UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 4, 2011

Mr. Michael J. Pacilio
President and Chief Nuclear Officer
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: THREE MILE ISLAND NUCLEAR STATION, UNIT 1 – RE-ISSUANCE LETTER REGARDING AMENDMENT NO. 273 RE: TECHNICAL SPECIFICATION CHANGES TO REFLECT CONTROL ROD DRIVE CONTROL SYSTEM UPGRADE (TAC NO. MD9762)

Dear Mr. Pacilio:

On May 27, 2010, the Nuclear Regulatory Commission (NRC or Commission) issued Amendment No. 273 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML092740791) to Three Mile Island Nuclear Station, Unit 1 (TMI-1) Renewed Facility Operating License No. DPR-50 in response to your application dated September 29, 2008, (ADAMS Accession No. ML082800174). Amendment No. 273 consisted of changes to the Technical Specifications (TS) including changes to pages 4-3 and 4-5.

On January 12, 2011, the Commission issued Amendment No. 274 (ADAMS Accession No. ML103120069) to TMI-1 Renewed Facility Operating License No. DPR-50 in response to your application dated March 24, 2010, (ADAMS Accession No. ML100840205). Amendment No. 274 also included changes to TS pages 4-3 and 4-5.

Amendment No. 273 was effective as of May 27, 2010 and was to be implemented prior to exceeding cold shutdown following the fall 2011 (T1R19) refueling outage. Amendment No. 274 was issued on January 12, 2011, to be implemented within 120 days. Because Amendment No. 274 was planned for implementation prior to Amendment No. 273, TS pages 4-3 and 4-5, issued with Amendment No. 274, did not reflect the changes that were made in Amendment No. 273. As was indicated in NRC letter dated January 12, 2011, this follow-up letter provides TS pages 4-3 and 4-5 that reflect the changes of both amendments such that the TS will be in the proper configuration upon implementation of Amendment No. 273.

Therefore, enclosed are TS pages 4-3 and 4-5 reflecting the changes made by both Amendment Nos. 273 and 274. Please utilize these pages, upon implementation of Amendment 273, in lieu of the versions of pages 4-3 and 4-5 that were originally issued to you by letter dated May 27, 2010.

If you have any questions, please contact me at 301-415-2833.

Sincerely,

Peter Bamford

Peter J. Bamford, Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-289

Enclosure:
As stated

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TABLE 4.1-1

INSTRUMENT SURVEILLANCE REQUIREMENTS

CHANNEL DESCRIPTION	CHECK(c)	TEST(c)	CALIBRATE(c)	REMARKS
1. Protection Channel Coincidence Logic	NA		NA	
2. Control Rod Drive Trip Breaker	NA		NA	(1) Includes independent testing of shunt trip and undervoltage trip features.
3. Power Range Amplifier	(1)	NA	(2)	(1) When reactor power is greater than 15%. (2) When above 15% reactor power run a heat balance check at the frequency specified in the Surveillance Frequency Control Program. Heat balance calibration shall be performed whenever heat balance exceeds indicated neutron power by more than two percent.
4. Power Range Channel			(1)(2)	(1) When reactor power is greater than 60% verify imbalance using incore instrumentation. (2) When above 15% reactor power calculate axial offset upper and lower chambers after each startup if not done within the previous seven days.
5. Intermediate Range Channel	(1)		NA	(1) When in service.
6. Source Range Channel	(1)		NA	(1) When in service.
7. Reactor Coolant Temperature Channel				

TABLE 4.1-1 (Continued)

CHANNEL DESCRIPTION	CHECK(c)	TEST(c)	CALIBRATE(c)	REMARKS
19. Reactor Building Emergency Cooling and Isolation System Analog Channels				
a. Reactor Building 4 psig Channels	(1)	(1)		(1) When CONTAINMENT INTEGRITY is required.
b. RCS Pressure 1600 psig	(1)	(1)	NA	(1) When RCS Pressure > 1800 psig.
c. Deleted				
d. Reactor Bldg. 30 psi pressure switches	(1)	(1)		(1) When CONTAINMENT INTEGRITY is required.
e. Reactor Bldg. Purge Line High Radiation (AH-V-1A/D)	(1)	(1)(2)		(1) When CONTAINMENT INTEGRITY is required.
f. Line Break Isolation Signal (ICCW & NSCCW)	(1)	(1)		(1) When CONTAINMENT INTEGRITY is required.
20. Reactor Building Spray System Logic Channel	NA		NA	
21. Reactor Building Spray 30 psig pressure switches	NA			
22. Pressurizer Temperature Channels		NA		
23. Control Rod Absolute Position	(1)	NA		(1) Check with Relative Position Indication
a. Zone Reference Switch	NA	R(1)	NA	(1) Verify switch functions
24. Control Rod Relative Position	(1)	NA		(1) Check with Absolute Position Indication
25. Core Flooding Tanks				
a. Pressure Channels Coolant	NA	NA		
b. Level Channels	NA	NA		
26. Pressurizer Level Channels		NA		

If you have any questions, please contact me at 301-415-2833.

Sincerely,

/ra/

Peter J. Bamford, Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
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