



10CFR50.59

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LR-N11-0037

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, DC 20555-0001

Hope Creek Generating Station  
Facility Operating License No. NPF-57  
NRC Docket No. 50-354

Subject: Reports of Changes, Tests and Experiments

Pursuant to the requirements of 10CFR50.59, "Changes, test and experiments", paragraph (d)(2), Hope Creek Generating Station (HCGS) is providing the required report for Facility Operating License No. NPF-57. This report provides a summary of changes, tests and experiments implemented at HCGS during the period of February 25, 2009 through January 17, 2011.

There are no new or revised regulatory commitments contained in this letter.

If you have any questions, please contact Mr. Phil Duca, at 856-339-1640.

Sincerely,

A handwritten signature in black ink, appearing to read "M. Gaffney".

Michael G. Gaffney  
Regulatory Assurance Manager – Hope Creek  
PSEG Nuclear LLC

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Attachment: 1) Hope Creek Summary of Changes, Test and Experiments

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Hope Creek Generating Station  
Facility Operating License NPF-57  
Docket No. 50-354  
Summary Report of Changes, Tests and Experiments

Replacement of HPCI Valve HV-8278

This change replaced a flexible wedge gate valve BJ-HV-8278 with a parallel slide gate valve using a larger motor operator. The change also restored a previously removed automatic closure signal to this valve when the HPCI turbine is tripped. This parallel slide gate valve is not subject to thermal binding as was the flexible wedge gate valve. The thermal binding potential led to the previous removal of the automatic closure signal.

Update of OPRM Set point computer code

An element in the development of the cycle specific Oscillation Power Range Monitors (OPRM) set points is the DIVOM (Delta CPR over Initial MCPR versus Oscillation Magnitude) analysis provided by GE (Global Nuclear Fuels or GE-Hitachi Nuclear Energy). GE uses the TRACG04 computer code to perform the DIVOM analysis. Recently GE updated TRACG04 to TRACG04P. Use of this different code required a safety evaluation.

Revision of the Leakage Reduction Program Calculation

Primary containment isolation valve (PCIV) isolation times were increased to minimize potential damage and resultant leakage rates and valve reliability for specific PCIVs. To accomplish this, the following activities were performed:

- Increased the allowable Emergency Safeguards Features (ESF) leak rate to 2.85 gpm.
- Updated the Main Steam Isolation Valve (MSIV) leakage release model.
- Increased the PCIV isolation time to 120 seconds.
- Revision of the postulated post accident dose to the control room, vital areas and offsite.

The PCIV change was never implemented in station procedures (see "Update of Dose Analysis Calculation" below).

Update of Dose Analysis Calculation

During reviews of the Cobalt-60 Test Assembly license amendment request (LAR) H09-01 requests for additional information (RAIs) were issued. To resolve these RAIs the increase in allowable PCIV closure time, and some of the previous changes to the MSIV leakage modeling contained in the previous revision of the dose analysis calculation were removed. Consequently the dose analysis calculations were revised.