.9.3.7.1. |
| C.III.1 3.12.6.2 | Jurisdictional Boundaries | Conforms. Addressed in DCD Section 3.9.3.7.1. |
| C.III.1 3.12.6.3 | Loads and Load Combinations | Conforms. Addressed in DCD Section 3.9 and DCD Appendix 3B. |
| C.III.1 3.12.6.4 | Pipe Support Baseplate and Anchor Bolt Design | Conforms. Addressed in DCD Section 3.9.3.7. |
| C.III.1 3.12.6.5 | Use of Energy Absorbers and Limit Stops | Conforms. Addressed in DCD Section 3.9.3.7. |
| C.III.1 3.12.6.6 | Use of Snubbers | Conforms. Addressed in DCD Section 3.9.3.7.1(3). |
| C.III.1 3.12.6.7 | Pipe Support Stiffnesses | Conforms. Addressed in DCD Section 3.7.3.3.1. |
| C.III.1 3.12.6.8 | Seismic Self-Weight Excitation | Conforms. Addressed in DCD Section 3.9.3.7.1. |
| C.III.1 3.12.6.9 | Design of Supplementary Steel | Conforms. Addressed in DCD Section 3.9.3.7.1. |
| C.III.1 3.12.6.10 | Consideration of Friction Forces | Conforms. Addressed in DCD Section 3.9.3.7.1(5). |
| C.III.1 3.12.6.11 | Pipe Support Gaps and Clearances | Conforms. Addressed in DCD Section 3.9.3.7.1. |
| C.III.1 3.12.6.12 | Instrumentation Line Support Criteria | Conforms. Addressed in DCD Section 3.9.3.7.1. |
| C.III.1 3.12.6.13 | Pipe Deflection Limits | Conforms. Addressed in DCD Section 3.9.2.1.1 and Chapter 14 . |
| C.III.1 3.13 | Threaded Fasteners – ASME code Class 1, 2, and 3 | Conforms |
| C.III.1 3.13.1.1 | Materials Selection | Conforms |
| C.III.1 3.13.1.2 | Special Materials Fabrication Processes and Special Controls | Conforms |
| C.III.1 3.13.1.3 | Fracture Toughness Requirements for Threaded Fasteners Made of Ferritic Materials | Conforms |

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| Section | Section Title | Conformance Evaluation |
|------------------|---|---|
| C.III.1 3.13.1.5 | Certified Material Test Reports | Conforms |
| C.III.1 3.13.2 | Inservice Inspection Requirements | Conforms |
| C.III.1 4.1 | Reactor: Summary Description | Conforms |
| C.III.1 4.2 | Fuel System Design | Conforms |
| C.III.1 4.3 | Nuclear Design | Conforms |
| C.III.1 4.4 | Thermal and Hydraulic Design | Conforms |
| C.III.1 4.5.1 | Control Rod Drive Structural Materials | Conforms |
| C.III.1 4.5.2 | Reactor Internal and Core Support Materials | Conforms |
| C.III.1 4.6 | Functional Design of Reactivity Control System | Conforms |
| C.III.1 5.1 | Reactor Coolant and Connecting Systems: Summary Description | Conforms |
| C.III.1 5.2.1 | Compliance with ASME Codes and Code Cases | Conforms |
| C.III.1 5.2.2.1 | Design Bases | Conforms |
| C.III.1 5.2.2.2 | Design Evaluation | Conforms |
| C.III.1 5.2.2.3 | Piping and Instrumentation Diagrams | Conforms |
| C.III.1 5.2.2.4 | Equipment and Component Description | Conforms |
| C.III.1 5.2.2.5 | Mounting of Pressure-Relief Devices | Conforms |
| C.III.1 5.2.2.6 | Applicable Codes and Classification | Conforms |
| C.III.1 5.2.2.7 | Material Specification | Conforms |
| C.III.1 5.2.2.8 | Process Instrumentation | Conforms |
| C.III.1 5.2.2.9 | System Reliability | Conforms |
| C.III.1 5.2.2.10 | Testing and Inspection | Conforms. Addressed in DCD Section 5.2.2.4, and in Section 3.9 and Chapter 14 . |
| C.III.1 5.2.3.1 | Material Specifications | Conforms |
| C.III.1 5.2.3.2 | Compatibility with Reactor Coolant | Conforms. Addressed in DCD Section 5.2.3. |
| C.III.1 5.2.3.3 | Fabrication and Processing of Ferritic Materials | Conforms |
| C.III.1 5.2.3.4 | Fabrication and Processing of Austenitic Stainless Steels | Conforms |

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(Sheet 14 of 39) [EF3 COL 1.9-3-A]

| Section | Section Title | Conformance Evaluation |
|-----------------|---|--|
| C.III.1 5.2.3.5 | Prevention of Primary Water Stress-Corrosion Cracking for Nickel-Based Alloys (PWRs only) | Not applicable. Applies only to PWRs. |
| C.III.1 5.2.3.6 | Threaded Fasteners | Conforms. Addressed in DCD Section 3.9.3.9. |
| C.III.1 5.2.4.1 | Inservice Inspection and Testing Program | Conforms. Addressed in DCD Section 5.2.4 and in Subsection 5.2.4. |
| C.III.1 5.2.4.2 | Preservice Inspection and Testing Program | Conforms. Addressed in DCD Section 5.2.4. |
| C.III.1 5.2.5 | Reactor Coolant Pressure Boundary Leakage Detection | Conforms |
| C.III.1 5.3.1.1 | Material Specifications | Conforms |
| C.III.1 5.3.1.2 | Special Processes Used for Manufacturing and Fabrication | Conforms |
| C.III.1 5.3.1.3 | Special Methods for Nondestructive Examination | Conforms |
| C.III.1 5.3.1.4 | Special Controls for Ferritic and Austenitic Stainless Steels | Conforms |
| C.III.1 5.3.1.5 | Fracture Toughness | Conforms |
| C.III.1 5.3.1.6 | Material Surveillance | Conforms. Addressed in DCD Section 5.3.1.6 and Subsection 5.3.1.8. |
| C.III.1 5.3.1.7 | Reactor Vessel Fasteners | RG does not contain any guidance in this section |
| C.III.1 5.3.2.1 | Limit Curves | Conforms |
| C.III.1 5.3.2.2 | Operating Procedures | Conforms. Addressed in DCD Sections 5.3.2.1, 5.3.2.2, and 5.3.3.6, and Subsection 5.3.3.6. |
| C.III.1 5.3.2.3 | Pressurized Thermal Shock (PWRs only) | Not applicable. Applies only to PWRs. |
| C.III.1 5.3.2.4 | Upper-Shelf Energy | Conforms |
| C.III.1 5.3.3 | Reactor Vessel Integrity | Conforms. Identification of a specific manufacturer is not required. |
| C.III.1 5.3.3.1 | Design | Conforms |
| C.III.1 5.3.3.2 | Materials of Construction | Conforms |
| C.III.1 5.3.3.3 | Fabrication Methods | Conforms |
| C.III.1 5.3.3.4 | Inspection Requirements | Conforms. Addressed in DCD Section 5.3.3.4. |
| C.III.1 5.3.3.5 | Shipment and Installation | Conforms. Addressed in DCD Section 5.3.3.5. |
| C.III.1 5.3.3.6 | Operating Conditions | Conforms. Addressed in DCD Section 5.3.3.6. |
| C.III.1 5.3.3.7 | Inservice Surveillance | Conforms. Addressed in DCD Section 5.3.3.7. |
| C.III.1 5.3.3.8 | Threaded Fasteners | Conforms. Addressed in DCD Section 3.9.3.9. |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
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| Section | Section Title | Conformance Evaluation |
|-----------------|--|---|
| C.III.1 5.4.1 | Reactor Coolant Pumps or Circulation Pumps (BWR) | Conforms |
| C.III.1 5.4.1.1 | Pump Flywheel Integrity (PWR) | Not applicable. Applies only to PWRs. |
| C.III.1 5.4.2 | Steam Generators (PWR) | Not applicable. Applies only to PWRs. |
| C.III.1.5.4.3 | Reactor Coolant System Piping and Valves | Conforms |
| C.III.1.5.4.4 | Main Steamline Flow Restrictions | Conforms |
| C.III.1.5.4.5 | Pressurizer | Not applicable. Applies only to PWRs. |
| C.III.1.5.4.6 | Reactor Core Isolation Cooling System (BWRs/Isolation Condenser System (Economic Simplified BWR)) | Conforms |
| C.III.1.5.4.7 | Residual Heat Removal System/Passive Residual Heat Removal System (Advanced Light-Water Reactor) Shutdown Cooling Mode of the Reactor Water Cleanup System (Economic Simplified BWR) | Conforms |
| C.III.1.5.4.8 | Reactor Water Cleanup System (BWR) Reactor Water Cleanup/Shutdown Cooling System (Economic Simplified BWR) | Conforms |
| C.III.1.5.4.9 | Reactor Coolant System Pressure Relief Devices/Reactor Coolant Depressurization Systems | Conforms |
| C.III.1.5.4.10 | Reactor Coolant System Component Supports | Conforms |
| C.III.1.5.4.11 | Pressurizer Relief Discharge System (PWRs only) | Not applicable. Applies only to PWRs. |
| C.III.1.5.4.12 | Reactor Coolant System High-Point Vents | Conforms |
| C.III.1.5.4.13 | Main Steamline, Feedwater, and Auxiliary Feedwater Piping | Conforms |
| C.III.1 6.1 | Engineered Safety Features: Engineered Safety Feature Materials | Conforms. Addressed in DCD Section 6.1. |
| C.III.1 6.1.1.1 | Materials Selection and Fabrication | Conforms |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
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| Section | Section Title | Conformance Evaluation |
|-----------------|---|---|
| C.III.1 6.1.1.2 | Composition and Compatibility of Core Cooling Coolants and Containment Sprays | Conforms. Addressed in DCD Sections 5.2.3.2, 5.4.8, 9.3.10, 5.2.3.4.1, 6.1.1.3.4, 9.1.3, 6.1.1.4, and 6.1.2. |
| C.III.1 6.1.2 | Organic Materials | Conforms |
| C.III.1 6.2 | Containment Systems | Conforms |
| C.III.1 6.2.1 | Containment Functional Design | Conforms |
| C.III.1 6.2.2 | Containment Heat Removal Systems | Conforms |
| C.III.1 6.2.3 | Secondary Containment Functional Design | Not Applicable. The ESBWR plant does not have a secondary containment. |
| C.III.1 6.2.4 | Containment Isolation System | Conforms. |
| C.III.1 6.2.5 | Combustible Gas Control in Containment | Conforms. |
| C.III.1 6.2.6 | Containment Leakage Testing | Conforms. Addressed in DCD Sections 6.2.6.1, 6.2.6.2, 6.2.6.3 and 6.2.6.4, and in Section 13.4 . Special testing requirements in RG 1.206, Section C.III.1 , Section 6.2.6.5 are not applicable to the ESBWR. |
| C.III.1 6.2.7 | Fracture Prevention of Containment Pressure Vessel | Conforms |
| C.III.1 6.3 | Emergency Core Cooling System | Conforms. There are no aspects of the site-specific design that affect the LOCA analyses in the DCD. |
| C.III.1 6.4 | Habitability Systems | Conforms |
| C.III.1 6.5 | Fission Product Removal and Control Systems | Conforms |
| C.III.1 6.6 | Inservice Inspection of Class 2 and 3 Components | Conforms. Addressed in DCD Section 6.6 and in Subsection 6.6.10.3 . |
| C.III.1 6.6.1 | Components Subject to Examination | Conforms |
| C.III.1 6.6.2 | Accessibility | Conforms |
| C.III.1 6.6.3 | Examination Techniques and Procedures | Conforms. Addressed in DCD Section 6.6.3.2. There are no special examination techniques required to meet the ASME Code. |
| C.III.1 6.6.4 | Inspection Intervals | Conforms. Addressed in DCD Section 6.6.4. |
| C.III.1 6.6.5 | Examination Categories and Requirements | Conforms. Addressed in DCD Section 6.6.3.1. |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
(Sheet 17 of 39) [EF3 COL 1.9-3-A]

| Section | Section Title | Conformance Evaluation |
|---------------|--|--|
| C.III.1 6.6.6 | Evaluation of Examination Results | Conforms (addressed in DCD Section 6.6.5), except that RG 1.206 references ASME Code Sections IWC-4000 and IWD-4000 for Class 2 and Class 3, respectively, whereas DCD Section 6.6.5 references IWA-4000. Later editions of ASME Code Section XI do not contain Sections IWC-4000 and IWD-4000, only IWA-4000. Therefore, the intent of the RG is met. |
| C.III.1 6.6.7 | System Pressure Tests | Conforms. Addressed in DCD Section 6.6.6. |
| C.III.1 6.6.8 | Augmented Inservice Inspection to Protect against Postulated Piping Failures | Conforms. Addressed in DCD Section 6.6.7. |
| C.III.1 6.7 | Main Steamline Isolation Valve Leakage Control Steam (BWRs) | Not applicable to the ESBWR. |
| C.III.1 7 | Instrumentation and Controls | Conforms. Addressed in DCD Chapter 7 and DCD Tier 1, and design-related ITAAC (DAC). There are no departures from the referenced certified design. |
| C.III.1 7.1 | Introduction | Conforms. There is no safety-related instrumentation, control, or supporting system that has not been addressed in the referenced certified design or other parts of the COL application. |
| C.III.1 7.2 | Reactor Trip System | Conforms. There is no reactor trip system instrumentation, control, or supporting system that has not been addressed in the referenced certified design or other parts of the COL application. |
| C.III.1 7.3 | Engineered Safety Features Systems | Conforms. There are no ESF systems I&C or supporting systems that have not been addressed in the referenced certified design or other parts of the COL application. |
| C.III.1 7.4 | Systems Required for safe Shutdown | Conforms. There are no safe-shutdown systems I&C or supporting systems that have not been addressed in the referenced certified design or other parts of the COL application. |
| C.III.1 7.5 | Information Systems Important to Safety | Conforms. There are no information systems important to safety that have not been addressed in the referenced certified design or other parts of the COL application. |
| C.III.1 7.6 | Interlock Systems Important to Safety | Conforms. There are no interlock systems important to safety that have not been addressed in the referenced certified design or other parts of the COL application. |

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| Section | Section Title | Conformance Evaluation |
|------------------|---|---|
| C.III.1 7.7 | Control Systems Not Required for Safety | Conforms. There is no control system instrumentation or supporting system that has not been addressed in the referenced certified design or other parts of the COL application. |
| C.III.1 7.8 | Diverse Instrumentation and Control Systems | Conforms. There is no diverse I&C system that has not been addressed in the referenced certified design or other parts of the COL application. |
| C.III.1 7.9 | Data Communication Systems | Conforms. There are no data communication systems that have not been addressed in the referenced certified design or other parts of the COL application. |
| C.III.1 8 | Electrical Power | Conforms |
| C.III.1 8.1 | Introduction | Conforms. There are no safety-related or RTNSS onsite AC or DC loads that are added to the referenced certified design. There are no safety-related or RTNSS electrical systems that are beyond the scope of the referenced certified design. |
| C.III.1 8.2.1 | Description | Conforms. Addressed in Section 8.2 . |
| C.III.1 8.2.2 | Analysis | Conforms. Addressed in Section 8.2 . |
| C.III.1 8.3.1.1 | AC Power Systems: Description | Conforms. Addressed in DCD Sections 8.3.1. |
| C.III.1 8.3.1.2 | Analysis | Not applicable. Does not request information for passive designs. |
| C.III.1 8.3.1.3 | Electrical Power System Calculations and Distribution System Studies for AC Systems | Conforms |
| C.III.1 8.3.2.1 | DC Power Systems: Description | Not applicable. Does not request information for passive designs. |
| C.III.1 8.3.2.2 | Analysis | Not applicable. Does not request information for passive designs. |
| C.III.1 8.3.2.3 | Electrical Power System Calculations and Distribution System Studies for DC Systems | Conforms |
| C.III.1 8.4.1(1) | Station Blackout: Description | Not applicable. Does not request information for passive designs. |
| C.III.1 8.4.1(2) | | Not applicable. Does not request information for passive designs. |
| C.III.1 8.4.1(3) | | Conforms. Addressed in Subsection 8.3.2.1.1 . |
| C.III.1 8.4.1(4) | | Conforms. Addressed in Subsection 8.3.2.1.1 . |

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| Section | Section Title | Conformance Evaluation |
|--|--|---|
| C.III.1 8.4.2 | Analysis | Not applicable. Does not request information for passive designs. |
| C.III.1 9.1.1 | Fuel Storage and Handling: Criticality Safety of Fresh and Spent Fuel Storage and Handling | Conforms. Addressed in DCD Sections 9.1.1 and 9.1.2. |
| C.III.1 9.1.2 | New and Spent Fuel Storage | Conforms. Addressed in DCD Section 9.1.2. |
| C.III.1 9.1.3 | Spent Fuel Pool Cooling and Cleanup System | Conforms. Addressed in DCD Section 9.1.3. |
| C.III.1 9.1.4 | Light Load Handling System (Related to Refueling) | Conforms |
| C.III.1 9.1.5 | Overhead Heavy Load Handling System | Conforms. Addressed in DCD Section 9.1.5.5 and in Subsection 9.1.4 and Subsection 9.1.5 . |
| C.III.1 9.2.1.1 | Station Service Water System (Open, Raw Water Cooling Systems): Design Bases | Conforms. Addressed in DCD Section 9.2.1.1. |
| C.III.1 9.2.1.2 | System Description | Conforms. Addressed in DCD Section 9.2.1.2 and in Subsection 9.2.1.2 . |
| C.III.1 9.2.1.3 | Safety Evaluation | Conforms. Addressed in DCD Section 9.2.1.3 and in Subsection 9.2.1.2 (for long-term corrosion and fouling). |
| C.III.1 9.2.1.4 | Inspection and Testing Requirements | Conforms. Addressed in DCD Section 9.2.1.4. |
| C.III.1 9.2.1.5 | Instrumentation Requirements | Conforms. Addressed in DCD Section 9.2.1.5. |
| C.III.1 9.2.2 | Cooling System for Reactor Auxiliaries (Closed Cooling Water Systems) | Conforms |
| C.III.1 9.2 (for DCD Section 9.2.3) | Makeup Water System Design Bases | Conforms. Design Bases, Safety Evaluation, Inspection and Testing Requirements, and Instrumentation are addressed in DCD Section 9.2.3. System Description is addressed in Subsection 9.2.3 . |
| C.III.1 9.2.4 | Potable and Sanitary Water Systems Design Basis | Conforms |
| C.III.1 9.2.5 | Ultimate Heat Sink | The design of the UHS is within the scope of the referenced certified design, and inspection and testing requirements are addressed in DCD Section 9.2.5. |

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(Sheet 20 of 39) [EF3 COL 1.9-3-A]

| Section | Section Title | Conformance Evaluation |
|---|---|---|
| C.III.1 9.2.6 | Condensate Storage Facilities | Conforms. There are no safety-related or RTNSS condensate storage facilities outside the scope of the referenced certified design that are sources of water for residual heat removal or sources of coolant inventory makeup for safety-related systems. |
| C.III.1 9.2 (for DCD Section 9.2.7) | Chilled Water System | Conforms. Addressed in DCD Section 9.2.7. |
| C.III.1 9.2 (for DCD Section 9.2.8) | Turbine Component Cooling Water System | Conforms. Addressed in DCD Section 9.2.8. |
| C.III.1 9.2 (for DCD Section 9.2.10) | Station Water System | Conforms. Design Bases, Safety Evaluation, Inspection and Testing Requirements, and Instrumentation are addressed in DCD Section 9.2.10. System Description is addressed in Subsection 9.2.10 . |
| C.III.1 9.3 | Process Auxiliaries | Conforms. Hydrogen Water Chemistry is addressed in Subsection 9.3.9 , Oxygen Injection System is addressed in Subsection 9.3.10 , Zinc Injection System is addressed in Subsection 9.3.11 , and Auxiliary Boiler System is addressed in DCD Section 9.3.12. |
| C.III.1 9.3.1 | Compressed Air Systems | Conforms. Instrument Air is addressed in DCD Section 9.3.6, Service Air is addressed in DCD Section 9.3.7, and High Pressure Nitrogen Supply System is addressed in DCD Section 9.3.8. |
| C.III.1 9.3.2 | Process and Postaccident Sampling Systems | Conforms |
| C.III.1 9.3.3 | Equipment and Floor Drain System | Conforms. Addressed in DCD Section 9.3.3. |
| C.III.1 9.3.4 | Chemical and Volume Control System (PWRs) (Including Boron Recovery System) | Not applicable. Applies only to PWRs. |
| C.III.1 9.3.5 | Standby Liquid Control System | Conforms |
| C.III.1 9.4 | Air Conditioning, Heating, Cooling, and Ventilation Systems | Conforms. Reactor Building HVAC System is addressed in DCD Section 9.4.6, Electric Building Heating, Ventilation, and Air Conditioning System is addressed in DCD Section 9.4.7, and Drywell Cooling System is addressed in DCD Section 9.4.8. |
| C.III.1 9.4.1 | Control Room Area Ventilation System | Conforms |

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| Section | Section Title | Conformance Evaluation |
|--------------------|---|---|
| C.III.1 9.4.2 | Spent Fuel Pool Area Ventilation Systems | Conforms |
| C.III.1 9.4.3 | Auxiliary and Radwaste Area Ventilation System | Conforms |
| C.III.1 9.4.4 | Turbine Building Area Ventilation System | Conforms |
| C.III.1 9.4.5 | Engineered Safety Feature Ventilation System | Conforms |
| C.III.1 9.5.1 | Fire Protection Program | Conforms |
| C.III.1 9.5.1.1(1) | | Conforms |
| C.III.1 9.5.1.1(2) | | Conforms |
| C.III.1 9.5.1.1(3) | | Conforms. Addressed in Section 1.7 . |
| C.III.1 9.5.1.1(4) | | Conforms. Will be completed in accordance with the milestones in Section 13.4 |
| C.III.1 9.5.1.1(5) | | Conforms. Will be completed in accordance with the milestones in Section 13.4 |
| C.III.1 9.5.1.1(6) | | Conforms |
| C.III.1 9.5.1.1(7) | | Conforms. Will be completed in accordance with the milestones in Section 13.4 |
| C.III.1 9.5.1.1(8) | | Conforms |
| C.III.1 9.5.1.1(9) | | Conforms. Addressed in DCD Sections 9.5.1.15 and 14.3, and in Section 13.4 |
| C.III.1 9.5.2 | Communication System | Conforms. Addressed in DCD Section 9.5.2 and in Subsection 9.5.2 . |
| C.III.1 9.5.3 | Lighting System | Conforms. Addressed in DCD Section 9.5.3. |
| C.III.1 9.5.4 | Diesel Generator Fuel Oil Storage and Transfer Systems | Conforms. Addressed in DCD Section 9.5.4 and in Subsection 9.5.4 . |
| C.III.1 9.5.4.1 | Design Basis | Conforms. Addressed in DCD Section 9.5.4. |
| C.III.1 9.5.4.2 | System Description | Conforms |
| C.III.1 9.5.4.3 | Safety Evaluation | Conforms |
| C.III.1 9.5.5 | Diesel Generator Cooling Water Systems | Conforms. Addressed in DCD Section 9.5.5. |
| C.III.1 9.5.6 | Diesel Generator Starting Systems | Conforms. Addressed in DCD Section 9.5.6. |
| C.III.1 9.5.7 | Diesel Generator Lubrication Systems | Conforms. Addressed in DCD Section 9.5.7. |
| C.III.1 9.5.8 | Diesel Generator Combustion Air Intake and Exhaust System | Conforms. Addressed in DCD Section 9.5.8. |

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| Section | Section Title | Conformance Evaluation |
|--------------------|--|--|
| C.III.1 10.1 | Steam and Power Conversion: Introduction | Conforms. There are no principal design features of the steam and power conversion system that are outside the scope of the referenced certified design. |
| C.III.1 10.2.1 (1) | Design Bases | Conforms. Addressed in DCD Section 10.2.1. |
| C.III.1 10.2.1 (2) | Design Bases | Conforms. Addressed in DCD Section 10.2.2. |
| C.III.1 10.2.1 (3) | Design Bases | Conforms. Addressed in DCD Sections 3.5.1, 3.5.3, 3.6, and 10.2.4, and DCD Figure 3.5-2. |
| C.III.1 10.2.2 (1) | Description | Conforms. Addressed in DCD Sections 10.2.2, 10.2.3, and DCD Figures 1.2-12 to 1.2-20, 3.5-2, and 10.1-1. |
| C.III.1 10.2.2 (2) | Description | Conforms. Addressed in DCD Sections 10.2.2 and 10.2.3. |
| C.III.1 10.2.2 (3) | Description | Conforms. Addressed in DCD Section 10.2.2 and DCD Figures 10.2-1, 10.2-2, and 10.2-3. |
| C.III.1 10.2.2 (4) | Description | Conforms. Addressed in DCD Sections 10.2.3 and 14.2.8. |
| C.III.1 10.2.2 (5) | Description | Conforms. Addressed in DCD Sections 12.2.1, 12.2.3, 12.4.4, Table 12.2-23a and DCD Figures 12.3-12 to 12.3-18 and 12.3-32 to 12.3-38. |
| C.III.1 10.2.2 (6) | Description | Conforms. Addressed in DCD Sections 3.6, 10.2.2, and 10.2.4. |
| C.III.1 10.2.3 (1) | Turbine Rotor Integrity | Conforms. Addressed in DCD Section 10.2.3 and Subsection 10.2.3.7. |
| C.III.1 10.2.3 (2) | Turbine Rotor Integrity | Conforms. Addressed in DCD Section 10.2.3 and Subsection 10.2.3.7. |
| C.III.1 10.2.3 (3) | Turbine Rotor Integrity | Conforms. Addressed in DCD Section 10.2.3 and Subsection 10.2.3.7. |
| C.III.1 10.2.3 (4) | Turbine Rotor Integrity | Conforms. Addressed in DCD Section 10.2.3 and Subsection 10.2.3.7. |
| C.III.1 10.2.3 (5) | Turbine Rotor Integrity | Conforms. Addressed in DCD Sections 10.2.2 and 10.2.3, and Subsection 10.2.3.8. |
| C.III.1 10.3 | Main Steam Supply System | Conforms. Addressed in DCD Section 10.3. |
| C.III.1 10.3.1 (1) | Design Bases | Conforms. Addressed in DCD Section 10.3.1. |
| C.III.1 10.3.1 (2) | Design Bases | Conforms. Addressed in DCD Section 10.3. |
| C.III.1 10.3.1 (3) | Design Bases | Conforms. Addressed in DCD Sections 10.3.2 and 10.3.3. |
| C.III.1 10.3.1 (4) | Design Bases | Conforms. Addressed in DCD Section 10.3. |
| C.III.1 10.3.1 (5) | Design Bases | Conforms. Addressed in DCD Section 10.3. |

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| Section | Section Title | Conformance Evaluation |
|--------------------|---|--|
| C.III.1 10.3.1 (6) | Design Bases | Conforms. Addressed in DCD Section 10.3. |
| C.III.1 10.3.2 | Description | Conforms. Addressed in DCD Section 10.3. |
| C.III.1 10.3.3 | Evaluation | Conforms. Addressed in DCD Section 10.3. |
| C.III.1 10.3.4 | Inspection and Testing Requirements | Conforms. Addressed in DCD Section 10.3.4. |
| C.III.1 10.3.5 | Water Chemistry (PWR Only) | Not applicable. Only applies to PWRs. |
| C.III.1 10.3.6 (1) | Steam and Feedwater System Materials | Conforms. Addressed in DCD Section 10.3.6. |
| C.III.1 10.3.6 (2) | Steam and Feedwater System Materials | Conforms. Addressed in DCD Sections 6.6 and 10.3.4. |
| C.III.1 10.3.6 (3) | Steam and Feedwater System Materials | Not applicable. DCD Section 10.3.6 states that there are no austenitic stainless steels in the steam and feedwater system piping. |
| C.III.1 10.3.6 (4) | Steam and Feedwater System Materials | Not Applicable. DCD Section 10.3.6 states that there are no austenitic stainless steels in the ASME Code Section III Class 1 and 2 portions of steam and feedwater piping. |
| C.III.1 10.3.6 (5) | Steam and Feedwater System Materials | Conforms. Addressed in DCD Section 10.3. |
| C.III.1 10.3.6 (6) | Steam and Feedwater System Materials | Not applicable |
| C.III.1 10.4 (1) | Other Features of the Steam and Power Conversion System | Conforms |
| C.III.1 10.4.1 | Main Condensers | Conforms. Sampling points for detection are discussed in DCD Section 10.4.1.5.4. Although sodium content and sampling for sodium content is not specifically mentioned in DCD Section 10.4.1, monitoring condensate for an increase in conductivity is considered an acceptable means to detect condenser tube leakage. A table of key parameters and associated action levels is provided as Table 10.4-201 . Alarm setpoints are established to provide an indication of abnormal chemistry conditions prior to reaching a recommended action level. |
| C.III.1 10.4.2 | Main Condenser Evacuation System | Conforms. There are no design features of the main condenser evacuation system that are outside the scope of the referenced certified design. |
| C.III.1 10.4.3 (1) | Turbine Gland Sealing System | Conforms. Addressed in DCD Section 10.4.3. |

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| Section | Section Title | Conformance Evaluation |
|--------------------|---|---|
| C.III.1 10.4.3 (2) | | Conforms with the following exception: For the operational phase, the QA Program is described in Chapter 17 , and is based on NQA-1, rather than RG 1.33. |
| C.III.1 10.4.4 (1) | Turbine Bypass System | Conforms. The Turbine Bypass System is consistent with the referenced certified design. |
| C.III.1 10.4.5 (1) | Circulating Water System | Conforms |
| C.III.1 10.4.5 (2) | | Not applicable. The circulating water system does not interface with the UHS. |
| C.III.1 10.4.6 (1) | Condensate Cleanup System | Conforms |
| C.III.1 10.4.6 (2) | | Conforms. Addressed in DCD Sections 10.4.1, 10.4.6, and 5.2.3, DCD Table 5.2-5, and in Table 10.4-201 . |
| C.III.1 10.4.6 (3) | | Conforms |
| C.III.1 10.4.6 (4) | | Not applicable. Only applies to PWRs. |
| C.III.1 10.4.7 (1) | Condensate and Feedwater Systems | Not applicable. Only applies to PWRs. |
| C.III.1 10.4.7 (2) | | Conforms. Addressed in DCD Sections 1.2.2 and 5.2.4, and DCD Tables 1.9-22 and 1.11-1. |
| C.III.1 10.4.7 (3) | | Not applicable. The condensate and feedwater systems are consistent with the referenced certified design. |
| C.III.1 10.4.8 | Steam Generator Blowdown System (PWR) | Not applicable. Only applies to PWRs. |
| C.III.1 10.4.9 | Auxiliary Feedwater System (PWR) | Not applicable. Only applies to PWRs. |
| C.III.1 11.1 | Source Terms | Conforms |
| C.III.1 11.2.1(1) | Liquid Waste Management Systems: Design Bases | Conforms. Addressed in DCD Section 11.2 and in Section 11.2 . |
| C.III.1 11.2.1(2) | Design Bases | Conforms. Addressed in DCD Section 11.2. |
| C.III.1 11.2.1(3) | Design Bases | Conforms. Addressed in DCD Section 11.2.1 and DCD Table 11.2-3. Conformance with RG 1.140 is addressed in DCD Section 9.4.3. |
| C.III.1 11.2.1(4) | Design Bases | Conforms. Addressed in DCD Section 9.4.3. |
| C.III.1 11.2.1(5) | Design Bases | Conforms. Addressed in DCD Sections 11.2.3 and 15.3.16 and in Subsection 2.4.13 . |
| C.III.1 11.2.1(6) | Design Bases | Conforms. Quality Assurance Program requirements are addressed in Chapter 17 . |
| C.III.1 11.2.1(7) | Design Bases | Conforms. Addressed in DCD Section 11.2.4. |

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| Section | Section Title | Conformance Evaluation |
|-------------------|--|---|
| C.III.1 11.2.1(8) | Design Bases | Conforms |
| C.III.1 11.2.1(9) | Design Bases | Conforms. Addressed in DCD Section 11.2.2 and in Section 11.2 . |
| C.III.1 11.2.2(1) | System Description | Conforms. Addressed in DCD Section 11.2.2. |
| C.III.1 11.2.2(2) | System Description | Conforms. Addressed in DCD Section 11.2.2. |
| C.III.1 11.2.2(3) | System Description | Conforms. Addressed in DCD Section 11.2.2. |
| C.III.1 11.2.2(4) | System Description | Conforms. Addressed in DCD Section 11.2.2. |
| C.III.1 11.2.3(1) | Radioactive Effluent Releases | Conforms. Addressed in DCD Sections 11.2 and 12.2, and in Section 12.2 . |
| C.III.1 11.2.3(2) | Radioactive Effluent Releases | Conforms. Addressed in DCD Sections 11.2 and 12.2, and in Section 12.2 . |
| C.III.1 11.3.1(1) | Gaseous Waste Management Systems: Design Bases | Addressed in DCD Section 11.3. Conforms with the following exception: No discussion is provided regarding the capability of and requirements for using portable processing equipment for refueling outages. |
| C.III.1 11.3.1(2) | Design Bases | Conforms. Addressed in DCD Section 11.3. |
| C.III.1 11.3.1(3) | Design Bases | Conforms. Addressed in DCD Section 11.3. |
| C.III.1 11.3.1(4) | Design Bases | Conforms. Quality Assurance Program requirements are addressed in Chapter 17 . |
| C.III.1 11.3.1(5) | Design Bases | Conforms. Addressed in DCD Section 11.3.5. |
| C.III.1 11.3.1(6) | Design Bases | Conforms. Addressed in DCD Section 12.3.1.5 |
| C.III.1 11.3.1(7) | Design Bases | Conforms. Addressed in DCD Section 11.3. |
| C.III.1 11.3.2(1) | System Description | Conforms. Addressed in DCD Section 11.3.2. |
| C.III.1 11.3.2(2) | System Description | Conforms. Addressed in DCD Section 11.3.2. |
| C.III.1 11.3.2(3) | System Description | Conforms. Addressed in DCD Section 11.3.2. |
| C.III.1 11.3.2(4) | System Description | Conforms. Addressed in DCD Sections 11.3.2, 11.3.3, and 9.4. |
| C.III.1 11.3.3 | Radioactive Effluent Releases | Conforms. Addressed in DCD Section 11.3 and 12.2, and in Section 12.2 . |
| C.III.1 11.4.1(1) | Solid Waste Management System: Design Bases | Conforms. Addressed in DCD Section 11.4 and in Section 11.4 . |
| C.III.1 11.4.1(2) | Design Bases | Conforms. Addressed in DCD Section 11.4 and in Section 11.4 . |
| C.III.1 11.4.1(3) | Design Bases | Conforms. Addressed in DCD Section 11.4 and in Section 11.4 . |

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| Section | Section Title | Conformance Evaluation |
|--------------------|---|---|
| C.III.1 11.4.1(4) | Design Bases | Conforms. Addressed in DCD Section 11.4 and in Sections 11.4, 13.5, and 17.5 . |
| C.III.1 11.4.1(5) | Design Bases | Conforms. Addressed in DCD Section 11.4 and in Section 11.4 . |
| C.III.1 11.4.1(6) | Design Bases | Conforms. |
| C.III.1 11.4.1(7) | Design Bases | Conforms. Addressed in DCD Section 11.4. |
| C.III.1 11.4.2(1) | System Description | Addressed in DCD Section 11.4 and in Section 11.4 . Conforms with the following exception: The FSAR provides a description of the PCP. Detailed waste packaging methodologies will be provided in the PCP. The implementation milestone is provided in Section 13.4 |
| C.III.1 11.4.2(2) | System Description | Addressed in DCD Section 11.4 and in Section 11.4 . Conforms with the following exception: The FSAR provides a description of the PCP. Detailed waste packaging methodologies will be provided in the PCP. The implementation milestone is provided in Section 13.4 |
| C.III.1 11.4.2(3) | System Description | Addressed in DCD Section 11.4 and in Section 11.4 . Conforms with the following exception: The FSAR provides a description of the PCP. Detailed waste packaging methodologies will be provided in the PCP. The implementation milestone is provided in Section 13.4 There are no temporary onsite storage facilities. |
| C.III.1 11.4.2 (4) | System Description | Conforms. Addressed in DCD Section 11.4. |
| C.III.1 11.4.3 (1) | Radioactive Effluent Releases | Addressed in DCD Section 11.4 and in Section 11.4 . Conforms with the following exception: The FSAR provides a description of the PCP. Detailed waste packaging methodologies will be provided in the PCP. The implementation milestone is provided in Section 13.4 . |
| C.III.1 11.4.3 (2) | Radioactive Effluent Releases | Conforms. Addressed in DCD Sections 3.1 and 11.4. |
| C.III.1 11.4.3 (3) | Radioactive Effluent Releases | Conforms. Addressed in DCD Section 12.2. |
| C.III.1 11.5.1 | Process and Effluent Radiological Monitoring and Sampling Systems: Design Bases | Conforms |
| C.III.1 11.5.2(1) | System Description | Conforms. Addressed in DCD Section 11.5. |

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| Section | Section Title | Conformance Evaluation |
|----------------------|--|---|
| C.III.1 11.5.2 (2) | System Description | Conforms with the following exception: Section 11.5 provides a description of the ODCM. The implementation milestone is provided in Section 13.4 |
| C.III.1 11.5.2 (3) | System Description | Conforms with the following exception: Section 11.5 and TS Section 5 provide a description of the radiological effluent controls. The implementation milestone is provided in Section 13.4 |
| C.III.1 11.5.2 (4) | System Description | Conforms with the following exception: Section 11.5 and TS Section 5 provide a description of the REMF. The implementation milestone is provided in Section 13.4 |
| C.III.1 11.5.2 (5) | System Description | Conforms. Addressed in DCD Sections 3.1 and 11.5. |
| C.III.1 11.5.2 (6) | System Description | Conforms |
| C.III.1 11.5.2 (7) | System Description | Conforms |
| C.III.1 11.5.3 | Effluent Monitoring and Sampling | Conforms |
| C.III.1 11.5.4 | Process Monitoring and Sampling | Conforms |
| C.III.1 12.1.1 | Policy Considerations | Conforms. Addressed in Section 12.1 and Section 12.5 . |
| C.III.1 12.1.2 | Design Considerations | Conforms. Addressed in Section 12.5 . |
| C.III.1 12.1.3 | Operational Considerations | Conforms. Addressed in Section 12.1 and Section 12.5 . |
| C.III.1 12.2.1 | Contained Sources | Conforms. Addressed in DCD Section 12.2.1. |
| C.III.1 12.2.2 | Airborne Radioactive Material Sources | Conforms |
| C.III.1 12.3.1 | Facility Design Features | Conforms |
| C.III.1 12.3.2 | Shielding | Conforms |
| C.III.1 12.3.3 | Ventilation | Conforms. Addressed in DCD Sections 9.4.1 and 12.3. |
| C.III.1 12.3.4 | Area Radiation and Airborne Radioactivity Monitoring Instrumentation | Conforms. Addressed in Sections 12.3 and 12.5 . |
| C.III.1 12.3.5 | Dose Assessment | Conforms. Addressed in DCD Section 12.4 and in Section 12.4 . |
| C.III.1 12.4 | Dose Assessment | Conforms |
| C.III.1 12.5 (1) (a) | Operational Radiation Protection Program: Organization | Conforms. Addressed in Section 12.5 and Section 13.1 |

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| Section | Section Title | Conformance Evaluation |
|------------------------------|--|---|
| C.III.1 12.5 (1) (b) | Facilities | Conforms |
| C.III.1 12.5 (1) (c) | Instrumentation and Equipment | Conforms |
| C.III.1 12.5 (1) (d) | Procedures | Conforms |
| C.III.1 12.5 (1) (e) | Training | Conforms. Addressed in Section 12.5 and Section 13.2 |
| C.III.1 12.5 (2) | | Conforms. Addressed in DCD Section 12.3. |
| C.III.1 12.5 (3) | | Conforms. Addressed in Section 12.5 , Section 13.1 , and Section 13.4 |
| C.III.1 12.5 (4) | | Conforms. Addressed in Section 13.4 |
| C.III.1 12.5, last paragraph | | Conforms. Addressed in Section 12.5 , Section 13.1 , Section 13.2 , and Section 13.5 . |
| C.III.1 12.5.1 | Organization | Conforms. Addressed in Section 12.5 and Section 13.1 |
| C.III.1 12.5.2 | Equipment, Instrumentation, and Facilities | Conforms |
| C.III.1 12.5.3 | Procedures | Addressed in Section 12.5 , Section 13.2 , Section 13.5 , and Section 17.5 . Conforms with one exception: With respect to RG 1.33, Detroit Edison's QA procedures follow NQA-1 rather than the older standards referenced in RG 1.33. The QA requirements are described in Section 17.5 . |
| C.III.1 13.1.1(1) | Organizational Structure of Applicant: Management and Technical Support Organization | Conforms. Addressed in Section 13.1 and Section 14.2 . |
| C.III.1 13.1.1(2) | | Conforms |
| C.III.1 13.1.1(3) | | Conforms |
| C.III.1 13.1.1(4) | | Conforms |
| C.III.1 13.1.1(5) | | Conforms |
| C.III.1 13.1.1(6) | | Conforms |
| C.III.1 13.1.1(7) | | Conforms. Addressed in Section 13.1 and Section 14.2 . |
| C.III.1 13.1.1.1 | Design, Construction, and Operating Responsibilities | Conforms |
| C.III.1 13.1.1.2 | Organizational Arrangement | Conforms. Addressed in Section 13.1 and Section 17.5 Fermi 3 is not a new, multi-unit plant site. |
| C.III.1 13.1.1.3 | Qualifications | Conforms. Addressed in Section 13.1 and Section 17.5 . |

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| Section | Section Title | Conformance Evaluation |
|--|--|--|
| C.III.1 13.1.2(1) | | Exception. The guidelines of RG 1.33 are met through equivalent administrative controls described in Chapter 17 . |
| C.III.1 13.1.2(2) | | Exception. The guidelines of RG 1.33 are met through equivalent administrative controls described in Chapter 17 . |
| C.III.1 13.1.2(3) | | Conforms. Addressed in Subsection 9.5.1 and Section 13.1 . |
| C.III.1 13.1.2(4) | | Conforms |
| C.III.1 13.1.2(5) | | Conforms |
| C.III.1 13.1.2(6) | | Conforms |
| C.III.1 13.1.2(7) | | Conforms |
| C.III.1 13.1.2(8) | | Conforms. Addressed in Appendix 13AA . |
| C.III.1 13.1.2.1 | Plant Organization | Conforms. Addressed in Section 13.1 and Section 17.5 . |
| C.III.1 13.1.2.2(1) | Plant Personnel Responsibilities and Authorities | Conforms. Addressed in Section 13.1 and Section 17.5 . |
| C.III.1 13.1.2.2(2) | | Conforms |
| C.III.1 13.1.2.2(3) | | Conforms |
| C.III.1 13.1.2.3 | Operating Shift Crews | Conforms |
| C.III.1 13.1.3.1 | Qualification Requirements | Conforms. Addressed in Section 13.1 and Section 17.5 . |
| C.III.1 13.1.3.2 | Qualifications of Plant Personnel | Exception. Resumes will not be included in the application, but will be available for inspection upon request. |
| C.III.1 13.2.1 | Plant Staff Training Program | Conforms |
| C.III.1 13.2.1.1 Licensed Staff (1) | | Conforms with the following exceptions: 1) this item discusses inclusion of details of the licensed training program. As noted in Appendix 13BB , the systematic approach to training (SAT) process is used to establish and maintain training programs. Course duration and content are determined by the SAT process and by administrative procedure and are not included in the FSAR section; 2) the requirement for a "contingency plan...in the event fuel loading is subsequently delayed" is met by the operator re-qualification program; and 3) the industry standard content for this section does not include a discussion of proposed schedule for licensed personnel. |

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| Section | Section Title | Conformance Evaluation |
|---|---------------|---|
| C.III.1 13.2.1.1 Licensed Staff (2) | | Conforms |
| C.III.1 13.2.1.1 Licensed Staff (3) | | Conforms |
| C.III.1 13.2.1.1 Licensed Staff (4) | | Conforms |
| C.III.1 13.2.1.1 Licensed Staff (5) | | Conforms |
| C.III.1 13.2.1.1 Licensed Staff (6) | | Conforms |
| C.III.1 13.2.1.1 Non-licensed Staff (1) | | Conforms |
| C.III.1 13.2.1.1 Non-licensed Staff (2) | | Conforms |
| C.III.1 13.2.1.1 Non-licensed Staff (3) | | Exception – This item discusses programs not covered under 10 CFR 50.120. As noted in Appendix 13BB , the systematic approach to training (SAT) process is used to establish and maintain training programs. Course duration and content are determined by the SAT process and by administrative procedure and are not included in the FSAR section. |
| C.III.1 13.2.1.1 Non-licensed Staff (4) | | Conforms. Addressed in Subsection 9.5.1 . |
| C.III.1 13.2.1.1 Non-licensed Staff (5) | | Conforms |
| C.III.1 13.2.1.1 Non-licensed Staff (6) | | Conforms with the following exception: The first part of this item discusses detailed course descriptions. As noted in Appendix 13BB , the systematic approach to training (SAT) process is used to establish and maintain training programs. Course duration and content are determined by the SAT process and by administrative procedure and are not included in the FSAR section. The implementation milestone is addressed in Section 13.4 . |

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| Section | Section Title | Conformance Evaluation |
|---|--|---|
| C.III.1 13.2.1.1 Non-licensed Staff (7) | | Conforms |
| C.III.1 13.2.1.2 | Coordination with Preoperational Tests and Fuel Loading | Conforms with the following exception - Rather than providing contingency plans for training in the event of significantly delayed fuel loading the retraining programs are utilized, as described in Appendix 13BB . Figure 13.1-202 shows the training schedule relative to fuel loading. |
| C.III.1 13.2.2(1) | Applicable NRC Documents: 10 CFR 19 | Conforms |
| C.III.1 13.2.2(2) | 10 CFR 26 | Conforms |
| C.III.1 13.2.2(3) | 10 CFR 50 | Conforms |
| C.III.1 13.2.2(4) | 10 CFR 50 Appendix E | Conforms |
| C.III.1 13.2.2(5) | 10 CFR 52 | Conforms |
| C.III.1 13.2.2(6) | 10 CFR 55 | Conforms |
| C.III.1 13.2.2(7) | RG 1.8 | Addressed in Table 1.9-202 . |
| C.III.1 13.2.2(8) | RG 1.149 | Addressed in Table 1.9-202 . |
| C.III.1 13.2.2(9) | NUREG-0711 | Conforms. HFE addressed in DCD Chapter 18. |
| C.III.1 13.2.2(10) | NUREG-1021 | Exception: Industry standard content for this section does not explicitly include discussion of compliance with NUREG-1021, Operator Licensing Examination Standards for Power Reactors. |
| C.III.1 13.2.2(11) | NUREG-1220 | Not applicable. NUREG provides instructions for NRC inspectors. |
| C.III.1 13.2.2(12) | GL 86-04 | Conforms |
| C.III.1 13.2.2(13) | RG 1.134 | Conforms. Industry standard content for this section does not explicitly include a discussion of compliance with RG 1.134, Medical Evaluations. |
| C.III.1 13.3(1) | Emergency Planning | Conforms. Addressed in the Emergency Plan in COLA Part 5. |
| C.III.1 13.3(2) | | Conforms. Addressed in the Emergency Plan in COLA Part 5. |
| C.III.1 13.3(3) | | Conforms. Addressed in the Emergency Plan in COLA Part 5. |

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| Section | Section Title | Conformance Evaluation |
|--------------------|---|--|
| C.III.1 13.3(4) | | Conforms. Addressed in Chapter 2 , and the Emergency Plan and Evacuation Time Estimate in COLA Part 5. |
| C.III.1 13.3(5) | | Conforms. Addressed in COLA Part 5. |
| C.III.1 13.3(6) | | Not applicable. Applies when state and/or local governments decline to participate in emergency planning and preparedness. |
| C.III.1 13.3(7) | | Conforms |
| C.III.1 13.3.1 (1) | Combined License Application and Emergency Plan Content | Conforms. Addressed in COLA Part 5. |
| C.III.1 13.3.1 (2) | | Conforms. Addressed in COLA Part 5 and COLA Part 10. |
| C.III.1 13.3.1 (3) | | Conforms. Addressed in Chapter 1 and the Emergency Plan in COLA Part 5. |
| C.III.1 13.3.1 (4) | | Conforms. Addressed in the Emergency Plan in COLA Part 5. |
| C.III.1 13.3.1 (5) | | Conforms. Addressed in the Emergency Plan in COLA Part 5. |
| C.III.1 13.3.1 (6) | | Conforms. Addressed in the Emergency Plan in COLA Part 5. |
| C.III.1 13.3.1 (7) | | Conforms. Addressed in Chapter 1 . |
| C.III.1 13.3.1 (8) | | Conforms. Addressed in the Emergency Plan in COLA Part 5. |
| C.III.1 13.3.1 (9) | | Conforms. Addressed in the Emergency Plan in COLA Part 5. |
| C.III.1 13.3.2 (1) | Emergency Plan Considerations for Multiunit Sites | Conforms. The Fermi 3 EP is a stand-alone plan and does not rely upon the EP for Fermi 2. |
| C.III.1 13.3.2 (2) | | Not applicable. The Fermi 3 EP is a stand-alone plan and does not rely upon the EP for Fermi 2. |
| C.III.1 13.3.2 (3) | | Conforms. Addressed in the Emergency Plan in COLA Part 5 and COLA Part 10. |
| C.III.1 13.3.2 (4) | | Conforms. Addressed in COLA Part 5. |
| C.III.1 13.3.2 (5) | | Conforms. Addressed in the Emergency Plan in COLA Part 5. |
| C.III.1 13.3.2 (6) | | Conforms. Addressed in the Emergency Plan and the Evacuation Time Estimate in COLA Part 5. |
| C.III.1 13.3.2 (7) | | Not applicable. Provisions for co-located licensees do not apply. |
| C.III.1 13.3.2 (8) | | Conforms. Addressed in COLA Part 10. |

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| Section | Section Title | Conformance Evaluation |
|--------------------|---|--|
| C.III.1 13.3.2 (9) | | Not applicable. There are no adjacent sites. |
| C.III.1 13.3.3 | Emergency Planning Inspections, Tests, Analyses, and Acceptance Criteria | Conforms. Addressed in COLA Part 10. |
| C.III.1 13.4 | Operational Program Implementation | Conforms |
| C.III.1 13.5.1 | Administrative Procedures | Conforms. Addressed in Sections 13.5 and Section 17.5 |
| C.III.1 13.5.2.1 | Operating and Emergency Operating Procedures | Conforms with the following exception: Subsection 13.5.1 identifies classes of procedures by topic or type in lieu of the specific title. Operating procedures will be developed after activities such as job and task analyses have been completed. |
| C.III.1 13.5.2.2 | Maintenance and Other Operating Procedures | Conforms |
| C.III.1 13.6 | Security | Conforms. Addressed in Sections 13.4 and Section 13.6 , and . |
| C.III.1 13.7 | FFD | Conforms |
| C.III.1 14.1 | Verification Program: Specific Information to be Addressed for the Initial Plant Test Program | Conforms. Addressed in Section 14.2 and Section 14.3 . |
| C.III.1 14.2 | Initial Plant Test Program | Conforms |
| C.III.1 14.2.1 | Summary of Test Program and Objectives | Conforms |
| C.III.1 14.2.2 | Organization and Staffing | Conforms. Addressed in DCD Section 14.2 and in Section 13.1 , Section 14.2 and Section 17.5 |
| C.III.1 14.2.3 | Test Procedures | Conforms. Addressed in DCD Section 14.2. |
| C.III.1 14.2.4 | Conduct of Test Program | Conforms. Addressed in DCD Section 14.2. |
| C.III.1 14.2.5 | Review, Evaluation, and Approval of Test Results | Conforms. Addressed in DCD Section 14.2. |
| C.III.1 14.2.6 | Test Records | Conforms |
| C.III.1 14.2.7 | Conformance of Tests Programs with Regulatory Guides | Conforms. Addressed in DCD Section 14.2.3. |
| C.III.1 14.2.8 | Utilization of Reactor Operating and Testing Experiences in Development of Test Program | Conforms. Addressed in DCD Section 14.2 and in Section 14.2 . |
| C.III.1 14.2.9 | Trial Use of Plant Operating and Emergency Procedures | Conforms. Addressed in DCD Section 14.2.5 and in Section 13.2 . |

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| Section | Section Title | Conformance Evaluation |
|-----------------|---|---|
| C.III.1 14.2.10 | Initial Fuel Loading and Initial Criticality | Conforms. Addressed in DCD Section 14.2.6. |
| C.III.1 14.2.11 | Test Program Schedule | Conforms. Addressed in DCD Section 14.2.7 and in Subsection 14.2.7 . |
| C.III.1 14.2.12 | Individual Test Descriptions | Conforms. Addressed in DCD Section 14.2.8 and in Subsection 14.2.9 . |
| C.III.1 14.3 | Inspections, Tests, Analyses, and Acceptance Criteria | Conforms. Addressed in COLA Part 10. |
| C.III.1 15.1 | Transient and Accident Analyses: Transient and Accident Classification | Conforms. There are no aspects of the site-specific design that affect the transient and accident analyses in the DCD. |
| C.III.1 15.2 | Frequency of Occurrence | Conforms |
| C.III.1 15.3 | Plant Characteristics Considered in the Safety Evaluation | Conforms |
| C.III.1 15.4 | Assumed Protection System Actions | Conforms |
| C.III.1 15.5 | Evaluation of Individual Initiating Events | Conforms. |
| C.III.1 15.6 | Event Evaluation | See below |
| C.III.1 15.6.1 | Identification of Causes and Frequency Classification | Conforms |
| C.III.1 15.6.2 | Sequence of Events and Systems Operation | Conforms |
| C.III.1 15.6.3 | Core and System Performance | Conforms |
| C.III.1 15.6.4 | Barrier Performance | Conforms |
| C.III.1 15.6.5 | Radiological Consequences | Conforms. Table 2.0-201 compares the site-specific short-term X/Qs for the EAB, LPZ, and control room to the X/Qs assumed in the DCD. |
| C.III.1 16.1 | Technical Specifications and Bases | Conforms. Addressed in COLA Part 4. There are no deviations from the generic TS bases. |
| C.III.1 16.2 | Content and Format of Technical Specifications and Bases | Conforms. Addressed in COLA Part 4. No plant-specific deviations from the referenced certified generic Technical Specifications or Bases are required and none are being requested (e.g., incorporation of TSTF travelers). |
| C.III.1 17.1 | Quality Assurance and Reliability Assurance: Quality Assurance During the Design and Construction Phase | Conforms |

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| Section | Section Title | Conformance Evaluation |
|----------------|---|--|
| C.III.1 17.2 | Quality Assurance During the Operations Phase | Conforms |
| C.III.1 17.3 | Quality Assurance Program Description | Conforms |
| C.III.1 17.4.1 | New Section 17.4 in the Standard Review Plan | Conforms |
| C.III.1 17.4.2 | Reliability Assurance Program Scope, Stages, and Goals | Not applicable |
| C.III.1 17.4.3 | Reliability Assurance Program Implementation | Conforms. Addressed in Section 17.4 and Section 17.6 . |
| C.III.1 17.4.4 | Reliability Assurance Program Information Needed in a COL Application | Conforms. Addressed in DCD Section 17.4 and in Section 17.4 , Section 17.5 , and Section 17.6 |
| C.III.1 17.5 | Quality Assurance Program Guidance | See below |
| C.III.1 17.5.1 | COL Applicant QA Program Responsibilities | Conforms |
| C.III.1 17.5.2 | Updated SRP Section 17.5 and the QA Program Description | Criterion I requires “retaining responsibility for the quality assurance program” while C.III.1.17.5.2 goes further to require “.. and maintain control over, those portions of the QA program delegated to other organizations.” Detroit Edison retained responsibility for the QA program (see FSAR Subsection 1.4.1), this responsibility was accomplished by fully delegating the work of establishing and executing the QA program under their QA program satisfying the requirements of Appendix B to 10 CFR Part 50. |
| C.III.1 17.5.3 | Evaluation of the QAPD Against the SRP and QAPD Submittal Guidance | Conforms |
| C.III.1 17.6 | Description of the Applicant’s Program for Implementation of 10 CFR 50.65, the Maintenance Rule | Conforms |
| C.III.1 17.6.1 | Scoping per 10 CFR 50.65(b) | Conforms |
| C.III.1 17.6.2 | Monitoring per 10 CFR 50.65(a) | Conforms |
| C.III.1 17.6.3 | Periodic Evaluation per 10 CFR 50.65(a)(3) | Conforms |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
(Sheet 36 of 39) [EF3 COL 1.9-3-A]

| Section | Section Title | Conformance Evaluation |
|--------------------|---|--|
| C.III.1 17.6.4 | Risk Assessment and Management per 10 CFR 50.65(a)(4) | Conforms |
| C.III.1 17.6.5 | Maintenance Rule Training and Qualification | Conforms |
| C.III.1 17.6.6 | Maintenance Rule Program Role in Implementation of Reliability Assurance Program (RAP) in the Operations Phase | Conforms |
| C.III.1 17.6.7 | Maintenance Rule Program Implementation | Conforms |
| C.III.1 Chapter 18 | Human Factors Engineering | Conforms |
| | HFE principles incorporated into: | |
| | (1) Planning and management | Conforms. Addressed in DCD Section 18.2. |
| | (2) Plant design processes not closed with design certification | Conforms. Addressed in DCD Tier 1, ITAAC Table 3.3-1. |
| | (3) HSI, procedures, and training | Conforms. Addressed in DCD Tier 1, ITAAC Table 3.3-1, Items 6, 7, and 8. |
| | (4) implementation of the design | Conforms. Addressed in DCD Tier 1, ITAAC Table 3.3-1, Item 10. |
| | (5) monitoring of performance at the site | Conforms. Addressed in DCD Tier 1, ITAAC Table 3.3-1, Item 11. |
| | Applicant program addresses normal and emergency, maintenance, test, inspection and surveillance activities | Conforms. Addressed in DCD Section 18.1. |
| | FSAR/DCD describe objectives and scope of the applicant's activities related to element, methodology, and results for (12 HFE elements) | Conforms. Addressed in DCD Sections 18.3 through 18.13. |
| | Applicant should reference detailed implementation plan reviewed and approved as part of design certification | Conforms. Addressed in DCD Section 18.2.1. |
| C.I 18.1 | HFE Program Management | Conforms. Addressed in DCD Sections 18.2.2 and 18.2.3. |
| C.I 18.1.1 | General HFE Program and Scope | Conforms. Addressed in DCD Sections 18.2.1 and 18.2.2. |
| C.I 18.1.2 | HFE Team and Organization | Conforms. Addressed in DCD Section 18.2.3. |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
(Sheet 37 of 39) [EF3 COL 1.9-3-A]

| Section | Section Title | Conformance Evaluation |
|--------------|--|---|
| C.I 18.1.3 | HFE Process and Procedures | Conforms. Addressed in DCD Section 18.2.1 and 18.2.2. |
| C.I 18.1.4 | HFE Issues Tracking | Conforms. Addressed in DCD Section 18.2.2. |
| C.I 18.1.5 | HFE Technical Program | Conforms. Addressed in DCD Sections 18.3 through 18.13. |
| C.I 18.2.1 | Objectives and scope | Conforms. Addressed in DCD Section 18.3.1. |
| C.I 18.2.2.1 | OER Process | Conforms. Addressed in DCD Section 18.3.2. |
| C.I 18.2.2.2 | Predecessor plants and systems | Conforms. Addressed in DCD Section 18.3.2.1. |
| C.I 18.2.2.3 | Risk-important human actions | Conforms. Addressed in DCD Section 18.3.2.2. |
| C.I 18.2.2.4 | HFE technology | Conforms. Addressed in DCD Section 18.3.2.3. |
| C.I 18.2.2.5 | Recognized industry issues | Conforms. Addressed in DCD Section 18.3.2.4. |
| C.I 18.2.2.6 | Issues Identified by plant personnel | Conforms. Addressed in DCD Section 18.3.2.5. |
| C.I 18.2.2.7 | Issue Analysis, Tracking, and Review | Conforms. Addressed in DCD Section 18.3.2.6. |
| C.I 18.2.3 | Results | Conforms. Addressed in DCD Section 18.3.3. |
| C.I 18.3.1 | Objectives and Scope | Conforms. Addressed in DCD Section 18.4.2. |
| C.I 18.3.1.1 | Functional Requirements Analysis | Conforms. Addressed in DCD Section 18.4.1. |
| C.I 18.3.1.2 | Function Allocation Analysis | Conforms. Addressed in DCD Section 18.4.2. |
| C.I 18.3.2.1 | Methodology for Functional Requirements Analysis | Conforms. Addressed in DCD Section 18.4.1. |
| C.I 18.3.2.2 | Methodology for Function Allocation Analysis | Conforms. Addressed in DCD Section 18.4.2. |
| C.I 18.3.3 | Results | Conforms. Addressed in DCD Sections 18.4.1 and 18.4.2. |
| C.I 18.4.1 | Objectives and Scope | Conforms. Addressed in DCD Sections 18.5.1 and 18.5.2 |
| C.I 18.4.2 | Methodology | Conforms. Addressed in DCD Sections 18.5.1 and 18.5.2 |
| C.I 18.4.3 | Results | Conforms. Addressed in DCD Sections 18.5.1 and 18.5.2 |
| C.I 18.5.1 | Objectives and Scope | Conforms. Addressed in DCD Section 18.6.2. |
| C.I 18.5.2 | Methodology | Conforms. Addressed in DCD Sections 18.6.4 and 18.6.5. |
| C.I 18.5.3 | Results | Conforms. Addressed in DCD Section 18.6.6. |
| C.I 18.6.1 | Objectives and Scope | Conforms. Addressed in DCD Section 18.7.1. |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
(Sheet 38 of 39) [EF3 COL 1.9-3-A]

| Section | Section Title | Conformance Evaluation |
|---------------|---|--|
| C.I 18.6.2 | Methodology | Conforms. Addressed in DCD Section 18.7.2. |
| C.I 18.6.3 | Results | Conforms. Addressed in DCD Section 18.7.3. |
| C.I 6.3.2.8 | Manual Actions | Conforms. Addressed in DCD Section 18.7.2. |
| C.I 18.7.1 | Objectives and scope | Conforms. Addressed in DCD Section 18.8.1. |
| C.I 18.7.2.1 | HSI Design Inputs | Conforms. Addressed in DCD Section 18.8.1. |
| C.I 18.7.2.2 | Concept of operations | Conforms. Addressed in DCD Section 18.8.1. |
| C.I 18.7.2.3 | Functional Requirements Specification | Conforms. Addressed in DCD Section 18.8.1. |
| C.I 18.7.2.4 | HSI Concept Design | Conforms. Addressed in DCD Section 18.8.1. |
| C.I 18.7.2.5 | HSI Detailed Design and Integration | Conforms. Addressed in DCD Section 18.8.1. |
| C.I 18.7.2.6 | HSI Tests and Evaluations | Conforms. Addressed in DCD Section 18.8.1. |
| C.I 18.7.3.1 | Overview of HSI Design and Its Key Features | Conforms. Addressed in DCD Section 18.8.1. |
| C.I 18.7.3.2 | Safety Aspects of the HSI | Conforms. Addressed in DCD Section 18.8.1. |
| C.I 18.7.3.3 | HSI Change Process | Conforms. Addressed in DCD Section 18.13.3. |
| C.I 18.8.1 | Objectives and Scope | Conforms. Addressed in DCD Section 18.9.1. |
| C.I 18.8.2 | Methodology | Conforms. Addressed in DCD Section 18.9.2. |
| C.I 18.8.3 | Results | Conforms. Addressed in DCD Section 18.9.3. |
| C.I 18.9.1 | Objectives and Scope | Conforms. Addressed in DCD Sections 18.10.1 and 18.10.2. |
| C.I 18.9.2 | Methodology | Conforms. Addressed in DCD Sections 18.10.3 and 18.10.4. |
| C.I 18.9.3 | Results | Conforms. Addressed in DCD Section 18.10.5. |
| C.I 18.10.1 | Objectives and Scope | Conforms. Addressed in DCD Section 18.11 and 18.11.1. |
| C.I 18.10.2 | Methodology | Conforms. Addressed in DCD Section 18.11. |
| C.I 18.10.2.1 | Operational Conditions Sampling | Conforms. Addressed in DCD Section 18.11. |
| C.I 18.10.2.2 | Design Verification | Conforms. Addressed in DCD Section 18.11. |
| C.I 18.10.2.3 | Integrated System Validation | Conforms. Addressed in DCD Section 18.11. |
| C.I 18.10.2.4 | Human Engineering Discrepancy Resolution | Conforms. Addressed in DCD Section 18.11. |
| C.I 18.10.3 | Results | Conforms. Addressed in DCD Section 18.11.2. |
| C.I 18.11.1 | Objectives and Scope | Conforms. Addressed in DCD Section 18.12.1. |
| C.I 18.11.2 | Methodology | Conforms. Addressed in DCD Section 18.12.2. |

Table 1.9-203 Conformance with the FSAR Content Guidance in RG 1.206
(Sheet 39 of 39) [EF3 COL 1.9-3-A]

| Section | Section Title | Conformance Evaluation |
|--------------------|--|--|
| C.I 18.11.3 | Results | Conforms. Addressed in DCD Section 18.12.3. |
| C.I 18.12.1 | Objectives and Scope | Conforms. Addressed in DCD Sections 18.13.1 and 18.13.2. |
| C.I 18.12.2 | Methodology | Conforms. Addressed in DCD Sections 18.13.2 and 18.13.3. |
| C.I 18.12.3 | Results | Conforms. Addressed in DCD Section 18.13.4. |
| C.III.1 Chapter 19 | Probabilistic Risk Assessment and Severe Accident Evaluation | Conforms. As discussed in RG 1.206, Section C.III.1 .10, the FSAR follows the organization and numbering of the referenced certified design. |

Table 1.9-204 Industrial Codes and Standards (Sheet 1 of 4) [EF3 SUP 1.9-1]

| Code or Standard Number | Year | Title |
|---|------|--|
| American Concrete Institute (ACI) | | |
| 318-08 | 2008 | Building Code Requirements for Structural Concrete and Commentary |
| 349-06 | 2006 | Code Requirements for Nuclear Safety-Related Concrete Structures and Commentary |
| American Nuclear Society (ANS) | | |
| 2.8 | 1992 | Determining Design Basis flooding at Power Reactor Sites an American |
| 3.1 | 1993 | Selection, Qualification, and Training of Personnel for Nuclear Power Plants |
| American National Standards Institute | | |
| N323A | 1997 | Radiation Protection Instrumentation Test and Calibration, Portable Survey Instruments |
| N323D | 2002 | Installed Radiation Protection Instrumentation |
| B30.2 | 2001 | Overhead and Gantry Cranes |
| American Society of Civil Engineers (ASCE) | | |
| ASCE 43-05 | 2005 | Seismic Design Criteria for Structures, Systems, and Components in Nuclear Facilities |
| ASCE SEI/ASCE 7-05 | 2005 | Minimum Design Loads for Buildings and other Structures |
| ASCE Practice No. 70 | 1990 | Evapotranspiration and Irrigation Water Requirements |
| American Society Heating, Refrigerating, and Air-Conditioning (ASHRAE) | | |
| ASHRAE Handbook | 2005 | American Society Heating, Refrigerating, and Air-Conditioning Engineers Handbook |
| American Society of Mechanical Engineers (ASME) | | |
| NQA-1 | 1994 | Quality Assurance Programs Requirements for Nuclear Facilities |
| Boiler and Pressure Vessel Code, Section IX | 2007 | Qualification Standard for Welding and Brazing Procedures, Welder, Brazers and Welding and Brazing Operators |
| OM Code | | Code for the Operation and Maintenance of Nuclear Power Plants |
| American Society for Testing and Materials (ASTM) | | |
| C88-05 | 2005 | Standard Test Method for Soundness of Aggregates by Use of Sodium Sulfate or Magnesium Sulfate |

Table 1.9-204 Industrial Codes and Standards (Sheet 2 of 4)

[EF3 SUP 1.9-1]

| Code or Standard Number | Year | Title |
|--|------|--|
| American Society for Testing and Materials (ASTM) (Continued) | | |
| C131-06 | 2006 | Standard Test Method for Resistance to Degradation of Small-Size Coarse Aggregate by Abrasion and Impact in the Los Angeles Machine |
| C535-03 | 2003 | Standard Test Method for Resistance to Degradation of Large-Size Coarse Aggregate by Abrasion and Impact in the Los Angeles Machine |
| D422-63 | 2002 | Standard Test Method for Particle-Size Analysis of Soils |
| D512-04 | 2004 | Standard Test Methods for Chloride Ion in Water |
| D516-02 | 2002 | Standard Test Methods for Sulfate Ion in Water, |
| D698-07 | 2007 | Standard Test Methods for Laboratory Compaction Characteristics of Soil Using Standard Effort (12,400 ft-lbf/ft ³ (600 kN-m/m ³)) |
| D854-06 | 2006 | Standard Test Methods for Specific Gravity of Soil Solids by Water Pycnometer |
| D1140-00 | 2006 | Standard Test Methods for Amount of Material in Soils Finer than No. 200 (75-mm) |
| D1557-07 | 2007 | Standard Test Methods for Laboratory Compaction Characteristics of Soil Using Modified Effort (56,000 ft-lbf/ft ³ (2,700 kN-m/m ³)) |
| D1586-99 | 1999 | Standard Test Method for Penetration Test and Split-Barrel Sampling of Soils |
| D1587-00 | 2000 | Standard Practice for Thin-Walled Tube Sampling of Soils for Geotechnical Purposes |
| D2113-06 | 2006 | Standard Practice for Rock Core Drilling and Sampling of Rock for Site Investigation |
| D2166-06 | 2006 | Standard Test Method for Unconfined Compressive Strength of Cohesive Soil |
| D2216-00 | 2000 | Standard Test Methods for Laboratory Determination of Water (Moisture) Content of Soil and Rock by Mass |
| D2435-04 | 2004 | Standard Test Methods for One-Dimensional Consolidation Properties of Soils Using Incremental Loading |
| D2487-06 | 2006 | Standard Practice for Classification of Soils for Engineering Purposes |
| D2488-06 | 2006 | Standard Practice for Description and Identification of Soils (Visual-Manual Procedure) |

Table 1.9-204 Industrial Codes and Standards (Sheet 3 of 4) [EF3 SUP 1.9-1]

| Code or Standard Number | Year | Title |
|--|------|--|
| American Society for Testing and Materials (ASTM) (Continued) | | |
| D2850-03a | 2003 | Standard Test Method for Unconsolidated-Undrained Triaxial Compression Test on Cohesive Soils |
| D3080-04 | 2004 | Standard Test Methods for Direct Shear Test of Soil Under Consolidated Drained Conditions |
| D3550-01 | 2001 | Standard Practice for Thick Wall, Ring-Lined, Split Barrel, Drive Sampling of Soils |
| D4220-95 | 2000 | Standard Practices for Preserving and Transporting Soil Samples |
| D4253-00 | 2000 | Standard Test Methods for Maximum Index Density and Unit Weight of Soils Using a Vibratory Table |
| D4254-06 | 2006 | Standard Test Methods for Minimum Index Density and Unit Weight of Soils and Calculation of Relative Density |
| D4318-05 | 2005 | Standard Test Methods for Liquid Limit, Plastic Limit, and Plasticity Index of Soils |
| D4767-04 | 2004 | Standard Test Method for Consolidated Undrained Triaxial Compression Test for Cohesive Soils |
| D5079-02 | 2002 | Standard Practices for Preserving and Transporting Rock Core Samples |
| D5084-03 | 2003 | Standard Test Methods for Measurement of Hydraulic Conductivity of Saturated Porous Materials Using a Flexible Wall Permeameter |
| D5607-02 | 2006 | Standard Test Methods for Performing Laboratory Direct Shear Strength of Rock Specimens Under Constant Normal Force |
| D6151-97 | 1997 | Standard Practice for Using Hollow-Stem Augers for Geotechnical Exploration and Soil Sampling |
| D6914-04 | 2004 | Standard Practice for Sonic Drilling for Site Characterization and the Installation of Subsurface Monitoring Devices, |
| D7012-07 | 2007 | Standard Test Methods for Compressive Strength and Elastic Moduli of Intact Rock Core Specimens under Varying States of Stress and Temperature |
| ASTM E-119 | 2007 | Fire Test of Building Construction Materials |
| G51-95 | 2005 | Standard Test Methods for Measuring pH of Soil for Use in Corrosion Testing |
| Applicable Building Codes | | |
| Michigan Building Code | 2003 | Michigan Building Code |

Table 1.9-204 Industrial Codes and Standards (Sheet 4 of 4) [EF3 SUP 1.9-1]

| Code or Standard Number | Year | Title |
|---|-------------|--|
| Applicable Building Codes (Continued) | | |
| Michigan Department of Transportation | 2003 | Standard Specifications for Construction, Section 902 – Aggregates |
| Institute of Electrical and Electronics Engineers (IEEE) | | |
| C2 | 2007 | National Electric Safety Code |
| National Fire Protection Association (NFPA) | | |
| NFPA 10 | 2007 | Standard for Portable Fire Extinguishers |
| NFPA 15 | 2007 | Standard for Water Spray Fixed Systems for Fire Protection |
| NFPA 25 | 2008 | Recommended Practices for Inspection, Testing, and Maintenance of Standpipes and Hose Systems |
| NFPA 45 | 2004 | Standard on Fire Protection for Laboratories Using Chemicals |
| NFPA 55 | 2005 | Standard for Storage, Use, and Handling of Compressed Gases and Cryogenic Fluids in Portable and Stationary Containers, Cylinders, and Tanks |
| NFPA 72 | 2007 | National Fire Alarm Code |
| NFPA 422 | 2004 | Guide for Aircraft Accident/Incident Response Assessment |
| NFPA 804 | 2006 | Standard for Fire Protection for Advanced Light Water Reactor Electric Generating Plants |
| Occupational Safety and Health Act (OSHA) | | |
| 29 CFR 1910 | 2006 | Occupational Safety and Health Standards |
| 29 CFR 1926 | 2006 | Safety and Health Regulations for Construction |
| United States Army Corps of Engineers (USACE) | | |
| EM 1110-2908 | 1994 | Engineering and Design Rock Foundations |
| Environmental Protection Agency (EPA) | | |
| 40 CFR 60 | 2006 | EPA Standards of Performance for Stationary Compression Ignition Internal Combustion Engines |

Table 1.9-205 NUREG Reports Cited (Sheet 1 of 2)

[EF3 SUP 1.9-2]

| NUREG No. | Issue Date | Title | Comment/Section Where Discussed |
|-----------------|------------|--|--|
| 0016, Rev. 1 | 01/1979 | Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Boiling Water Reactors (BWRs) | 12.2, Table 1.9-202 RG 1.111 |
| 0570 | 06/1979 | Toxic Vapor Concentrations in the Control Room Following a Postulated Accidental Release | 6.4 |
| 0612 | 07/1980 | Control of Heavy Loads at Nuclear Power Plants | 13.5, 9.1.5 |
| 0737 | 11/1980 | Clarification of TMI Action Plan Requirements | 12.5, 13.5, Appendix 14AA |
| 0800 | 03/2007 | Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants | 1.1, 2.0, 2.2, 2.3, 2.4, 2.5, 9.3, 11.5, Appendix 14AA |
| 1437 | 05/1996 | Generic Environmental Impact Statement for License Renewal of Nuclear Plants, U.S. Nuclear Regulatory Commission | 12.2 |
| 1736 | 10/2001 | Consolidated Guidance: 10 CFR Part 20 – Standards for Protection Against Radiation | 1.9 |
| 1805 | 12/2004 | Fire Dynamics Tools (FDTs) Quantitative Fire Hazard Analysis Methods for the U.S. Nuclear Regulatory Commission Fire Protection Inspection Program | 2.2 |
| 0654/FEMA-REP-1 | 11/1980 | Criteria for Preparation and Evaluation of Radiological Emergency Response Plans and Preparedness in Support of Nuclear Power Plants | 1.9, Emergency Plan |
| 0696 | 02/1981 | Functional Criteria for Emergency Response Facilities”, Final Report | Emergency Plan |
| 0728 | 04/2005 | NRC Incident Response Plan | Emergency Plan |
| 0737, Sup 1 | 12/1982 | Requirements for Emergency Response Capability | 13.5, Emergency Plan |
| CR-1745 | 11/1980 | Analysis of Techniques for Estimating Evacuation Times for Emergency Planning Zones, | Emergency Plan, ETE |
| CR-2650 | 10/1982 | Allowable Shipment Frequencies for the Transport of Toxic Gases Near Nuclear Power Plants | 2.2.3 |
| CR-2858 | 11/82 | PAVAN: An Atmospheric Dispersion Program for Evaluating Design Basis Accidental Releases of Radioactive Materials for Nuclear Power Stations | 2.3.4 |
| CR-2919 | 09/1982 | XOQDDOQ: Computer Program for the Meteorological Evaluation of Routine Effluent Releases at Nuclear Power Stations | 2.3.5, Appendix 2B |
| CR-3145 | 10/1992 | Geophysical Investigations of the Western Ohio – Indiana Region | 2.5.1, 2.5.2 |

Table 1.9-205 NUREG Reports Cited (Sheet 2 of 2)

[EF3 SUP 1.9-2]

| NUREG No. | Issue Date | Title | Comment/Section Where Discussed |
|-----------|------------|--|---------------------------------|
| CR-4013 | 04/1986 | LADTAP II Technical Reference and User Guide | 12.2 |
| CR-4461 | 02/2007 | Tornado Climatology of the United States | 2.3 |
| CR-4653 | 03/1987 | GASPAR II Technical Reference and User Guide | 12.2 |
| CR-4831 | 03/1992 | State of the Art in Evacuation Time Estimate Studies for Nuclear Power Plants | Emergency Plan |
| CR-5250 | 1986 | Seismic Hazard Characterization of 69 Nuclear Plant Sites East of the Rocky Mountains: Questionnaires," prepared by Lawrence Livermore National Laboratory | 2.5.2 |
| CR-5503 | 07/1999 | Techniques for Identifying Faults and Determining Their Origins | 2.5.3 |
| CR-5512 | 10/1992 | Residual Radioactive Contamination from Decommissioning | 2.4 |
| CR-6331 | 07/1997 | Atmospheric Relative Concentrations in Building Wakes | 2.3.4 |
| CR-6372 | 04/1997 | Recommendations for Probabilistic Seismic Hazard Analysis: Guidance on Uncertainty and Use of Experts | 2.5.2 |
| CR-6624 | 10/1992 | Recommendations for Revision of Regulatory Guide 1.78 | 2.2 |
| CR-6948 | 11/2007 | Integrated Ground-Water Monitoring Strategy for NRC-Licensed Facilities and Sites: Logic, Strategic Approach and Discussion | 2.4.12 |
| CR-6728 | 10/2001 | Technical Basis for Revision of Regulatory Guidance on Design Ground Motions: Hazard- and Risk-consistent Ground Motion Spectra Guidelines | 2.5 |
| CR-6863 | 01/2005 | Development of Evacuation Time Estimate Studies for Nuclear Power Plants | Emergency Plan |

1.10 Summary of COL Items

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

Add the following at the end of this section.

EF3 SUP 1.10-1

[Table 1.10-201](#) lists the FSAR location(s) where the individual COL items from the DCD are addressed.

Table 1.10-201 Summary of FSAR Sections Where DCD COL Items Are Addressed
(Sheet 1 of 7) [EF3 SUP 1.10-1]

| Item No. | Subject/Description of Item | FSAR Section |
|----------|---|---|
| 1.1-1-A | Establish Rated Electrical Output | 1.1.2.7 |
| 1.3-1-A | Update Table | 1.3.1 |
| 1.7-1-H | Final Design Configuration Confirmation | 1.7 |
| 1.9-3-A | SRP and Regulatory Guide Applicability | SRP: Table 1.9-201 RGs: 1.9.1 and 1.9.2 RG 1.206: Table 1.9-203 |
| 1.11-1-A | Address Table 1.11-1 Items That Refer to Notes (2) and (7) | 1.11.1 and Table 1.11-201 |
| 1C.1-1-A | Handling of Safeguards Information | Appendix 1C, Table 1C-201 |
| 1C.1-2-A | Emergency Preparedness and Response Actions | Appendix 1C, Table 1C-202 |
| 2.0-1-A | Site Characteristics Demonstration | 2.0 |
| 2.0-2-A | Site Location and Description Information in Accordance with SRP 2.1.1 | 2.0 and 2.1.1 |
| 2.0-3-A | Site-Specific Exclusion Area Authority and Control Information in Accordance with SRP 2.1.2 | 2.0 and 2.1.2 |
| 2.0-4-A | Describe the Population Distribution in Accordance with SRP 2.1.3 | 2.0 and 2.1.3 |
| 2.0-5-A | Identify Potential Hazards in the Site Vicinity, in Accordance with SRP 2.2.1 - 2.2.2 | 2.0 and 2.2 |
| 2.0-6-A | Evaluation of Potential Accidents in Accordance with SRP 2.2.3 | 2.0 and 2.2.3 |
| 2.0-7-A | Regional Climatology in Accordance with SRP 2.3.1 | 2.0 and 2.3.1 |
| 2.0-8-A | Local Meteorology in Accordance with SRP 2.3.2 | 2.0 and 2.3.2 |
| 2.0-9-A | Onsite Meteorological Measurement Programs in Accordance with SRP 2.3.3 | 2.0 and 2.3.3 |
| 2.0-10-A | Short-Term Diffusion Estimates for Accidental Atmospheric Releases in Accordance with SRP 2.3.4 | 2.0 and 2.3.4 |
| 2.0-11-A | Long-Term Diffusion Estimates in Accordance with SRP 2.3.5 | 2.0 and 2.3.5 |
| 2.0-12-A | Hydraulic Description Maximum Ground Water Level in Accordance with SRP 2.4.1 | 2.0 and 2.4.1 |
| 2.0-13-A | Protection of Below-Grade Penetrations and Access Openings from Floods in Accordance with SRP 2.4.2 | 2.0 and 2.4.2 |
| 2.0-14-A | Probable Maximum Flood on Streams and Rivers in Accordance with SRP 2.4.3 | 2.0 and 2.4.3 |
| 2.0-15-A | Potential Dam Failures Seismically Induced in Accordance with SRP 2.4.4 | 2.0 and 2.4.4 |

Table 1.10-201 Summary of FSAR Sections Where DCD COL Items Are Addressed
(Sheet 2 of 7) [EF3 SUP 1.10-1]

| Item No. | Subject/Description of Item | FSAR Section |
|------------|--|--|
| 2.0-16-A | Probable Maximum Surge and Seiche Flooding in Accordance with SRP 2.4.5 | 2.0 and 2.4.5 |
| 2.0-17-A | Probable Maximum Tsunami in Accordance with SRP 2.4.6 | 2.0 and 2.4.6 |
| 2.0-18-A | Ice Effects in Accordance with SRP 2.4.7 | 2.0 and 2.4.7 |
| 2.0-19-A | Cooling Water Canals and Reservoirs in Accordance with SRP 2.4.8 | 2.0 and 2.4.8 |
| 2.0-20-A | Channel Diversion in Accordance with SRP 2.4.9 | 2.0 and 2.4.9 |
| 2.0-21-A | Flooding Protection Requirements in Accordance with SRP 2.4.10 | 2.0 and 2.4.10 |
| 2.0-22-A | Cooling Water Supply in Accordance with SRP 2.4.11 | 2.0 and 2.4.11 |
| 2.0-23-A | Groundwater in Accordance with SRP 2.4.12 | 2.0 and 2.4.12 |
| 2.0-24-A | Accidental Releases of Liquid Effluents in Ground and Surface Waters in Accordance with SRP 2.4.13 | 2.0 and 2.4.13 |
| 2.0-25-A | Technical Specifications and Emergency Operation Requirements in Accordance with SRP 2.4.14 | 2.0 and 2.4.14 |
| 2.0-26-A | Basic Geologic and Seismic Information in Accordance with SRP 2.5.1 | 2.0 and 2.5.1 |
| 2.0-27-A | Vibratory Ground Motion in Accordance with SRP 2.5.2 | 2.0 and 2.5.2 |
| 2.0-28-A | Surface Faulting in Accordance with SRP 2.5.3 | 2.0 and 2.5.3 |
| 2.0-29-A | Stability of Subsurface Materials and Foundations in Accordance with SRP 2.5.4 | 2.0 and 2.5.4 |
| 2.0-30-A | Stability of Slopes in Accordance with SRP 2.5.5 | 2.0 and 2.5.5 |
| 2A.2-1-A | Confirmation of the ESBWR γ/Q Values | 2.3.4.3 and 2A.2.4 |
| 2A.2-2-A | Confirmation of the Reactor Building γ/Q Values | 2A.2.5 |
| 3.9.9-1-A | Reactor Internals Vibration Analysis, Measurement and Inspection Program | 3.9.2.4 |
| 3.9.9-2-A | ASME Class 2 or 3 or Quality Group D Components with 60-Year Design Life | 3.9.3.1 |
| 3.9.9-3-A | Inservice Testing Programs | 3.9.6 |
| 3.9.9-4-A | Snubber Inspection and Test Program | 3.9.3.7.1(3)e |
| 3.10.4-1-A | Dynamic Qualification Report | 3.10.1.4 |
| 3.11-1-A | Environmental Qualification Document (EQD) | 3.11.4.4 |
| 4.3-1-A | Variances from Certified Design | 4.3.3.1 |
| 4A-1-A | Variances from Certified Design | 4A.1 |
| 5.2-1-A | Preservice and Inservice Inspection Program Description | 5.2.4, 5.2.4.3.4, 5.2.4.6, 5.2.4.11, and 6.6 |

Table 1.10-201 Summary of FSAR Sections Where DCD COL Items Are Addressed
(Sheet 3 of 7) [EF3 SUP 1.10-1]

| Item No. | Subject/Description of Item | FSAR Section |
|------------|---|---------------------------------|
| 5.2-2-A | Leak Detection Monitoring | 5.2.5 and 5.2.5.9 |
| 5.2-3-A | Preservice and Inservice Inspection NDE Accessibility Plan Description | 5.2.4.2 |
| 5.3-2-A | Materials and Surveillance Capsule | 5.3.1.8 |
| 6.4-1-A | CRHA Procedures and Training | 6.4.4 |
| 6.4-2-A | Toxic Gas Analysis | 6.4.5 |
| 6.6-1-A | Program Description | 6.6 |
| 6.6-2-A | PSI/ISI NDE Accessibility Plan Description | 6.6.2 |
| 8.2.4-1-A | Transmission System Description | 8.2.1.1 |
| 8.2.4-2-A | Switchyard Description | 8.2.1.2.1 |
| 8.2.4-3-A | Normal Preferred Power | 8.2.1.2 |
| 8.2.4-4-A | Alternate Preferred Power | 8.2.1.2 |
| 8.2.4-5-A | Protective Relaying | 8.2.1.2.2 |
| 8.2.4-6-A | Switchyard DC Power | 8.2.1.2.1 |
| 8.2.4-7-A | Switchyard AC Power | 8.2.1.2.1 |
| 8.2.4-8-A | Switchyard Transformer Protection | 8.2.1.2.1 |
| 8.2.4-9-A | Stability and Reliability of the Offsite Transmission Power Systems | 8.2.2.1 |
| 8.2.4-10-A | Interface Requirements | 8.2.2.1 |
| 8.3.4-1-A | Safety-Related Battery Float and Equalizing Voltage Values | 8.3.2.1.1 |
| 8.3.4-2-A | Identification and Monitoring of Underground or Inaccessible Power and Control Cables to the PSWS and DG Fuel Oil Transfer System Equipment That Have Accident Mitigating Functions | 8.3.3.2 |
| 8A.2.3-1-A | Cathodic Protection System | 8A.2.1 |
| 9.1-4-A | Fuel Handling Operations | 9.1.4.13, 9.1.4.18 and 9.1.4.19 |
| 9.1-5-A | Handling of Heavy Loads | 9.1.5.6, 9.1.5.8, and 9.1.5.9 |
| 9.2.1-1-A | Material Selection | 9.2.1.2 |
| 9.2.5-1-A | Post Seven day Makeup to Ultimate Heat Sink (UHS) | 9.2.5 |
| 9.3.2-1-A | Post-Accident Sampling Program | 9.3.2.2 |
| 9.3.9-1-A | Implementation of Hydrogen Water Chemistry | 9.3.9 |
| 9.3.9-2-A | Hydrogen and Oxygen Storage and Supply | 9.3.9.2 |
| 9.3.10-1-A | Oxygen Storage Facility | 9.3.10.2 |
| 9.3.11-1-A | Determine Need for Zinc Injection System | 9.3.11.2 |
| 9.3.11-2-A | Provide System Description for Zinc Injection System | 9.3.11.4 |

Table 1.10-201 Summary of FSAR Sections Where DCD COL Items Are Addressed
(Sheet 4 of 7) [EF3 SUP 1.10-1]

| Item No. | Subject/Description of Item | FSAR Section |
|------------|---|---|
| 9.5.1-1-A | Secondary Firewater Storage Source | 9.5.1.4 |
| 9.5.1-2-A | Secondary Firewater Capacity | 9.5.1.4 |
| 9.5.1-4-A | Piping and Instrument Diagrams | 9.5.1.2, 9.5.1.4, 9.5.1.5, and Figure 9.5-201 |
| 9.5.1-5-A | Fire Barriers | 9.5.1.10 |
| 9.5.1-6-A | Smoke Control | 9.5.1.11 |
| 9.5.1-7-A | Fire Hazards Analysis (FHA) Compliance Review | 9.5.1.12 |
| 9.5.1-8-A | Fire Protection (FP) Program Description | 9.5.1.15 |
| 9.5.1-10-A | Fire Brigade | 9.5.1.15.4, 13.1.2.1.5 |
| 9.5.1-11-A | Quality Assurance | 9.5.1.15.9 |
| 9.5.2-1-A | Emergency Notification System | 9.5.2.2 |
| 9.5.2-2-A | Grid Transmission Operator | 9.5.2.2 |
| 9.5.2-3-A | Offsite Interfaces (1) | 9.5.2.2 |
| 9.5.2-4-A | Offsite Interfaces (2) | 9.5.2.2 |
| 9.5.2-5-A | Fire Brigade Radio System | 9.5.2.2 |
| 9.5.4-1-A | Fuel Oil Capacity | 9.5.4.2 |
| 9.5.4-2-A | Protection of Underground Piping | 9.5.4.2 |
| 9A.7-1-A | Yard Fire Zone Drawings | 9A.4.7 |
| 9A.7-2-A | Fire Hazards Analysis for Site Specific Areas | 9A.4.7, 9A.5.7, 9A.5.8, and 9A.5.9 |
| 10.2-1-A | Turbine Maintenance and Inspection Program | 10.2.2.4, 10.2.2.7, 10.2.3.6 and 10.2.3.7 |
| 10.2-2-A | Turbine Missile Probability Analysis | 10.2.3.8 |
| 10.4-1-A | Leakage (of Circulating Water Into the Condenser) | 10.4.6.3 |
| 11.2-1-A | Implementation of IE Bulletin 80-10 | 11.2.2.3 |
| 11.2-2-A | Implementation of Part 20.1406 | 11.2.2.3 |
| 11.4-1-A | SWMS Processing Subsystem Regulatory Guide Compliance | 11.4.2.3.5 |
| 11.4-2-A | Compliance with IE Bulletin 80-10 | 11.4.2.3.5 |
| 11.4-3-A | Process Control Program | 11.4.2.3.5 |
| 11.4-4-A | Temporary Storage Facility | 11.4.1 |
| 11.4-5-A | Compliance with Part 20.1406 | 11.4.1 |
| 11.5-1-A | Subsystem Lower Limit of Detection | 11.5.4.7 |

Table 1.10-201 Summary of FSAR Sections Where DCD COL Items Are Addressed
(Sheet 5 of 7) [EF3 SUP 1.10-1]

| Item No. | Subject/Description of Item | FSAR Section |
|----------|---|--|
| 11.5-2-A | Offsite Dose Calculation Manual | 11.5.4.4 , 11.5.4.5 , 11.5.5.8 |
| 11.5-3-A | Process and Effluent Monitoring Program | 11.5 and 11.5.4.6 and Table 11.5-201 |
| 11.5-4-A | Site Specific Offsite Dose Calculation | 11.5.4.8 |
| 11.5-5-A | Instrument Sensitivities | 11.5.4.9 |
| 12.1-1-A | Regulatory Guide 8.10 | Appendix 12BB |
| 12.1-2-A | Regulatory Guide 1.8 | Appendix 12BB |
| 12.1-3-A | Operational Considerations | Appendix 12BB |
| 12.1-4-A | Regulatory Guide 8.8 | Appendix 12BB |
| 12.2-2-A | Airborne Effluents and Doses | 12.2.2.1 , 12.2.2.2 , and Table 2.0-201 |
| 12.2-3-A | Liquid Effluents and Doses | 12.2.2.4 |
| 12.2-4-A | Other Contained Sources | 12.2.1.5 |
| 12.3-2-A | Operational Considerations | 12.3.4 |
| 12.3-4-A | Compliance with 10 CFR 20.1406 | 12.3.1.5 |
| 12.5-1-A | Equipment, Instrumentation, and Facilities | Appendix 12BB |
| 12.5-2-A | Compliance with 10 CFR Part 50.34(f)(2)(xxvii) and NUREG-0737 Item III.D.3.3 | Appendix 12BB |
| 12.5-3-A | Radiation Protection Program | Appendix 12BB |
| 13.1-1-A | Organizational Structure | 9.5.1.15.3 , 13.1.1 through 13.1.3 , and Appendix 13AA |
| 13.2-1-A | Reactor Operator Training | 13.2.1 and Appendix 13BB |
| 13.2-2-A | Training for Non-Licensed Plant Staff | 13.2.2 and Appendix 13BB |
| 13.3-1-A | Identification of OSC and Communication Interfaces with Control Room and TSC | 13.3 and COLA Part 5, Sections II.F and II.H |
| 13.3-2-A | Identification of EOF and Communication Interfaces with Control Room and TSC | 13.3 and COLA Part 5, Sections II.F and II.H |
| 13.3-3-A | Decontamination Facilities | 13.3 and COLA Part 5, Section II.J |
| 13.4-1-A | Operation Programs | 13.4 |

Table 1.10-201 Summary of FSAR Sections Where DCD COL Items Are Addressed
(Sheet 6 of 7) [EF3 SUP 1.10-1]

| Item No. | Subject/Description of Item | FSAR Section |
|-----------|---|-------------------------|
| 13.4-2-A | Implementation Milestones | 13.4 |
| 13.5-1-A | Administrative Procedures Development Plan | 13.5.1 |
| 13.5-2-A | Plant Operating Procedures Development Plan | 13.5.2 |
| 13.5-3-A | Emergency Procedures Development | 13.5.2 |
| 13.5-4-A | Implementation of the Plant Procedures Plan | 13.5, 13.5.2 |
| 13.5-5-A | Procedures Included in Scope of Plan | 13.5.2 |
| 13.5-6-A | Procedures for Calibration, Inspection and Testing | 13.5.2 |
| 13.6-6-A | Key Control | 13.6.1.1.5 |
| 13.6-7-A | Redundancy and Equivalency of the CAS and Secondary Alarm Station | Physical Security Plan |
| 13.6-8-A | No Single Act Requirement for CAS and Secondary Alarm Station | 13.6.2 |
| 13.6-9-A | Operational Alarm Response Procedures | 13.6.1.1.3 |
| 13.6-10-A | Operational Surveillance Test Procedures | 13.6.1.1.8 |
| 13.6-11-A | Maintenance Test Procedures | 13.6.1.1.8 |
| 13.6-12-A | Operational Response Procedures to Security Events | 13.6.2 |
| 13.6-13-A | Operational Alarm Response Procedures | 13.6.1.1.3 |
| 13.6-14-A | Administrative Controls to Sensitive Cabinets | 13.6.1.1.5 |
| 13.6-15-A | Administrative Controls to Sensitive Equipment | 13.6.1.1.5 |
| 13.6-16-A | External Bullet Resisting Enclosures | 13.6.2 |
| 13.6-17-A | Site-Specific Locations of Security Barriers | 13.6.2 |
| 13.6-18-A | Ammunition for Armed Responders | 13.6.2 |
| 13.6-19-A | Site-Specific Update of the ESBWR Safeguards Assessment Report | 13.6.2 |
| 13.6-20-A | Physical Security ITAAC | 13.6.2 |
| 14.2-1-A | Description - Initial Test Program Administration | 14.2.2.1, Appendix 14AA |
| 14.2-2-A | Startup Administrative Manual | 14.2.2.1 |
| 14.2-3-A | Test Procedures | 14.2.2.2 |
| 14.2-4-A | Test Program Schedule and Sequence | 14.2.7 |
| 14.2-5-A | Site Specific Tests | 14.2.9 |
| 14.2-6-A | Site Specific Test Procedures | 14.2.9 |
| 14.3-1-A | Emergency Planning ITAAC | 14.3.8 |
| 14.3-2-A | Site-Specific ITAAC | 14.3.9 |
| 14.3A-1-1 | Establish a Schedule for Design Acceptance Criteria ITAAC Closure | 14.3A.1 |
| 16.0-1-A | COL Applicant Bracketed Items | 5.3.1.5, COLA Part 4 |

Table 1.10-201 Summary of FSAR Sections Where DCD COL Items Are Addressed
(Sheet 7 of 7) [EF3 SUP 1.10-1]

| Item No. | Subject/Description of Item | FSAR Section |
|-----------------|--|----------------------------|
| 17.2-1-A | QA Program for the Construction and Operations Phases | 17.2 |
| 17.2-2-A | QA Program for Design Activities | 17.2 |
| 17.3-1-A | Quality Assurance Program Document | 17.3 |
| 17.4-1-A | Identification of Site-Specific SSCs Within the Scope of the RAP | |
| 17.4-2-A | Operation Reliability Assurance Activities | 17.4.1 |
| 18.13-1-A | Milestone for HPM Implementation | 18.13.3 |
| 19.2.6-1-A | Seismic High Confidence Low Probability of Failure Margins | 19.2.3.2.4 |

1.11 Technical Resolutions of Task Action Plan Items, New Generic Issues, New Generic Safety Issues and Chernobyl Issues

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

1.11.1 Approach

Add the following at the end of this section.

| | |
|-------------------------|--|
| EF3 COL 1.11-1-A | Table 1.11-201 supplements DCD Table 1.11-1 to address the site-specific aspects of items that refer to Notes (2) and (7). |
|-------------------------|--|

| | |
|-----------------------|---|
| EF3 SUP 1.11-1 | Table 1.11-202 supplements DCD Table 1.11-1 to provide references to FSAR locations that provide additional information on specific issues. |
|-----------------------|---|

1.11.2 COL Information

1.11-1-A Address Table 1.11-1 Items that refer to Notes (2) and (7)

| | |
|-------------------------|---|
| EF3 COL 1.11-1-A | This COL item is addressed in Section 1.11 and Table 1.11-201 . |
|-------------------------|---|

Table 1.11-201 COL Item Resolutions Related to NUREG-0933 Table II Task Action Plan Items and New Generic Issues (Sheet 1 of 2) [EF3 COL 1.11-1-A]

| Action Plan Item/Issue Number | Description | Associated Location(s) Where Discussed and/or Technical Resolution |
|-------------------------------|--|--|
| Task Action Plan Items | | |
| A-33 | NEPA Review of Accident Risks | This environmental issue involves consideration of accidents on a risk specific basis. This subject is addressed in COLA Part 3, Chapter 7 . |
| B-1 | Environmental Technical Specifications | Issue is addressed in COLA Part 4, Sections 5.5.1 and 5.5.3, which address the Offsite Dose Calculation Manual and Radioactive Effluent Controls Program. See also Subsection 11.5.4.5 and Subsection 11.5.4.6 . |
| B-28 | Radionuclide/Sediment Transport Program | Issue is addressed in COLA Part 4, Sections 5.5.1 and 5.5.3, which address the Offsite Dose Calculation Manual and Radioactive Effluent Controls Program. See also Subsection 11.5.4.5 and Subsection 11.5.4.6 . |
| B-37 | Chemical Discharges to Receiving Waters | Issue is addressed in COLA Part 3, Section 3.3 , Section 3.6 , Section 5.2 and Subsection 5.3.2.2.2 . |
| B-38 | Reconnaissance Level Investigations | Issue is addressed in Chapter 2 and COLA Part 3, Chapter 2 . |
| B-39 | Transmission Lines | Issue is addressed in COLA Part 3, Section 3.7 , Section 4.6 , and Section 5.6 . |
| B-40 | Effects of Power Plant Entrainment on Plankton | Issue is addressed in COLA Part 3, Subsection 5.3.1.2 . |
| B-41 | Impacts on Fisheries | Impact of power plant operation on fishery resources is addressed in COLA Part 3, Subsection 5.3.1.2 and Subsection 5.2.2.1 . |
| B-42 | Socioeconomic Environmental Impacts | Issue is addressed in COLA Part 3, Section 4.4 and Section 5.8 . |
| B-43 | Value of Aerial Photographs for Site Evaluation | Work completed to date on this issue is published in NUREG/CR-2861. The use of aerial photography is discussed in COLA Part 3, Section 2.1 and Subsection 2.4.1 . |
| C-16 | Assessment of Agricultural Land in Relation to Power Plant Siting and Cooling System Selection | (3) The impact of construction and power plant operation on agricultural land use is addressed in COLA Part 3, Section 4.1 and Section 5.1 . Water use for agricultural lands is addressed in COLA Part 3, Subsection 2.3.2 and Subsection 2.3.3 . |

**Table 1.11-201 COL Item Resolutions Related to NUREG-0933 Table II Task Action
Plan Items and New Generic Issues (Sheet 2 of 2) [EF3 COL 1.11-1-A]**

| Action Plan Item/Issue Number | Description | Associated Location(s) Where Discussed and/or Technical Resolution |
|-------------------------------------|--------------------|--|
| NEW GENERIC ISSUES | | |
| 184 | Endangered Species | Issue is addressed in COLA Part 3, Subsection 2.4.1.2 , Subsection 2.4.2.2 , Subsection 2.4.2.4 Subsection 4.3.1 , Subsection 4.3.2.4 , Subsection 5.3.1.2 , and Subsection 5.3.3.2 . |

**Table 1.11-202 Supplementary Resolutions Related to NUREG-0933 Table II TMI
Action Plan Items and Human Factors Issues** [EF3 COL 1.11-2]

| Action Plan Item/Issue Number | Description | Associated Location(s) Where Discussed and/or Technical Resolution |
|-------------------------------------|---|--|
| TMI Action Plan Items | | |
| 1.A.1.1 | Shift Technical Advisor | Subsection 13.1.2 and DCD Section 18.6 |
| 1.A.1.2 | Shift Supervisor Administrative Duties | Subsection 13.1.2 |
| 1.A.1.3 | Shift Manning | Subsection 13.1.2 , Table 13.1-202 , Figure 13.1-203 , and DCD Section 18.6 |
| 1.A.2.1(1) | Qualifications – Experience | Subsection 13.1.3 , Table 13.1-201 , Section 17.5 , and DCD Section 18.6 |
| 1.C.3 | Shift Supervisor Responsibilities | Subsection 13.1.2 |
| 1.F.2(6) | Increase the Size of Licensees' QA Staff | Table 13.1-201 and Section 17.5 |
| 1.F.2(9) | Clarify Organizational Reporting Levels for the QA Organization | Subsection 13.1.1 , Table 13.1-201 , and Section 17.5 |
| II.B.3 | Post Accident Sampling | Appendix 12BB |
| III.D.3.3 | In-Plant Radiation Monitoring | Appendix 12BB |
| Human Factors Issues | | |
| HF1.1 | Shift Staffing | Table 13.1-202 and Subsection 13.1.2 |

EF3 SUP 1.12-1

1.12 Impact of Construction Activities on Fermi 2

1.12.1 Introduction

Paragraph 10 CFR 52.79(a)(31) requires that the FSAR include the following information:

For nuclear power plants to be operated on multi-unit sites, an evaluation of the potential hazards to the structures, systems, and components important to safety of operating units resulting from construction activities, as well as a description of the managerial and administrative controls to be used to provide assurance that the limiting conditions for operation are not exceeded as a result of construction activities at the multi-unit sites.

Accordingly, the evaluation of the potential impact of the construction of Fermi 3 on Fermi 2 structures, systems, and components (SSCs) important to safety is summarized below, along with a description of the managerial and administrative controls used to provide assurance that Fermi 2 limiting conditions for operation (LCOs) are not exceeded as a result of Fermi 3 construction activities. This evaluation involves several sequential steps:

- Identification of potential construction activity hazards
- Identification of SSCs important to safety
- Identification of LCOs
- Identification of impacted SSCs and LCOs
- Identification of applicable managerial and administrative controls

1.12.2 Potential Construction Activity Hazards

Fermi 3 is located on the existing Fermi site on a parcel of land adjacent to and generally southwest of the operating unit, Fermi 2, as shown in [Figure 2.1-204](#).

Based on experience from similar projects, the scope of work necessary to construct Fermi 3 is well understood. In general, it includes, but is not necessarily limited to, activities such as site exploration, grading, clearing and installation of drainage and erosion control measures; boring, drilling, dredging, demolition and excavating; storage and warehousing of equipment; and construction, erection and fabrication of new facilities. These activities involve major ESBWR standard plant structures such as the Reactor Building, Control Building, Fuel Building, Turbine Building,

Radioactive Waste Building and Electrical Building; as well as related support facilities such as transformers, switchyard(s), transmission lines, cooling water structures and systems, water treatment facilities, storage tanks, etc.

The applicable time period for such activities starts when work is first performed under the COL for Fermi 3 and ends for each Fermi 3 SSC when responsibility for that SSC is transferred to the accountable operating organization.

Each of the types of construction activities necessary to build a new unit was examined to identify the potential hazards to the existing units. The resulting list of construction activities and potential hazards is shown in [Table 1.12-201](#).

1.12.3 Structures, Systems and Components Important to Safety

Consistent with 10 CFR 50.34 and 10 CFR 50, Appendix A, Fermi 2 SSCs important to safety were identified in Chapter 3 of the Fermi 2 Updated Final Safety Analysis Report (UFSAR) ([Reference 1.12-201](#)); additionally, information in Chapters 4, 5, 6, 7, 8 and 9 of the Fermi 2 UFSAR was utilized.

1.12.4 Limiting Conditions for Operation

Pursuant to 10 CFR 50.36, LCOs are the lowest functional capability or performance levels of equipment required for safe operation of a facility and are established in operating unit technical specifications for each item meeting one or more of the following criteria:

- Criterion 1 – Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary.
- Criterion 2 – A process variable, design feature, or operating restriction that is an initial condition of a design basis accident (DBA) or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
- Criterion 3 – A SSC that is part of the primary success path and which functions or actuates to mitigate a DBA or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
- Criterion 4 – A SSC which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

The applicable LCOs are found in the Fermi 2 Technical Specifications ([Reference 1.12-202](#)).

1.12.5 **Impacted Structures, Systems and Components and Limiting Conditions for Operation**

The information described in [Subsection 1.12.2](#) through [Subsection 1.12.4](#) was evaluated to identify Fermi 2 SSCs and LCOs that might be impacted by Fermi 3 construction activities. For example, internal/in-plant Fermi 2 LCO parameters such as “Control Rod OPERABILITY,” “Shutdown Margin,” and “RCS Specific Activity” were eliminated by examination. Similarly, SSCs both internal and specific to Fermi 2 are not affected. These include items such as the Hydraulic Control Units, Fuel Storage Racks and Control Rod Drive Assemblies.

For each of the potential hazards listed in [Table 1.12-201](#), [Table 1.12-202](#) presents the potential consequences to the SSCs of the existing unit that were identified in the above process.

1.12.6 **Managerial and Administrative Controls**

[START COM 1.12-001] Managerial and administrative controls are utilized to identify preventive and mitigative measures and provide notification of hazardous activity initiation in order to prevent or minimize exposure of SSCs to the identified hazards. Applicable managerial and administrative controls are listed in [Table 1.12-203](#). **[END COM 1.12-001]**

Specific hazards, impacted SSCs, and managerial and administrative controls will be developed and implemented as work progresses on site. For example, prior to construction activities that involve the use of large construction equipment such as cranes, managerial and administrative controls will be in place to prevent adverse impacts on Fermi 2 overhead power lines, switchyard, security boundary, etc., by providing the necessary restrictions on the use of large construction equipment.

1.12.7 **References**

- 1.12-201 Enrico Fermi Unit 2, Updated Final Safety Analysis Report, Revision 14, November 2006.
- 1.12-202 Enrico Fermi Unit 2 Technical Specifications.

Table 1.12-201 Potential Hazards to Fermi 2 from Fermi 3 Construction Activities
(Sheet 1 of 2) [EF3 SUP 1.12-1]

| Construction Activity | Potential Hazards |
|--|--|
| Site Exploration, Grading, Clearing, Installation of Drainage and Erosion Control Measures, etc. | Impact on Overhead Power Lines |
| | Impact on Transmission Towers |
| | Impact on Underground Conduits, Piping, Tunnels, etc. |
| | Impact on Site Access and Egress |
| | Impact on Drainage Facilities and Structures |
| | Impact on Onsite Transportation Routes |
| | Impact on Slope Stability |
| | Impact of Increased Soil Erosion and Local Flooding |
| | Impact of Construction-Generated Dust and Equipment Exhausts |
| | Impact of Encroachment on Plant Protected or Vital Areas |
| Boring, Drilling, Pile Driving, Dredging, Demolition, Excavation, etc. | Impact of Encroachment on Structures and Facilities |
| | Impact on Underground Conduits, Piping, Tunnels, etc. |
| | Impact on Foundation Integrity |
| | Impact on Structural Integrity |
| | Impact on Slope Stability |
| | Impact of Ground Vibration |
| Equipment Movement, Material Delivery, Vehicle Traffic, etc. | Impact of Overpressure from Use of Explosives |
| | Impact on Overhead Power Lines |
| | Impact on Transmission Towers |
| | Impact on Underground Conduits, Piping, Tunnels, etc. |
| | Impact of Crane Load Drops |
| | Impact of Crane or Crane Boom Failures |
| | Impact of Vehicle Accidents |
| Equipment And Material Laydown, Storage, Warehousing, etc. | Impact of Vehicle Runaways |
| | Impact of Releases of Stored Flammable, Hazardous or Toxic Materials |
| | Impact of Increase Local Flooding |
| | Impact of Wind-Generated, Construction-Related Debris and Missiles |

Table 1.12-201 Potential Hazards to Fermi 2 from Fermi 3 Construction Activities
(Sheet 2 of 2) [EF3 SUP 1.12-1]

| Construction Activity | Potential Hazards |
|---|--|
| General Construction, Erection, Fabrication, etc. | Impact on Instrumentation and Control Systems and Components |
| | Impact on Electrical Systems and Components |
| | Impact on Cooling Water Systems and Components |
| | Impact on Radioactive Waste Release Points and Parameters |
| | Impact of Abandonment of SSCs |
| | Impact of Relocation of SSCs |
| Connection, Integration, Tie-In, Testing, etc. | Impact on Instrumentation and Control Systems and Components |
| | Impact on Electrical and Power Systems and Components |
| | Impact on Cooling Water Systems and Components |
| General Site Construction Activities | Impact on Site Security Systems |

Table 1.12-202 Potential Consequences to Fermi 2 Due to Potential Hazards Resulting from Fermi 3 Construction Activities) (Sheet 1 of 3)
[EF3 SUP 1.12-1]

| Potential Hazard | Potential Consequences |
|--|--|
| Primary Containment | |
| Impact of Crane or Crane Boom Failures | Building Degradation Due to Crane Boom Failure |
| Impact of Wind-Generated Construction-Related Debris and Missiles | Effects of Construction-Related Debris or Missiles |
| Impact of Overpressure from Use of Explosives | Building Degradation Due to Structural Damage as a Result of Explosion |
| Reactor Auxiliary Building | |
| Impact of Crane or Crane Boom Failures | Building Degradation Due to Crane Boom Failure |
| Impact of Wind-Generated Construction-Related Debris and Missiles | Effects of Construction-Related Debris or Missiles |
| Impact of Overpressure from Use of Explosives | Building Degradation Due to Structural Damage as a Result of Explosion |
| Control Center Complex (Including Cable Spreading Room) | |
| Impact of Crane or Crane Boom Failures | Building Degradation Due to Crane Boom Failure |
| Impact of Wind-Generated Construction-Related Debris and Missiles | Effects of Construction-Related Debris or Missiles |
| Impact of Overpressure from Use of Explosives | Building Degradation Due to Structural Damage as a Result of Explosion |
| Impact of vibratory ground motion | Operational disruptions due to vibration induced spurious trips |
| Control Room Emergency Filtration and Control Center Air Conditioning Systems | |
| Impact of Construction-Generated Dust and Equipment Exhausts | Effects of Construction-Generated Dust and Equipment Exhausts on Control Room Habitability Systems Air Intakes |
| Impact of Releases of Flammable, Hazardous or Toxic Materials | Effects of Releases of Flammable, Hazardous or Toxic Materials on Control Room Habitability Systems Design Basis |
| Diesel Generators | |
| Impact of Construction-Generated Dust and Equipment Exhausts | Effects of Construction-Generated Dust and Equipment Exhausts on Emergency Diesel Generator Combustion Air Intakes |
| Fire Protection System | |
| Impact on Underground Conduits, Piping, Tunnels, etc. | Degradation of FPS Availability or Capacity |
| Impact of the Relocation of SSCs | Degradation of FPS Availability or Capacity |

Table 1.12-202 Potential Consequences to Fermi 2 Due to Potential Hazards Resulting from Fermi 3 Construction Activities) (Sheet 2 of 3)
[EF3 SUP 1.12-1]

| Potential Hazard | Potential Consequences |
|---|---|
| Gaseous Radioactive Waste Management System | |
| Impact on Radioactive Waste Release Points and Parameters | Building and Facility Effects on Gaseous Release X/Q and D/Q Assumptions |
| Turbine Building | |
| Impact of vibratory ground motion | Operational disruptions due to vibration induced spurious trips |
| Impact of Crane or Crane Boom Failures | Building Degradation Due to Crane Boom Failure |
| Impact of Wind-Generated Construction-Related Debris and Missiles | Effects of Construction-Related Debris or Missiles |
| Impact of Overpressure from Use of Explosives | Building Degradation Due to Structural Damage as a Result of Explosion |
| Offsite Power System | |
| Impact on overhead power lines | Transmission line disruptions due to grading or clearing, equipment movement, crane boom failures, etc. |
| Impact on transmission towers | Transmission line disruptions due to grading or clearing, equipment movement, crane boom failures, etc. |
| Impact of vibratory ground motion | Operability disruptions due to vibration induced spurious trips |
| Impact on electrical systems and components | Operability disruptions due to equipment movement, system interconnections, etc. |
| Onsite Power Systems | |
| Impact of vibratory ground motion | Operability disruptions due to vibration induced spurious trips |
| Impact on electrical systems and components | Operability disruptions due to vibration induced spurious trips, system interconnections, etc. |
| General Service Water Intake Building | |
| Impact of crane or crane boom failures | Building degradation due to crane boom failure |
| Impact of wind-generated construction-related debris and missiles | Construction-related debris or missile |
| Residual Heat Removal Complex | |
| Impact of Crane or Crane Boom Failures | Building Degradation Due to Crane Boom Failure |
| Impact of Wind-Generated Construction-Related Debris and Missiles | Effects of Construction-Related Debris or Missiles |
| Impact of Overpressure from Use of Explosives | Building Degradation Due to Structural Damage as a Result of Explosion |

**Table 1.12-202 Potential Consequences to Fermi 2 Due to Potential Hazards
Resulting from Fermi 3 Construction Activities) (Sheet 3 of 3)**
[EF3 SUP 1.12-1]

| Potential Hazard | Potential Consequences |
|---|---|
| Impact on underground conduits, piping, tunnels, etc. | Degradation of RHR System and Emergency Diesel Generator availability or capacity |
| Impact on cooling water systems and structures | Degradation of RHR System and Emergency Diesel Generator availability or capacity |

Table 1.12-203 Managerial and Administrative Controls for Fermi 3 Construction
Activity Hazards (Sheet 1 of 2) [EF3 SUP 1.12-1]

| Hazard | Control |
|--|--|
| Impact on overhead power lines | Administrative controls for appropriate standoff and/or installation of temporary support towers |
| Impact on transmission towers | Administrative controls for appropriate standoff and/or installation of temporary support towers |
| Impact on underground conduits, piping, tunnels, etc. | Administrative controls to identify potentially affected SSCs; evaluation to ensure structural integrity during construction; and/or temporary measures to mitigate impacts |
| Impact of construction-generated dust and equipment exhausts | Administrative controls to avoid or minimize construction dust (for example, use of water spray trucks) and/or enhanced monitoring of potentially affected system intakes, filters, etc. |
| Impact of overpressure from use of explosives | Administrative controls to coordinate transport, storage and use of explosives and/or temporary measures to mitigate impacts |
| Impact of vehicle accidents | Administrative controls to respond to site accidents (for example, construction fire brigade and/or hazardous materials response team) |
| Impact of ground vibration | Administrative controls to identify potentially affected SSCs, and/or temporary measures to mitigate impacts |
| Impact of crane or crane boom failures | Administrative controls for appropriate standoff and/or load limits (for example, minimum standoff distances and/or load limitations) |
| Impact of releases of flammable, hazardous or toxic materials | Administrative controls on quantities and types of flammable, hazardous or toxic materials |
| Impact of wind-generated, construction-related debris and missiles | Administrative controls on equipment and material storage and transport, and for reducing power or shutting down Fermi 2 during high winds or high wind warnings |
| Impact on electrical systems and components | Administrative controls to identify potentially affected SSCs; evaluation to ensure system and component integrity during construction; and/or temporary measures to mitigate impacts |
| Impact on cooling water systems and components | Administrative controls to identify potentially affected SSCs; evaluation to ensure system and component integrity during construction; and/or temporary measures to mitigate impacts |
| Impact on radioactive waste release points and parameters | Enhanced monitoring and control to ensure releases are within limits |

Table 1.12-203 Managerial and Administrative Controls for Fermi 3 Construction Activity Hazards (Sheet 2 of 2) [EF3 SUP 1.12-1]

| Hazard | Control |
|---------------------------------|---|
| Impact of relocation of SSCs | Administrative controls to identify potentially affected SSCs effects of releases of flammable, hazardous or toxic materials on control room habitability systems design basis evaluation to ensure system and component integrity during construction; and/or temporary measures to mitigate impacts |
| Impact on site security systems | Administrative controls to coordinate construction activities with Fermi 2 physical protection personnel and procedures |

Appendix 1A Response to TMI Related Matters

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

Table 1A-1, 10 CFR 50.34(f)(3)(i), TMI Item I.C.5

Add the following to the end of the ESBWR Resolution statement:

STD SUP 1A.1-1

ESBWR construction and operations engineers are also continually involved in reviewing industry experience from these same sources in accordance with the administrative procedures described in DCD Section 18.3.2.

Table 1A-1, 10 CFR 50.34(f)(3)(iii), TMI Item I.F.2

Add the following to the end of the ESBWR Resolution statement:

STD SUP 1A.1-1

The Quality Assurance Program described in [Chapter 17](#) also meets the requirements of issue I.F.2 as they apply to the construction and operation of the ESBWR.

Table 1A-1, 10 CFR 50.34(f)(3)(vii), TMI Item II.J.3.1

Add “13.1” as an “Associated Location(s)” and add the following to the end of the ESBWR Resolution statement:

STD SUP 1A.1-1

The ESBWR construction and operations teams have also developed a management plan for the ESBWR project that consists of a properly structured organization with open lines of communication, clearly defined responsibilities, well-coordinated technical efforts, and appropriate control channels.

The organizational structure is discussed in [Section 13.1](#).

Appendix 1B Plant Shielding to Provide Access to Areas and Protect Safety Equipment for Post-Accident Operation [II.B.2]

This section of the referenced DCD is incorporated by reference with no departures or supplements.

Appendix 1C Industry Operating Experience

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

Appendix 1C.1 Evaluation

Replace the last paragraph with the following.

**STD COL 1C.1-1-A
STD COL 1C.1-2-A
STD SUP 1C-1**

DCD Tables 1C-1 and 1C-2 are supplemented by [Tables 1C-201](#) and [1C-202](#). These tables address Generic Letters and Bulletins that have been in effect/issued up to six months before the COL application submittal date, and after the SRP revisions that are applicable to this FSAR. They also address Generic Letter 82-39 and IE Bulletin 2005-02, which were identified in the DCD as the responsibility of the COL applicant.

Appendix 1C.2COL Information

STD COL 1C.1-1-A

1C.1-1-A Handling of Safeguards Information

This COL item is addressed in Section 1C.1 and the [Table 1C-201](#) entry for Generic Letter 82-39.

STD COL 1C.1-2-A

1C.1-2-A Emergency Preparedness and Response Actions

This COL item is addressed in Section 1C.1 and the [Table 1C-202](#) entry for IE Bulletin 2005-02.

Table 1C-201 Operating Experience Review Results Summary—Generic Letters

| No. | Issue Date | Title | Evaluation Result or Location(s) Where Discussed |
|--------------------|------------|--|--|
| [STD COL 1C.1-1-A] | | | |
| 82-39 | 12/22/82 | Problems with the Submittals of 10 CFR 73.21 Safeguards Information Licensing Review | Not Applicable. Is an administrative communication. The site has an approved procedure for handling Safeguards Information including how to mail such information to authorized recipients. |
| [EF53 DEP11.4-1] | | | |
| 81-38 | 11/10/81 | Storage of Low-Level Radioactive Wastes at Power Reactor Sites | The Radwaste Building includes space for processing and storage of low level waste. Storage space is provided for at least 10 years of packaged Class B and C waste and approximately 3 months worth of packaged Class A waste. Section 11.4 |

Table 1C-202 Operating Experience Review Results Summary—IE Bulletins
[STD COL 1C.1-2-A]

| No. | Issue Date | Title | Evaluation Result or Location(s) Where Discussed |
|---------|------------|---|--|
| 2005-02 | 07/18/05 | Emergency Preparedness and Response Actions for Security-Based Events | COLA Part 5, Emergency Plan |