

PMSTPCOL NPEmails

From: Eudy, Michael
Sent: Monday, February 07, 2011 3:42 PM
To: Chappell, Coley; Bill Mookhoek; Scott Head
Cc: STPCOL; Wunder, George; Abeywickrama, Bernadette; Anand, Raj; Tonacci, Mark
Subject: STP Chapter 11 Advanced Phase 4 SER (NON PUBLIC)
Attachments: ML110340256(trans letter).pdf; ML110340320(non public).pdf

Importance: High

Coley et al,

In preparation for the upcoming 2011 ACRS ABWR subcommittee meetings in support of the South Texas Project Units 3 & 4 COLA, I have attached the Phase 4 SER/OIs for Chapter 11 along with a transmittal letter. Please note that the enclosed SER and letter are electronic courtesy copies provided in order for STPNOC to begin a proprietary review. Hard copies are also being mailed to Mr. Scott Head. We are requesting for STPNOC to perform their proprietary review within 10 days from the date of the above letter. Until a determination that the enclosures do not contain proprietary information, these documents should not be released to the public. In addition, both documents are password protected at this time.

In addition to Chapter 11, NRC staff plans to present Chapter 12, 13 and 16 at the March 9, 2011 ACRS ABWR subcommittee meeting. Proposed meeting agenda and other details will be provided at a later date. Please let me know if you have any questions.

Michael A. Eudy - Project Manager
U.S. Nuclear Regulatory Commission
NRO/DNRL/NGE1&2
301-415-3104

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From: Eudy, Michael

Created By: Michael.Eudy@nrc.gov

Recipients:

"STPCOL" <STP.COL@nrc.gov>
Tracking Status: None
"Wunder, George" <George.Wunder@nrc.gov>
Tracking Status: None
"Abeywickrama, Bernadette" <Bernadette.Abeywickrama@nrc.gov>
Tracking Status: None
"Anand, Raj" <Raj.Anand@nrc.gov>
Tracking Status: None
"Tonacci, Mark" <Mark.Tonacci@nrc.gov>
Tracking Status: None
"Chappell, Coley" <ccchappell@STPEGS.COM>
Tracking Status: None
"Bill Mookhoek" <wemookhoek@stpegs.com>
Tracking Status: None
"Scott Head" <smhead@stpegs.com>
Tracking Status: None

Post Office: HQCLSTR01.nrc.gov

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February 3, 2011

Mr. Scott Head, Manager
Regulatory Affairs
Nuclear Innovation North America LLC
4000 Avenue F
Bay City, TX 77414

SUBJECT: SOUTH TEXAS PROJECT UNITS 3 AND 4 COMBINED LICENSE
APPLICATION – ADVANCED SAFETY EVALUATION FOR CHAPTER 11,
“RADIOACTIVE WASTE MANAGEMENT”

Dear Mr. Head:

The U.S. Nuclear Regulatory Commission (NRC) staff is preparing an advanced safety evaluation (SE) for each chapter of the South Texas Project (STP) Units 3 and 4 Combined License Application (COLA) submitted by STP Nuclear Operating Company (STPNOC) on September 20, 2007.

The staff's advanced SE for Chapter 11, Radioactive Waste Management, is being provided to the Advisory Committee on Reactor Safeguards (ACRS) Subcommittee to support the upcoming meeting of the ACRS Subcommittee, scheduled to be held on March 9, 2011.

The enclosed advanced SE is being provided only to STPNOC for review of proprietary information pursuant to Title 10 of the *Code of Federal Regulations* (10 CFR) 2.390 “Public Inspections, Exemptions, Requests for Withholding.” The staff concludes that the enclosed SE does not contain any information for which exemption from public disclosure has been sought or approved or any factual errors. However, the NRC will withhold the enclosed SE from public disclosure for ten business days from the date of this letter to allow you the opportunity to verify the staff's conclusion that the SE contains no such exempt information or any factual errors. If within that ten day period, you do not request that all or portions of the SE be withheld from public disclosure, the enclosure will be made available for public inspection through the NRC Public Document Room and the Publicly Available Records component of NRC's Agencywide Documents Access and Management System (ADAMS) and placed on the NRC's public web page for this application.

**Document transmitted herewith
contains sensitive unclassified
information. When separated from the
enclosure, this document is
“DECONTROLLED.”**

S. Head

-2-

If you have any questions or comments concerning this matter, I can be reached at 301-415-1494 or via e-mail address at George.Wunder@nrc.gov.

Sincerely,

/RA/

George F. Wunder, Lead Project Manager
BWR Projects Branch
Division of New Reactor Licensing
Office of New Reactors

Docket Nos.: 52-012
52-013

Enclosure:
As stated

cc: w/o encl. see next page

S. Head

-2-

If you have any questions or comments concerning this matter, I can be reached at 301-415-1494 or via e-mail address at George.Wunder@nrc.gov.

Sincerely,

/RA/

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BWR Projects Branch
Division of New Reactor Licensing
Office of New Reactors

Docket Nos.: 52-012
52-013

Enclosure:
As stated

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GWunder, NRO

MTonacci, NRO

RAnand, NRO

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NRO-002

OFFICE	DNRL/BWR:PM	DNRL/BWR:LA	DNRL/BWR:PM (Lead)
NAME	RAnand (for MEudy)	BAbeywickrama (e-mail)	GWunder
DATE	2/3/11	2/3/11	2/3/11

OFFICIAL RECORD COPY

cc:

Certrec Corporation
4200 South Hulen, Suite 422
Fort Worth, TX 76109

Mr. Brian Almon
Public Utility Commission
William B. Travis Building
PO Box 13326
1701 North Congress Avenue
Austin, TX 78701-3326

Ms. Michele Boyd
Legislative Director
Energy Program
Public Citizens Critical Mass Energy
and Environmental Program
215 Pennsylvania Avenue, SE
Washington, DC 20003

Mr. Ed Halpin
President & CEO
STP Nuclear Operating Company
P.O. Box 289
Wadsworth, TX 77483

Mr. Scott Head
Manager, Regulatory Affairs
STP Units 3 & 4
Nuclear Innovation North America, LLC
4000 Avenue F
Bay City, TX 76109

Ms. Susan M. Jablonski
Office of Permitting, Remediation
and Registration
Texas Comm. on Env. Quality
MC-122
P.O. Box 13087
Austin, TX 78711-3087

Judge
Matagorda County
Matagorda County Courthouse
1700 Seventh Street
Bay City, TX 77414

M. A. McBurnett
Senior VP, Oversight & Regulatory Affairs
Nuclear Innovation North America, LLC
4000 Avenue F
Bay City, TX 77414

Bill Mookhoek
Licensing Supervisor
STP Units 3 and 4
4000 Avenue F
Bay City, TX 77414

Mr. Terry Parks
Chief Inspector
Texas Department of Licensing
and Regulation
Boiler Division
P.O. Box 12157
Austin, TX 78711

Kathy C. Perkins, RN, MBA
Assistant Commissioner
Division for Regulatory Services
Texas Department of State Health Services
P.O. Box 149347
Austin, Texas 78714-9347

Policy Director
Environmental and Natural Resources
P. O. Box 12428
Austin, TX 78711-3189

Mr. Frank M. Quinn
8 Oak Avenue
Gaithersburg, MD 20877-2705

COL - STP Nuclear - South Texas Mailing List

Regional Administrator
Region IV
U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive
Suite 400
Arlington, TX 76011-8064

Alice Hamilton Rogers, PE
Inspections Unit Manager
Texas Department of State Health Services
P.O. Box 149347
Austin, Texas 78714-9347

Ms. Jamey Seeley
General Counsel and Secretary
Nuclear Innovation North America, LLC
521 Fifth Avenue, 30th Floor
New York, NY 10175

Mr. Robert E. Sweeney
IBEX ESI
4641 Montgomery Avenue
Suite 350
Bethesda, MD 20814

Mr. Steve Winn
President and CEO
Nuclear Innovation North America LLC
521 Fifth Avenue, 30th Floor
New York, NY 10175

COL - STP Nuclear - South Texas Mailing List

Email

APH@NEI.org (Adrian Heymer)
awc@nei.org (Anne W. Cottingham)
BrinkmCB@westinghouse.com (Charles Brinkman)
chris.maslak@ge.com (Chris Maslak)
cindyrae 52@yahoo.com
cwaltman@roe.com (C. Waltman)
david.lewis@pillsburylaw.com (David Lewis)
Derlinda.Bailey@chguernsey.com (Derinda Bailey)
donald.woodlan@luminant.com (Donald Woodlan)
dwstillwell@stpegs.com (Bill Stillwell)
dwuokko@tane.toshiba.com (Dale Wuokko)
ed.burns@earthlink.net (Ed Burns)
eliza.seedcoalition@gmail.com (Elza Brown)
erg-xl@cox.net (Eddie R. Grant)
gzinke@entergy.com (George Alan Zinke)
jerald.head@ge.com (Jerald G. Head)
jim.riccio@wdc.greenpeace.org (James Riccio)
Joseph Hegner@dom.com (Joseph Hegner)
junichi uchiyama@mnes-us.com (Junichi Uchiyama)
karen@seedcoalition.org (Karen Hadden)
kdrichards@stpegs.com (Kevin Richards)
KSutton@morganlewis.com (Kathryn M. Sutton)
kwaugh@impact-net.org (Kenneth O. Waugh)
lchandler@morganlewis.com (Lawrence J. Chandler)
ldblaylock@cpsenergy.com (Larry Blaylock)
louis.eichenberger@ge.com (Louis Eichenberger)
mamcburnett@stpegs.com (Mark McBurnett)
Marc.Brooks@dhs.gov (Marc Brooks)
maria.webb@pillsburylaw.com (Maria Webb)
mark.beaumont@wsms.com (Mark Beaumont)
Mark.Crisp@chguernsey.com (Mark Crisp)
matias.travieso-diaz@pillsburylaw.com (Matias Travieso-Diaz)
maurerbf@westinghouse.com (Brad Maurer)
media@nei.org (Scott Peterson)
MSF@nei.org (Marvin Fertel)
nirsnet@nirs.org (Michael Mariotte)
Nuclaw@mindspring.com (Robert Temple)
patriciaL.campbell@ge.com (Patricia L. Campbell)
Paul@beyondnuclear.org (Paul Gunter)
pbessette@morganlewis.com (Paul Bessette)
pshastings@duke-energy.com (Peter Hastings)
RJB@NEI.org (Russell Bell)
Rschrauder@tane.toshiba.com (Robert Schrauder)
sabinski@suddenlink.net (Steve A. Bennett)

COL - STP Nuclear - South Texas Mailing List

sandra.sloan@areva.com (Sandra Sloan)
sfrantz@morganlewis.com (Stephen P. Frantz)
smhead@stpegs.com (Scott Head)
stephan.moen@ge.com (Stephan Moen)
Tansel.Selekler@nuclear.energy.gov (Tansel Selekler)
tom.miller@hq.doe.gov (Tom Miller)
Vanessa.quinn@dhs.gov (Vanessa Quinn)
Wanda.K.Marshall@dom.com (Wanda K. Marshall)
wayne.marquino@ge.com (Wayne Marquino)
wemookhoek@stpegs.com (Bill Mookhoek)

11.0 RADIOACTIVE WASTE MANAGEMENT

11.1 Source Terms

This section addresses the sources of radioactivity that are generated within the core and that have the potential to leak into the reactor coolant system during normal operations, including anticipated operational occurrences (AOOs), as a result of defects in the fuel cladding. The radioactive source terms are used to identify the potential dose to members of the public and plant operators as a result of normal plant operation. This discussion includes consideration of parameters used to determine the (1) concentration of each isotope in the reactor coolant and steam, (2) fraction of fission product activity released into the reactor coolant, and (3) concentrations of all nonfission product radioactive isotopes in the reactor coolant.

Section 11.1 of the South Texas Project (STP) combined license (COL) Final Safety Analysis Report (FSAR) incorporates by reference (with one departure [STD DEP 10.1-3] and no supplements) Section 11.1, "Source Terms," of the certified Advanced Boiling-Water Reactor (ABWR) design control document (DCD) Revision 4, which is referenced in Title 10 of the Code of Federal Regulations (10 CFR) Part 52, Appendix A. U.S. Nuclear Regulatory Commission (NRC) staff reviewed the application and checked the referenced DCD to ensure that no issue relating to this section remains for review.¹ The staff's review confirmed that there is no outstanding issue related to this section and one departure that does not require NRC approval. Pursuant to 10 CFR 52.63(a)(5) and Part 52, Appendix A, Section VI.B.1, all nuclear safety issues relating to the source terms have been resolved.

Tier 2 Departure Not Requiring Prior NRC Approval

- STD DEP 10.3-1 Source Term Parameter

This departure increases the steam flow rate to 7.65E+06 from 7.63E+06 kilograms per hour (kg/hr) (an increase of less than 0.3 percent) due to the use of the new Toshiba turbine in place of the General Electric (GE) turbine. This increase will have a negligible effect on the source term used in the design of the radwaste management systems.

The applicant's evaluation in accordance with 10 CFR Part 52, Appendix A, Section VIII, Item B.5 characterizes the departure as not requiring prior NRC approval. With respect to the impact of this departure on this section, NRC staff found it reasonable that this departure does not require prior NRC approval. The applicant's process for evaluating departures and other changes to the DCD is subject to NRC inspection.

11.2 Liquid Waste Management System

11.2.1 Introduction

The liquid waste management system (LWMS) controls, collects, processes, handles, stores, and disposes of liquid radioactive waste generated as the result of normal operation, including AOOs. The LWMS reduces and controls radioactive releases into the environment. The LWMS

¹ See "Finality of Referenced NRC Approvals" in SER Section 1.1.3 for a discussion on the staff's review related to verification of the scope of information to be included in a COL application that references a design certification.

2. A radiation monitor in the discharge line that will automatically terminate liquid waste discharges from the low conductivity waste (LCW), high conductivity waste (HCW), or the detergent waste subsystem if radiation measurements exceed a predetermined level set by the COL applicant to meet 10 CFR Part 20, Sections 1001 through 2402, and Appendix B Table 2, Column 2 for the applicable subsystem shall be provided.
3. Specific administrative controls and liquid effluent source terms to limit the liquid wastes to 3,700 megabecquerels per year (MBq)/yr (excluding tritium) shall be provided.
4. Procedures for demonstration of compliance with 10 CFR Part 50, Appendix I, Sections II and III shall be provided.
5. Administrative controls to limit the instantaneous discharge concentrations of the radionuclides in liquid effluents to an unrestricted area to within 10 times the limits in 10 CFR Part 20, Appendix B, Table 2 Column 2, shall be provided.
6. Quality assurance (operations) provisions of the liquid radwaste systems shall be provided.

11.2.3 Regulatory Basis

The relevant requirements of the Commission regulations for the LWMS and associated acceptance criteria are in Section 11.2 of NUREG–0800, “Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants,” the Standard Review Plan (SRP). In particular, the regulatory basis for reviewing and accepting the information for the LWMS is in 10 CFR 20.1301(e), 10 CFR 20.1302, 10 CFR 20.1406, 10 CFR 50.34a; Appendix A to 10 CFR Part 50, General Design Criteria (GDC) 60 and 61; Appendix I to 10 CFR Part 50, Section IID; and 10 CFR 52.80(a). SRP acceptance criteria include codes and standards listed in Table 1 of RG 1.143; Regulatory Position C.1.1 of RG 1.143; RGs 1.109, 1.110, 1.112, 1.113, and 4.21; and Branch Technical Position (BTP) 11-6 of Section 11.2 to the SRP. RG 1.140 addresses the design considerations of ventilation systems described in FSAR Revision 2 Section 9.4, for building exhaust systems and venting tanks and vessels. SRP Section 11.2 (NUREG–0800) contains complete descriptions of the applicable regulatory and acceptance criteria.

In addition, the applicant identifies one Tier 2 departure in accordance with Section VIII, “Processes for Changes and Departures,” of “Appendix A to Part 52--Design Certification Rule for the U.S. Advanced Boiling Water Reactor.” Tier 2 departures are subject to the requirements in Section VIII.B.5 of 10 CFR Part 52, Appendix A, which are similar to the requirements in 10 CFR 50.59.

11.2.4 Technical Evaluation

As documented in NUREG–1503, NRC staff reviewed and approved Section 11.2 of the certified ABWR DCD. The staff reviewed Section 11.2 of the STP Units 3 and 4 COL FSAR and checked the referenced ABWR DCD to ensure that the combination of information in the COL FSAR and information in the ABWR DCD represents the complete scope of information relating

Evaluation of Plant Specific Information

Compliance with the Cost-Benefit Ratio in 10 CFR Part 50, Appendix I

To assess compliance with numerical guidance II.D in Appendix I to Part 50, NRC staff evaluated the methodology used to determine the cost-benefit ratio prescribed in RG 1.110. However, the staff was unable to determine whether the applicant conforms to the guidance prescribed in RG 1.110. The staff issued RAI 11.02-1. The applicant's response to this RAI dated April 23, 2009 (ML091170081), states that the required information is available for review. RAI 11.02-1 was closed as unresolved. The staff issued supplemental RAI 11.02-6 and requested the same information requested in RAI 11.02-1. The applicant's response to this RAI dated October 12, 2009 (ML092890082), provides all of the required information and calculations justifying the information in FSAR Section 11.2 concerning the LWMS cost-benefit analysis.

The results of the applicant's analysis show that the lowest cost option for liquid radwaste treatment system augment is a 10,000 gallon hold-up tank to the LWMS effluent stream before entry into the discharge canal. Assuming that this tank will eliminate all radioactivity from the liquid effluent, the resulting cost per person-rem dose reduction was \$3 for total body man-rem dose. This cost reduction is below the cost criterion of \$14,343 per person-rem for an augment in accordance with 10 CFR Part 50, Appendix I, Section II.D. The applicant therefore concluded that the LWMS meets as low as reasonably achievable (ALARA) requirements and requires no augments.

The staff performed an independent assessment using the staff's calculated population doses (see SER Section 12.2) and the guidance in RG 1.110 and arrived at the same conclusion. Because the staff's population doses differ from the applicant's doses, the staff obtained different but acceptable cost-benefit ratios for the addition of a 10,000 gallon hold-up tank to the LWMS effluent stream before entry into the discharge canal. SER Table 11.2-1 lists the cost-benefit costs calculated by the applicant and the staff for comparison. Accordingly, RAI 11.02-1 and RAI 11.02-6 are closed.

Table 11.2-1 Comparison of LWMS Cost-Benefit Ratios (Per Man-rem)

Organ/Body	Applicant Costs to Meet the Design Criteria *	Applicant Costs to Augment the LWMS	NRC Costs to Meet the Design Criteria Calculations *	NRC Costs to Augment the LWMS
Total Body	\$3	\$14,343	\$403	\$14,343
Thyroid	\$3	\$14,343	\$112	\$14,343

* Based upon liquid population doses in SER Section 12.2.

Compliance with 10 CFR 20.1406

FSAR Subsection 11.2.1.2.4 addresses general compliance with 10 CFR 20.1406 and RG 4.21, as they relate to facility design and operational procedures for permanently installed subsystems in minimizing the contamination of the facility and the generation of radioactive

waste. In addition, the DCD commits the COL applicant to follow the guidance of Inspection and Enforcement (IE) Bulletin 80-10 to (1) avoid the cross-contamination of nonradioactive systems, and (2) avoid unmonitored and uncontrolled radioactive releases into the environment.

The applicant also addresses compliance with the requirements of GDC 61 by meeting the guidelines of RG 1.143, as supported with additional commitments pertaining to design guidance in regard to natural phenomena hazards, internal and external man-induced hazards, and quality group classification and quality assurance provisions for radioactive waste management systems, structures and components, described in Section 11.2 of the FSAR, and confirming the installation of steel liners in cubicles where liquid radioactive waste tanks are located. These commitments also fulfill the requirements of 10 CFR 20.1406 and the guidance of RG 4.21, in minimizing the contamination of the facility and the generation of radioactive waste, and the requirements of IE Bulletin 80-10 in avoiding the cross-contamination of nonradioactive systems and unmonitored and uncontrolled radioactive releases into the environment. The staff found this approach acceptable.

Compliance with 10 CFR 20.1406 – Condensate Storage Tank

NRC staff reviewed various DCD Sections and FSAR Sections 9.2.4, 9.2.9, 11.2.1, 11.2.2, and 11.2.3 to determine the design, function, and relationship of the CST with the LWMS and 10 CFR 20.1406 compliance. The staff issued RAIs 11.02-7, 11.02-8, and 11.02-9 requesting the applicant to provide (1) the CST radioactive source terms and volumes; (2) the actual location of the CST and information that addresses 10 CFR 20.1406 concerns; and (3) the CST radioactive source term inventory addressing the concentration limits, the associated dose rates surrounding the tank, and data similar to all other tanks in the LWMS. The applicant responded to these RAIs in a letter dated October 29, 2009 (ML093640053). An additional letter dated December 30, 2009 (ML1000501183), replaced the previous response. RAIs 11.02-7, 11.02-8, and 11.02-9 were open items in the SER with open items.

The staff reviewed RAI 11.02-7 and requested additional data in supplemental RAI 12.02-18. RAI 11.02-7 is closed and will be addressed in the supplemental request for information in Section 12.2. RAI 11.02-8 is closed as the applicant provided changes to the information in Subsection 11.2.1.2.4 of the FSAR concerning (1) the CST location outdoors; (2) equipping the CST with alarms in the control room to monitor the liquid levels, all tank overflows, and drains and sample lines routed to the LWMS; and (3) providing a dike around the CST with a drain line routed to the LWMS from the surrounding dike. RAI 11.02-9 is closed because the applicant included the response to this RAI with the response to RAI 11.02-7. The staff's evaluation of the applicant's response to supplemental RAI 12.02-18 is in Section 12.2 of this SER.

Conformance with BTP 11-6 – Postulated Radioactive Releases Due To Liquid-Containing Tank Failures Review Responsibilities

NRC staff reviewed FSAR Subsection 15.7.3.1 in reference to BTP 11-6. The staff determined that the LWMS design includes the use of steel-lined compartments containing liquid radwastes up to a height capable of containing the release of all liquid radwastes into the LWMS compartment. Because of the design capabilities, the guidance in BTP 11-6 assumes that there will not be any major accident releasing liquid radwastes into the environment via the liquid pathway. The staff found that the consequences of an accidental failure of the LWMS would be mitigated by the design of the radwaste building. However, this analysis did not include an evaluation of the outdoor CST. The applicant addresses this evaluation in the RAI responses in

FSAR Sections 11.2 and 2.4.13 and in part of the response to RAI 12.02-18, which is evaluated in Section 12.2 of this SER.

Conformance with IE Bulletin 80-10

The guidance in IE Bulletin 80-10 includes information on the identification and restriction of noncontaminated systems that have the potential of becoming contaminated. In FSAR Subsection 11.2.1.2, "Design Criteria," the applicant commits to procedures ensuring that the guidance and information in IE Bulletin 80-10 are followed.

Subsection 11.2.1.2.4 addresses design requirements to (1) minimize contamination of the facility and the environment, (2) facilitate decommissioning, and (3) minimize the generation of radioactive waste in compliance with 10 CFR 20.1406. NRC staff found the applicant's commitment to use the guidance in IE Bulletin 80-10 to prevent the contamination of nonradioactive systems acceptable. Chapter 12 of this SER discusses compliance with 10 CFR 20.1406.

COL License Information Item

- COL License Information Item 11.1 Plant-Specific Liquid Radwaste Information

ABWR DCD Tier 2 Subsection 11.2.5.1, "COL License Information," states that the COL applicant shall provide several COL license information items that apply on a plant-specific basis. For the STP COL application, these COL license information items are addressed as follows:

The COL applicant shall provide the following items on a plant-specific basis:

1. DCD COL License Information Item 11.1, Item 1 states that the applicant shall provide "Compliance with Appendix I to 10 CFR 50 and the guidance given in ANSI N13.1, 'Guide to Sampling Airborne Radioactive Materials in Nuclear Facilities,' Regulatory Guide (RG) 1.21, 'Measuring and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants,' and RG 4.15, 'Quality Assurance for Radiological Monitoring Programs (Normal Operation)—Effluent Streams and the Environment.' "

In Section 11.2.5, the applicant commits to comply with the Appendix I to 10 CFR Part 50 and refers to DCD Subsection 12.2.2.4 to describe how this compliance will be accomplished for liquid and gaseous effluents. The NRC staff's review found that DCD Subsection 12.2.2.4 did not describe how compliance with Appendix I to 10 CFR Part 50 is accomplished. The staff also found that RG 4.15 is discussed in Section 11.2.5 of the FSAR but is not listed in Section 1.9 or in Section 1.9S as a regulatory guide utilized in this FSAR. The staff requested the applicant to appropriately incorporate RG 4.15 into the FSAR.

The staff issued RAI 11.02-10 requesting the applicant to describe how compliance with Appendix I to 10 CFR Part 50 will be accomplished and how the applicant will reference RG 4.15 in the FSAR.

The applicant's response to RAI 11.02-10 dated June 10, 2010 (ML101650103), states that the reference to RG 4.15, Revision 1 will be inserted into Section 1.9S in the next

revision of the FSAR as a utilized regulatory guide. The applicant also refers to revised Section 11.5.7S concerning the use of RG 4.15 for sampling, analyzing, and testing radioactive effluent samples.

The applicant also changed the numbering for FSAR Subsection 12.2.2.2 to Subsection 12.2.2.4. Revised Sections 11.5.7S and 11.2.5 now refer to this subsection for compliance with 10 CFR Part 50, Appendix I. The staff will review the next FSAR revision for all changes and pointers. RAI 11.02-10 is being tracked as **Confirmatory Item 11.02-10**.

2. DCD COL License Information Item 11.1, Item 2 states that “A radiation monitor in the discharge line automatically terminates liquid waste discharges from the sample tanks in the LCW, HCW or detergent waste subsystem if radiation measurements exceed a predetermined level set to meet 10 CFR 20, Sections 1001 – 2402 and Appendix B Table 2 Column 2 for the applicable subsystem is provided.”

In Section 11.5, the applicant states that a radiation monitor in the discharge line is required for plant operations to monitor liquid discharges from the radwaste system and to alarm and initiate an automatic closure of the waste discharge valve on the affected treatment system. The operation limits for this monitor are specified in the Offsite Dose Calculation Manual (ODCM) as required by RG 1.21, Revision 1.

The staff found this information acceptable because having a monitor with automatic isolation capabilities is an acceptable method for meeting the requirements in 10 CFR 50.34a and GDC 60 and 61. The monitoring system for process and effluent radiological monitoring and sampling systems are also addressed in Section 11.5 of the FSAR. Therefore, the applicant has adequately addressed COL License Information Item 11.1, Item 2.

3. DCD COL License Information Item 11.1, Item 3 states that “Specific administrative controls and liquid effluent source terms to limit the liquid wastes to 3700 MBq/yr (excluding tritium) shall be provided.”

In Section 11.2, the applicant states that the ODCM will provide specific administrative controls and liquid effluent source terms to limit the liquid wastes to 3,700 MBq/yr (excluding tritium). The ODCM will be implemented per the schedule in Table 13.4S-1.

The staff found this information acceptable because this COL license information item is addressed in Section 11.2 and in Table 13.4S-1. Therefore, the applicant has adequately addressed COL License Information Item 11.1, Item 3.

4. DCD COL License Information Item 11.1, Item 4 states that “Procedures for demonstration of compliance with 10 CFR Part 50, Appendix I, Sections II and III shall be provided.”

The applicant states that the Process and Effluent Monitoring and Sampling Program has specific procedures for complying with 10 CFR Part 50, Appendix I, Sections II and III. This program requires specific procedures for implementing the ODCM and the Radiological Environmental Monitoring Program (REMP). The requirements in these programs comply with 10 CFR Part 50, Appendix I and will be implemented per the schedule in Table 13.4S-1.

The staff found this information acceptable because this COL license information item is addressed in Section 11.5 and in Table 13.4S-1. Therefore, the applicant has adequately addressed COL License Information Item 11.1, Item 4.

5. DCD COL License Information Item 11.1, Item 5 states that there will be “Administrative controls to limit the instantaneous discharge concentrations of the radionuclides in liquid effluents to an unrestricted area to within 10 times the limits in 10 CFR Part 20, Appendix B, Table 2 Column 2.”

In Section 11.2, the applicant states that the ODCM has administrative controls to limit the instantaneous discharge concentrations of the radionuclides in liquid effluents to an unrestricted area to within 10 times the limits in 10 CFR Part 20, Appendix B, Table 2 Column 2, and will be implemented per the schedule in Table 13.4S-1.

The staff evaluated this information item in SER Sections 11.5 and 13.4.

6. DCD COL License Information Item 11.1, Item 6 states that “Quality assurance (operations) provisions of the liquid radwaste systems shall be provided.”

The applicant states that the nonsafety-related Structure, System, and Component (SSC) Quality Control Program for the LWMS is described in the STP Units 3 and 4 Quality Assurance Program Description in Section 17.5S.

The staff evaluated this information item in SER Section 17.5S.

Supplemental Information

Preoperational Tests

In COL FSAR Subsection 14.2.12.1.75, “Liquid and Solid Radwaste Systems Preoperational Tests,” and in FSAR Subsection 14.2.12.2.38, “Liquid Radioactive Waste Management System Performance,” the applicant states that the LWMS will be tested during the Preoperational Test Program. The LWMS equipment will be performance tested to demonstrate conformance with design process capabilities. The LWMS shall be capable of collecting, processing, and controlling the liquid wastes as designed such that the release of radioactive liquid effluents remain within the limits specified by the plant Technical Specifications or license conditions. NRC staff found this testing acceptable because it conforms to the guidance in SRP Section 14.2 and SRP Section 11.2 and because these tests determine the correct installation and functional operability of the system equipment.

Inspections, Tests, Analyses, and Acceptance Criteria (ITAAC) The ITAAC are all incorporated by reference with no changes

11.2.5 Post Combined License Activities

There are no post COL activities related to this section.

11.2.6 Conclusion

NRC staff reviewed the STP Unit 3 and 4 COL FSAR and checked the referenced ABWR DCD. The staff's review confirmed that the application has addressed the relevant information. With the exception of **Confirmatory Item 11.02-10** no additional outstanding information is expected to be addressed in the COL FSAR related to this section.

The staff compared the COL information in the application to the relevant NRC regulations, the acceptance criteria in NUREG-0800 Section 11.2, and other NRC regulatory guides. Pending resolution of confirmatory item 11.02-10, the staff concludes that the applicant has met the applicable regulations and is in conformance with the applicable guidance.

The staff also concluded pending resolution of confirmatory item 11.02-10, that the LWMS includes the equipment necessary to control releases of radioactive materials in liquid effluents, in accordance with GDC 60 and 61 of Appendix A to 10 CFR Part 50 and the requirements of 10 CFR 50.34a. The staff concluded that the LWMS design is acceptable and meets the requirements of 10 CFR 20.1406, 10 CFR 50.34a, GDC 60 and 61, and Appendix I, Section II.D to 10 CFR Part 50.

These conclusions are based on the following considerations:

1. By preparing a plant-specific, cost-benefit analysis in accordance with RG 1.110, the applicant meets the requirements of Section II.D of Appendix I to 10 CFR Part 50, with respect to meeting the ALARA criterion using site-specific conditions. The staff considered the potential effectiveness of augmenting the LWMS using items of reasonably demonstrated technology. The staff determined that further effluent treatment will not affect reductions in cumulative population doses reasonably expected within an 80-kilometer (km) (50-mile [mi]) radius of the reactor, at a cost of less than \$1,000 per total body man-rem or thyroid man-rem.
2. The applicant meets the requirements of GDC 60 and 61 with respect to controlling releases of radioactive materials into the environment, by assuring that the LWMS design includes the equipment to monitor and control releases of radioactive materials in liquid effluents. A radiation monitor tracks all releases, and it will generate a signal to terminate liquid radwaste releases before the discharge concentration exceeds a predetermined setpoint. The COL applicant is required to identify appropriate operational setpoints for the LWMS radiation monitor in the plant-specific ODCM, as described in Section 11.2.5.
3. The applicant demonstrates compliance with the requirements of GDC 61 by meeting the guidelines of RG 1.143, as supported with additional commitments described in Subsection 11.2.1.2.4 and the installation of steel liners in cubicles where liquid radioactive waste tanks are located, as described in Subsection 11.2.1.2. These commitments also fulfill the requirements of 10 CFR 20.1406 and the guidance of RG 4.21, in minimizing the contamination of the facility and the generation of radioactive waste, and satisfies IE Bulletin 80-10 in avoiding the cross-contamination of nonradioactive systems and unmonitored and uncontrolled radioactive releases into the environment.

4. The staff reviewed the provisions incorporated in the applicant's design to:
- control the release of radioactive materials in liquids resulting from inadvertent tank overflows.
 - avoid the contamination of nonradioactive systems.
 - prevent uncontrolled and unmonitored releases of radioactive materials into the environment.
 - avoid interconnections with potable and sanitary water systems.

The staff concluded that the measures proposed by the applicant are consistent with GDC 60 and 61, IE Bulletin 80-10, and the guidance of RGs 1.143 and 4.21 for liquids and liquid wastes produced during normal operations and AOOs.

The applicant demonstrates compliance with the requirements of 10 CFR 52.80(a) for the LWMS. If the inspections, tests, and analyses are performed as required, and the respective acceptance criteria are met, there will be reasonable assurance that the LWMS has been constructed and will be operated in accordance with the license, the Atomic Energy Act, and NRC regulations

11.3 Gaseous Waste Management System

11.3.1 Introduction

The gaseous waste management system (GWMS) receives and processes radioactive gases and hydrogen bearing gases generated during process operation. There are two main sources of plant gaseous radioactive effluents. One source is from building ventilation systems servicing radiologically controlled areas, and the other source is from the power cycle offgas system (OGS). The GWMS and its OGS are used to control, collect, process, hold for decay, and discharge gaseous radioactive wastes generated during normal operations, including AOOs. The major components of the OGS include preheaters; recombiners; cooler/condensers; dryers; activated charcoal beds (guard and delay); and associated valves, pumps, and instrumentation. The OGS is located in the turbine building. The gases removed from the condenser are radioactive and must be treated before being released into the environment to ensure that radioactivity levels are reduced to acceptable levels and ALARA. The GWMS is designed to reduce and control radioactive releases into the environment. Releases from the OGS are directed via the turbine building stack. Releases from building ventilation exhaust systems servicing radiologically controlled areas are directed through their respective buildings, the reactor/fuel building stack, the turbine building stack, and the radwaste building stack. The primary release point for an ABWR plant is the reactor building plant stack, which is a combined release point for the reactor building, turbine building, and radwaste building.

11.3.2 Summary of Application

Section 11.3 of the STP Units 3 and 4 COL FSAR incorporates by reference Section 11.3 of the certified ABWR DCD, Revision 4, referenced in 10 CFR Part 52, Appendix A with the following departures:

Tier 2 Departures Requiring Prior NRC Approval

- STD DEP 10.4-5 Condensate and Feedwater System

In Section 2.2, "Departures from the Generic Technical Specifications," of Part 7 of the COL application, the applicant identifies two effects from this departure: technical specification (TS) and non-TS effects. In Section 11.3, this departure refers only to Table 11.3-3, Figure 11.3-1, and Figure 11.3-2, which relate to the changing equipment items: recombiner to reheater, catalyst to recombiner, and recombiner condenser to condenser. These changes in equipment items are considered to have no effects on the TS.

This departure makes the following changes to the GWMS:

- Changes the offgas recombiner from an integral unit to an independent pre-heater, recombiner, and condenser arranged in a recombiner train.

Tier 2 Departures Not Requiring Prior NRC Approval

- STD DEP 10.4-3 Main Condenser Evacuation System

This site-specific departure adds an additional mechanical vacuum pump, so the design now consists of two vacuum pumps.

- STD DEP 11.3-1 Gaseous Waste Management System

This departure modifies the GWMS design approved in the ABWR DCD. The changes include deleting equipment, deleting tanks, maximizing charcoal efficiency, creating one pathway for gaseous treatment versus parallel lines, and adding an evacuation system to provide a stable offgas flow to the plant exhaust.

This departure makes the following specific changes to the GWMS:

- Adds an offgas evacuation system downstream of the high efficiency particulate air (HEPA) filter to stabilize the offgas flow to the plant exhaust.
- Revises the charcoal adsorber vault temperature to a tighter range that maximizes charcoal efficiency.
- Changes the number of charcoal adsorber vessels from nine (one guard bed and eight adsorbers) to five (one guard bed and four adsorbers).
- Changes the arrangement of the charcoal adsorbers from four parallel lines, each with two adsorbers in a series, to four larger adsorbers in a series.
- Revises the mass of charcoal in each of the charcoal adsorber vessels from 13,600 kg (for the eight adsorbers) to 27,200 kg (for the four larger adsorbers). The total charcoal mass in the adsorbers is unchanged. The accident analyses in FSAR Section 15.7 assume that the charcoal adsorbers downstream of the guard bed will be bypassed, so the accident analyses are unaffected.

- Changes the mass of charcoal in the guard bed from 4,500 kg to 4,721 kg in Section 11.3, which is consistent with the accident analysis described in Section 15.7.1.

COL License Information Item

- COL License Information Item 11.2 Compliance with Appendix I to 10 CFR Part 50

DCD directs the COL applicant to demonstrate compliance with Appendix I to 10 CFR Part 50 numerical guidelines for offsite radiation doses as a result of gaseous or airborne radioactive effluents during normal plant operations, including anticipated operational occurrences shall be provided.

11.3.3 Regulatory Basis

The regulatory basis of the information incorporated by reference is in NUREG–1503. In addition, the relevant requirements of Commission regulations for the GWMS, and associated acceptance criteria, are in Section 11.3 of NUREG–0800.

In particular, the regulatory basis for accepting the supplementary information on GWMS is established in 10 CFR 20.1406; 10 CFR 50.34a; Appendix A to 10 CFR Part 50, GDC 60 and 61; Appendix I to 10 CFR Part 50, Section II.D; and 10 CFR 52.80(a). SRP acceptance criteria include codes and standards listed in Table 1 of RG 1.143; Regulatory Position C.2 of RG 1.143; RGs 1.112, 1.110, and 4.21; and BTP 11-5 of Section 11.3 to the SRP. The guidance of RG 1.140 is in the descriptions of design considerations of ventilation systems in FSAR Revision 0, Section 9.4, for building exhaust systems and venting tanks and vessels. Complete descriptions of the applicable regulatory and acceptance criteria are in SRP Section 11.3 (NUREG–0800).

In accordance with Section VIII, “Processes for Changes and Departures,” of “Appendix A to Part 52--Design Certification Rule for the U.S. Advanced Boiling Water Reactor,” the applicant identifies Tier 2 departures. Tier 2 departures affecting the TS require prior NRC approval and are subject to the requirements of 10 CFR Part 52, Appendix A, Section VIII.C.4. Tier 2 departures not requiring prior NRC approval are subject to the requirements in Section VIII.B.5 of 10 CFR Part 52, Appendix A, which are similar to the requirements in 10 CFR 50.59.

11.3.4 Technical Evaluation

As documented in NUREG–1503, NRC staff reviewed and approved Section 11.3 of the certified ABWR DCD. The staff reviewed Section 11.3 of the STP Units 3 and 4 COL FSAR. The staff also checked the referenced ABWR DCD to ensure that the combination of information in the COL FSAR and information in the ABWR DCD represents the complete scope of information relating to this review topic.¹ The staff’s review confirmed that the information in the application and the information incorporated by reference address the required information relating to the GWMS.

¹ See “*Finality of Referenced NRC Approvals*” in SER Section 1.1.3 for a discussion on the staff’s review related to verification of the scope of information to be included in a COL application that references a design certification.

The applicant provides new and revised information on the STP Units 3 and 4 GWMS. The staff reviewed the application using the guidance of SRP Section 11.3 and RG 1.206 Section C.I.11.3, with the following specific considerations:

Tier 2 Departures Requiring Prior NRC Approval

- STD DEP 10.4-5 Condensate and Feedwater System

In Section 2.2, “Departures from the Generic Technical Specifications,” of Part 7 of the COL application, the applicant identifies two effects from this departure: TS and non-TS effects. In Section 11.3, this departure refers only to Table 11.3-3, Figure 11.3-1, and Figure 11.3-2, which relate to the changing equipment items: recombiner to reheater, catalyst to recombiner, and recombiner condenser to condenser. The applicant states that these changes in equipment items are considered to have no effects on the TS.

During the review of STD DEP 10.4-5, the staff identified areas in need of additional information to complete the evaluation. The staff noted that the description in FSAR Subsection 11.3.3.3, “Process Facility,” removes provisions for supplying an alternative source to cool the offgas condenser. The staff requested additional information in RAI 11.3-03 to confirm that the applicant has satisfied the guidance in SRP 11.3, “Gaseous Waste Management System.”

The applicant responded to this RAI in a letter dated September 3, 2009 (ML092510039). In RAI 11.3-03, the staff asked the applicant to provide in the FSAR an additional discussion on how the turbine building cooling water provides equivalent reliability and quality such as reactor condensate as a suitable coolant for the offgas condensers. The applicant’s response provides information confirming the reliability and water quality of the turbine cooling water (TCW). The applicant concludes that the TCW will at least be as reliable as the condensate system and the water will be of higher quality and cooler than the water in the condensate system. The staff found that this design change does not adversely affect the GWMS or change the safety conclusions stated in Section 11.3 of the SER for the referenced ABWR DCD. Therefore, RAI 11.03-3 is closed.

Tier 2 Departures Not Requiring Prior NRC Approval

The applicant’s evaluation in accordance with 10 CFR Part 52, Appendix A, Section VIII, Item B.5, determines that the following departures do not require prior NRC approval. With respect to the impact of these departures on this section, NRC staff found it reasonable that these departures do not require prior NRC approval. The applicant’s process for evaluating departures and other changes to the DCD is subject to NRC inspections.

- STD DEP 10.4-3 Main Condenser Evacuation System

This site-specific departure adds an additional mechanical vacuum pump, which changes the design to two vacuum pumps and changes the source of motive steam supplying the steam jet air ejectors during power operation.

This departure does not require any further evaluation as it pertains to the review scope of this section. The main condenser vacuum releases continue to be processed through the monitored and sampled combined plant release point as designed. Furthermore, the staff’s review and evaluation of the acceptability of this departure are in Section 10 of this SER.

- STD DEP 11.3-1

Gaseous Waste Management System

This departure makes the following changes to the GWMS:

- Adds an offgas evacuation system downstream of the HEPA filter to stabilize the offgas flow to the plant exhaust.
- Revises the charcoal adsorber vault temperature to a tighter range to maximize charcoal efficiency.
- Changes the number of charcoal adsorber vessels from nine (one guard bed and eight adsorbers) to five (one guard bed and four adsorbers). This departure also changes the arrangement of the charcoal adsorbers from four parallel lines, each with two adsorbers in a series, to four larger adsorbers in a series.
- Revises the mass of charcoal in each charcoal adsorber vessel from 13,600 kg (for the eight adsorbers) to 27,200 kg (for the four larger adsorbers); the total mass of charcoal in the adsorbers is unchanged. Note that the accident analyses in Section 15.7 assume the bypass of the charcoal adsorbers downstream of the guard bed, so the accident analyses are unaffected.
- Changes the mass of charcoal in the guard bed from 4,500 kg to 4,721 kg in Section 11.3 to be consistent with the accident analyses described in Section 15.7.1.

During the review of Departures STD DEP 11.3-1 and STD DEP 10.4-5, the staff identified areas in need of additional information to complete its evaluation. The staff noted that Table 11.3-3, "Equipment Malfunction Analysis," includes only the preheater, recombiner, and condenser. The staff also noted substantial changes in Figure 11.3-1, "Offgas System PFD," and Figure 11.3-2, "Offgas System P&ID," including the removal of notes and other design information. The staff requested additional information in RAI 11.3-04 to confirm the acceptability of the applicant's evaluation per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5.

The applicant responded to this RAI in a letter dated September 3, 2009 (ML092510039). The applicant states that due to a redesigned offgas system, the DCD P&ID's (including the notations) are not applicable. The staff found that the changes to the table and figures listed above do not conflict with the applicant's evaluation per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5.

The applicant's response to RAI 11.3-04 also includes additional information pertaining to 10 CFR 20.1406 and proposes to add Subsection 11.3.2.1, "Offgas System Compliance with Part 20.1406," to the FSAR to confirm compliance with this regulation. The staff reviewed this additional information for Subsection 11.3.2.1 and found it acceptable regarding the minimization of radioactive waste generation. The offgas system is designed to minimize the generation of radioactive waste in that the charcoal in the carbon beds is not intended to be replaced during the entire plant life. The charcoal delay beds will not contribute to or be processed into solid radwaste, and removal will be at the end of plant life. Accordingly, the question is resolved and RAI 11.03-4 is closed.

Section 3 of Part 7, "Departures Report," of the COL application indicates that this standard departure was evaluated and found to comply with the requirements of 10 CFR Part 52,

Appendix A, Section VIII.B.5. The staff was not able to determine the acceptability of the applicant's evaluation per the requirements in 10 CFR Part 52, Appendix A, Section VIII.B.5. Accordingly, the staff audited the applicant and conducted an independent evaluation. The purpose of these activities was to ascertain whether enough information existed to determine the acceptability of the applicant's screening evaluations for each technical and regulatory aspect of this departure. The staff found sufficient information for this particular departure. Therefore, the applicant's evaluation is acceptable because it satisfies the requirements in Section VIII.B.5 of Appendix A to 10 CFR Part 52.

Evaluation of Plant-Specific Information

Compliance with 10 CFR 20.1406

10 CFR 20.1406 requires the applicant to provide a description of how facility design and procedures for operations (a) will minimize, to the extent practicable, contamination of the facility and the environment; (b) will facilitate eventual decommissioning; and (c) will minimize, to the extent practicable, the generation of radioactive waste.

The applicant provides information in Section 11.2 related to compliance with 10 CFR 20.1406 to minimize contamination to facilitate decommissioning. The applicant lists several measures that will prevent the spread of contamination, including design features using engineering controls; criteria for selecting materials and equipment used in contaminated areas; the use of containments, caches, and enclosures to contain spills and releases; surveys of potentially contaminated systems, equipment, and components; and the use of temporary and permanent design modifications to prevent and limit the spread of contamination. Most of the items listed in Section 11.2 were taken from the list of practical measures to prevent the spread of contamination in NEI 07-03A, "Generic FSAR Template Guidance for Radiation Protection Program Description." To determine compliance with the guidance in RG 4.21, the applicant states that the application will incorporate NEI 08-08A, "Generic FSAR Template Guidance for Life Cycle Minimization of Contamination," which was developed to address the guidance in RG 4.21.

The applicant meets the requirements in 10 CFR 20.1406 and the guidance of RG 4.21 by describing how facility design features and procedures for operations (described in FSAR Sections 11.2 and 12.3) will minimize, to the extent practicable, contamination of the facility and the environment; will facilitate eventual decommissioning; and will minimize, to the extent practicable, the generation of radioactive waste.

Compliance with the Cost-Benefit Ratio in 10 CFR Part 50, Appendix I

To assess compliance with numeric guideline II.D in 10 CFR Part 50, Appendix I, NRC staff evaluated the methodology used to determine the cost-benefit ratio prescribed in RG 1.110. Because the staff was not able to determine whether the applicant is in conformance with the guidance prescribed in RG 1.110, the staff issued RAI 11.03-1. The applicant's initial response to this RAI dated May 21, 2009 (ML091460243), did not supply enough information and RAI 11.03-1 was closed as unresolved. The staff then issued supplemental RAI 11.03-7. The applicant's response to RAI 11.03-7 dated October 12, 2009 (ML092890082), provides all essential information and the methodology used to determine the cost-benefit ratio prescribed in RG 1.110.

The results of the applicant's analysis show that the lowest cost option for gaseous radwaste treatment system augment is an addition of one 3-ton charcoal adsorber to the GWMS effluent stream before entry into the plant vent line. Assuming that this augment will eliminate all radioactivity from the gaseous effluent, the resulting cost per person-rem dose reduction is \$580 for total body dose. This amount is below the cost criterion of \$7,750 per person-rem for an augment in 10 CFR Part 50, Appendix I, Section II.D. Thus, the applicant concludes that the GWMS meets ALARA requirements and no augments are necessary.

The staff performed an independent assessment using the population doses calculated by the staff (see SER Section 12.2) and the guidance in RG 1.110 and reached the same conclusion. Because the staff's population doses were different from the applicant's doses, the staff obtained different, but acceptable cost-benefit ratios for the addition of one 3-ton charcoal adsorber to the GWMS effluent stream prior to entry into the plant vent stack. SER Table 11.3-1 lists the costs calculated by the applicant and staff for comparison. RAI 11.03-7 is closed.

Table 11.3-1 - Comparison of GWMS Cost-Benefit Ratios (Per Person-Rem)

Organ/Body	Applicant Costs to Meet the Design Criteria *	Applicant Costs to Augment the GWMS	NRC Costs to Meet the Design Criteria Calculation *	NRC Costs to Augment the GWMS
Total Body	\$580	\$7,750	\$746	\$7,755
Thyroid	\$2,380	\$7,750	\$924	\$7,755

*Based upon liquid population dose in SER Section 12.2.

Compliance with BTP 11-5 Postulated Radioactive Releases Due To a Waste Gas System Leak or Failure

NRC staff evaluated conformance with BTP 11-5, which addresses the consequences of the waste gas system failure. The staff issued RAI 11.03-2 requesting the applicant to provide information and details on how the applicant performed this failure analysis. The staff also issued RAI 11.03-5 requesting the applicant to review the information and the data presented in Subsection 15.7.1.1 with the updated BTP 11-5, Revision 3. The applicant's responses to these RAIs dated September 3, 2009 (ML092510039), state that there are no departures from Subsection 15.7.1.1 of the certified design. DCD Subsection 15.7.1.1 analyses are based on the inadvertent bypass of the downstream charcoal delay beds, and the analyses are identical to the STP COL 15.7.1.1 event and the event described in BTP 11-5. The staff's review determined that the analysis in Subsection 15.7.1.1 is identical to that described by BTP 11-5 and the information provided by the applicant is acceptable. RAI 11.03-2 and RAI 11.03-5 are closed.

COL License Information Item

COL License Information Item 11.2 Compliance with Appendix I to 10 CFR Part 50

ABWR DCD Tier 2 Subsection 11.3.11.1, "COL License Information," states that the COL applicant shall provide the COL license information item that applies on a plant-specific basis. For the STP COL application, this COL license information item is addressed as follows:

COL License Information Item 11.2 states that the applicant shall demonstrate compliance with Appendix I to 10 CFR Part 50 numerical guidelines for offsite radiation doses resulting from gaseous or airborne radioactive effluents during normal plant operations, including AOOs. Subsection 11.3.11.1 of the FSAR states that all required information concerning this COL license information is in FSAR Subsection 12.2.2.2.2. This COL license information item is addressed in Section 12.2.

Supplemental Information

Preoperational Tests

In COL FSAR Subsection 14.2.12.1.35, "Gaseous Radwaste Management/Offgas Systems Preoperational Tests," the applicant states that the GWMS is tested during the Preoperational Test Program. The GWMS equipment will be performance tested to demonstrate conformance with design process capabilities. An integrity test is performed on the system upon completion. The staff found that this item conforms to the guidance in SRP 14.2 because these tests will determine correct installation and functional operability of the system equipment.

ITAAC

The ITAAC are all incorporated by reference with no changes.

11.3.5 Post Combined License Activities

There are no post COL activities related to this section.

11.3.6 Conclusion

NRC staff reviewed the STP Unit 3 and 4 COL FSAR and checked the referenced ABWR DCD. The staff's review confirmed that the application has addressed the relevant information, and no outstanding information is expected to be addressed in the COL FSAR related to this section.

The staff compared the additional COL information in the application to the relevant NRC regulations, the acceptance criteria in NUREG-0800 Section 11.3, and other NRC regulatory guides. The staff concluded that the applicant is in compliance with the regulations.

The staff also concluded that the GWMS (as a permanently installed system and in combination with other plant systems) includes the equipment necessary to control releases of radioactive materials in gaseous effluents in accordance with GDC 60, and 61 of Appendix A to 10 CFR Part 50 and the requirements in 10 CFR 50.34a.

The staff concluded that the GWMS design is acceptable and meets the requirements of 10 CFR 20.1406; 10 CFR 50.34a; GDC 60, and 61; and Section II.D of Appendix I to 10 CFR Part 50. This conclusion is based on the following:

1. The applicant meets the requirements of GDC 60 and 61 with respect to controlling releases of radioactive materials into the environment, by assuring that the GWMS design includes the equipment to monitor and control releases of radioactive materials in gaseous and particulate effluents.
2. The applicant meets the requirements of Section II.D of Appendix I to 10 CFR Part 50, with respect to meeting the ALARA criterion using site-specific conditions. The staff considered the potential effectiveness of augmenting the GWMS using items of reasonably demonstrated technology. The staff determined that further effluent treatment will not affect reductions in cumulative population doses reasonably expected within an 80-km (50-mi) radius of the reactor, at a cost of less than \$1,000 per total body man-rem or per thyroid man-rem.
3. The applicant meets the requirements in 10 CFR 20.1406 and the guidance of RG 4.21 by describing how facility design features and procedures for operations (described in FSAR Sections 11.3 and 12.3) will minimize, to the extent practicable, contamination of the facility and the environment; will facilitate eventual decommissioning; and will minimize, to the extent practicable, the generation of radioactive waste.
4. The staff confirmed that the applicant has incorporated by reference Section 15.7 of DCD Tier 2 Revision 4, the results of the analysis of a waste gas system failure, and that no changes to the results are necessary. Section 15.7 of the certified ABWR DCD, Revision 4, demonstrates that the OGS design meets the applicable guidelines of SRP Section 11.3 and BTP 11-5.
5. The applicant demonstrates compliance with the requirements of 10 CFR 52.80(a) for the GWMS. If the inspections, tests, and analyses are performed as required, and the respective acceptance criteria are met, there will be reasonable assurance that the GWMS has been constructed and will be operated in accordance with the license, the Atomic Energy Act, and NRC regulations.

11.4 Solid Waste Management System

11.4.1 Introduction

This FSAR section addresses how the solid waste management system (SWMS) manages radioactive wastes such as liquid, wet, and dry solid wastes produced during normal operations and AOOs. The review includes an evaluation of any additional equipment that may be necessary to process liquid, dry, and wet wastes and (1) route them to the point of discharge from the SWMS; or (2) prepare them for shipment to authorized offsite disposal sites or licensed radioactive waste processors. The SWMS has no safety-related function. There is no liquid or gaseous plant discharge from the SWMS. A failure of any subsystem within the SWMS does not compromise any safety-related system or component or prevent a shutdown of the plant.

11.4.2 Summary of Application

The applicant incorporates by reference Tier 1 Section 2.9, "Radioactive Waste System," of the certified ABWR DCD. Tier 2 Section 11.4 contains one departure that changes the design of the SWMS. In addition, this section addresses COL License Information Item 11.3.

Tier 2 Departure Not Requiring Prior NRC Approval

- STD DEP 11.4-1 Radioactive Solid Waste Update

The applicant completely replaces Tier 2 Section 11.4 of the certified ABWR DCD, including all subsections, figures, and tables. This standard departure from the DCD SWMS design deletes the solidification, incineration, and compacting processes. Thus, no part of ABWR DCD Section 11.4 is incorporated by reference.

COL License Information Item

- COL License Information Item 11.3 Plant-Specific Solid Radwaste Information

DCD COL License Information Item 11.3 directs the COL applicant to provide the following six plant-specific items:

1. Description of the incinerator complete with the source of incinerator heat, heat source storage facility and specific fire protection features to prevent any undue fire hazard shall be provided.
2. Demonstration that the wet waste solidification process and the spent resin and sludge dewatering process will result in products that comply with 10 CFR 61.56 shall be provided.
3. Establishment and implementation of a process control program (PCP) for solidifying the evaporator concentrates, using an approved solidification agent, and the dewatering processing of the spent resins and filter sludges shall be provided.
4. Discussion of onsite storage of low-level waste beyond that discussed in Tier 2 shall be provided.
5. Demonstration that all radioactive waste shipping packages meet the requirements in 10 CFR Part 71 shall be provided.
6. Establishment of setpoints for the liquid discharge radiation monitor based on the as-built design.

11.4.3 Regulatory Basis

The relevant requirements of the Commission regulations for the SWMS, and associated acceptance criteria, are in Section 11.4 of NUREG-0800. The regulatory basis is established in several codes and standards that include Table 1 of RG 1.143; Regulatory Positions C.3.2 and C3.3 of RG 1.143; 10 CFR Parts 20, 50, and 71; 49 CFR Part 173; and disposal site waste form requirements for burial at a low-level waste disposal site that is licensed in accordance with 10 CFR Part 61, or equivalent Agreement State regulations.

consistent with the requirements of 10 CFR Part 52, Appendix A, Section VIII.B.5, the staff reviewed the applicant's evaluation procedures and processes and the bases for the applicant's determination that prior NRC approval is not required for this departure.

The staff conducted an audit of the 10 CFR Part 52, Appendix A, Section VIII.B.5 evaluation process (ML092510426). The staff then issued RAI 11.04-5 requesting the applicant to re-evaluate the initial departure evaluations to determine whether Departure STD DEP 11.4-1 "would" create a system malfunction or cause a malfunction with a different result. The applicant's response to this RAI dated October 12, 2009 (ML092890082), reevaluates Departure STD DEP 11.4-1 and concludes that the changes to the SWMS will not increase the consequences of an equipment malfunction or cause a malfunction with a different result.

The staff reviewed the applicant's response to RAI 11.04-5 and found that it addresses the requirements of the STP procedures and 10 CFR Part 52, Appendix A, Section VIII.B.5. The response provides sufficient information to conclude that the applicant evaluated the SWMS departure in accordance with 10 CFR Part 52. The staff found the applicant's response acceptable, and RAIs 11.04-1 and 11.04-5 are closed.

Evaluation of Plant-Specific Information

Compliance with 10 CFR 20.1406

10 CFR 20.1406 requires the applicant to provide a description of how facility design and procedures for operations (a) will minimize, to the extent practicable, contamination of the facility and the environment; (b) will facilitate eventual decommissioning; and (c) will minimize, to the extent practicable, the generation of radioactive waste.

The applicant provides information in Section 11.4 related to compliance with 10 CFR 20.1406 to minimize contamination to facilitate decommissioning. The applicant lists several measures that will prevent the spread of contamination, including design features using engineering controls; criteria for selecting materials and equipment used in contaminated areas; the use of containments, caches, and enclosures to contain spills and releases; surveys of potentially contaminated systems, equipment, and components; and the use of temporary and permanent design modifications to prevent and limit the spread of contamination. Most of the items listed in Section 11.4 were taken from the list of practical measures to prevent the spread of contamination in NEI 07-03A, "Generic FSAR Template Guidance for Radiation Protection Program Description." To determine compliance with the guidance in RG 4.21, the applicant states that the application will incorporate NEI 08-08A, "Generic FSAR Template Guidance for Life Cycle Minimization of Contamination," which was developed to address the guidance in RG 4.21.

The applicant meets the requirements in 10 CFR 20.1406 and the guidance of RG 4.21 by describing how facility design features and procedures for operations (described in FSAR Sections 11.2, 11.4 and 12.3) will minimize, to the extent practicable, contamination of the facility and the environment; will facilitate eventual decommissioning; and will minimize, to the extent practicable, the generation of radioactive waste.

.COL License Information Item

- COL License Information Item 11.3 Plant-Specific Solid Radwaste Information

ABWR DCD Tier 2 Section 11.4.3, "COL License Information," states that the COL applicant shall provide several COL license information items that apply on a plant-specific basis. For the STP COL application, these COL license information items are addressed as follows:

1. DCD COL License Information Item 11.3 Item 1 states, "A description of the incinerator complete with the source of incinerator heat, heat source storage facility, and specific fire protection features to prevent any undue fire hazard shall be provided."

The applicant states that this departure completely replaces the previously approved SWMS design in NUREG-1503 and deletes the incinerator process. Thus, the information requested in this COL license information item is no longer necessary or applicable.

Therefore, the applicant has adequately addressed COL License Information Item 11.3, Item 1.

2. DCD COL License Information Item 11.3 Item 2 states, "Demonstration that the wet waste solidification process and the spent resin and sludge dewatering process will result in products that comply with 10 CFR 61.56 shall be provided."

Departure STD DEP 11.4-1 in FSAR Section 11.4 deletes the solidification process, so that portion of the COL license information item is no longer necessary or applicable.

The staff's review of this departure found that the applicant has provided informative descriptions of the spent resin and sludge dewatering process and the mobile dewatering processing subsystem equipment, in sufficient detail to confirm compliance with 10 CFR 61.56. Therefore, the applicant has adequately addressed COL License Information Item 11.3, Item 2.

3. DCD COL License Information Item 11.3 Item 3 states, "Establishment and implementation of a process control program (PCP) for solidifying the evaporator concentrates, using an approved solidification agent, and the dewatering processing of the spent resins and filter sludges shall be provided."

The applicant points out that Table 13.4S-1, "Operational Programs Required by NRC Regulation and Program Implementation," provides a milestone date for fuel loading to implement the PCP. The staff has identified the implementation of the PCP as a license condition.

The staff issued RAI 11.04-2 requesting the applicant to provide additional information concerning key elements of the program that are to be included in the PCP. The applicant's response dated June 26, 2008 (ML08197231), states that the STP site PCP used by Units 1 and 2 will also be used by Units 3 and 4.

The staff issued an additional RAI 11.04-4 requesting the applicant to address the PCP and the potential incorporation of the NEI 07-10A, "Generic FSAR Template Guidance for Process Control Program (PCP)," in the FSAR. The applicant's response to this RAI dated September 3, 2009 (ML092510039), modifies Section 11.4.3 to state that the PCP will incorporate the guidance in NEI 07-10A. The FSAR has been revised to incorporate these

changes, therefore, RAI 11.04-2 and RAI 11.04-4 are closed. Therefore, the applicant has adequately addressed COL License Information Item 11.3, Item 3.

4. DCD COL License Information Item 11.3 Item 4 states, "A discussion of onsite storage of low-level waste beyond that discussed in Tier 2 shall be provided."

The applicant discusses onsite storage space for a 6-month volume of packaged waste beyond that discussed in ABWR DCD Section 11.4.2. The applicant also provides estimates of expected Class A, B, and C radwaste volumes.

The staff issued RAI 11.04-3 requesting the applicant to provide additional information for the radwaste data already presented. The staff questioned whether the data represents one or both units. The staff also asked for clarification of the radwaste volume differences. The applicant's response dated May 21, 2009 (ML091460243), clarifies the radwaste information and revises Subsection 11.4.2.2.6 in Revision 3 of the FSAR. The staff found that the applicant's response clarifies the data and the waste volumes for each unit, this information has been incorporated into the FSAR and RAI 11.04-3 is closed. Therefore, the applicant has adequately addressed COL License Information Item 11.3, Item 4.

5. DCD COL License Information Item 11.3 Item 5 states, "Demonstration that all radioactive waste shipping packages meet the requirements in 10 CFR Part 71 shall be provided."

In COL FSAR Subsection 11.4.3,1, "Plant-Specific Solid Radwaste Information," the applicant states that "Radioactive waste shipping packages meet the requirements in 10 CFR Part 71 for STP 3 & 4 as provided in the plant radiation protection program as described in Section 12.5.3." Subsection 12.5.3.1 states that the operational Radiation Protection Program is described in Section 12.5S. Section 12.5S states that NEI 07-03A provides the Operational Radiation Protection Program for STP Units 3 and 4. The program states, "Prior to initial transfer, transport, or disposal of radioactive materials, the organization, facilities, equipment, instrumentation, and procedures will be in place as necessary to assure compliance with 10 CFR Part 20, Subpart K, and applicable requirements in 10 CFR Part 71."

The staff found this information sufficient to close this item, because the applicant has incorporated NEI 07-03A to provide the Radiation Protection Program guidance for compliance with 10 CFR Part 71 requirements. Therefore, the applicant has adequately addressed COL License Information Item 11.3, Item 5.

6. DCD COL License Information Item 11.3 Item 6 states, "Based on the as-built design, establish set points for the liquid discharge radiation monitor."

The applicant addresses this item in FSAR Section 11.5 by noting that the ODCM describes the establishment of the liquid discharge radiation monitor setpoints.

The staff issued RAI 11.04-6 requesting the applicant to clarify the tracking of COL license information items between DCD Table 1.9-1 and Section 11.4 and FSAR Section 11.4.3. The applicant's response to this RAI dated December 9, 2009 (ML09348068), commits to revising Section 11.4 as requested. This revision has been incorporated into the FSAR and RAI 11.04-6 is therefore closed. The applicant has adequately addressed COL License Information Item 11.3, Item 6.

Supplemental Information

Preoperational Tests

In COL FSAR Subsection 14.2.12.1.75, "Liquid and Solid Radwaste Systems Preoperational Tests," the applicant states that the SWMS is tested during the Preoperational Test Program. The SWMS equipment will be performance tested to demonstrate conformance with design process capabilities. An integrity test is performed on the system upon its completion. NRC staff found that this item conforms to the guidance in SRP Section 14.2.

ITAAC

The ITAAC are all incorporated by reference with no changes.

11.4.5 Post Combined License Activities

License Condition for the PCP before Fuel Loading

NRC staff included the following two license conditions:

1. "Prior to fuel loading, the licensee shall implement an operational program for process and effluent monitoring and sampling." The program shall include the following subprograms and documents:
 - a. Radiological Effluent Technical Specifications/Standard Radiological Effluent Controls
 - b. Offsite Dose Calculation Manual
 - c. Radiological Environmental Monitoring Program
 - d. Process Control Program
2. The licensee shall submit to the NRC a schedule, no later than 12 months after the issuance of the combined operating license that, supports planning for the conduct of NRC inspections of the four operating programs and documents listed in the above license condition (number 1). The schedule shall be updated every 6 months until 12 months before the scheduled fuel loading and every month thereafter, until either the four operational programs and documents have been fully implemented or the plant has been placed in commercial service.

11.4.6 Conclusion

The staff compared the COL information in the application to the relevant NRC regulations, the acceptance criteria in NUREG-0800 Section 11.4, and other NRC regulatory guides. The staff concluded that the applicant is in compliance with the regulations.

The staff also concluded that the SWMS, either as a permanently installed system or in combination with other plant systems such as the LWMS and GWMS, is acceptable and meets the requirements of Table 1 of RG 1.143; Regulatory Positions C.3.2 and C3.3 of RG 1.143; 10 CFR Parts 20, 50, and 71; 49 CFR Part 173; and disposal site waste form requirements for burial at a low-level waste disposal site that is licensed in accordance with 10 CFR Part 61, or equivalent Agreement State regulations. The SWMS also meets the applicable SRP branch technical positions and regulatory guides.

This conclusion is based on the following:

1. The applicant's proposed PCP for STP Units 3 and 4 and the operating procedures as they relate to classifying, processing, and disposing of wastes, meet the requirements of 10 CFR Part 61; 10 CFR 20.2006; 10 CFR 20.2007; and 10 CFR 20.2108. The staff concluded that the endorsement of NEI PCP Template 07-10A and the SWMS supplemental information in FSAR Section 11.4 are consistent with the requirements of Generic Letter 89-01, the guidance in NUREG-1302 for BWR plants, and the guidance in NUREG-0133.
2. The applicant meets the requirements of 10 CFR 20.1406, IE Bulletin 80-10, and the guidance in RG 4.21 with respect to providing a description of how facility design features and procedures for operations described in FSAR Sections 11.4 and 12 will minimize, to the extent practicable, contamination of the facility and the environment; will facilitate eventual decommissioning; and will minimize, to the extent practicable, the generation of radioactive waste.

The applicant demonstrates compliance with the requirements of 10 CFR 52.80(a) for the SWMS. If the inspections, tests, and analyses are performed as required, and the respective acceptance criteria are met, there will be reasonable assurance that the SWMS has been constructed and will be operated in accordance with the license, the Atomic Energy Act, and NRC regulations.

11.5 Process and Effluent Radiological Monitoring Instrumentation and Sampling Systems

11.5.1 Introduction

The process radiation monitoring system (PRMS) is used to monitor liquid and gaseous process streams and effluents from the LWMS, GWMS, and SWMS during normal operations and AOOs and under post-accident conditions. Another objective is to alert control room operators of abnormal levels of radioactivity in process streams and liquid and gaseous effluents in order to activate signals that initiate automatic safety functions, isolate process streams, and terminate effluent discharges if predetermined radioactivity levels or release rates exceed established alarm setpoints. The PRMS generates signals to initiate the operation of certain safety-related equipment to control radioactive releases under normal and abnormal operations and accident conditions. Another function of the PRMS is to provide the means to collect samples from process and effluent streams for radiological analyses that assess compliance with NRC regulations.

The PRMS consists of skid-mounted and permanently installed sampling and monitoring equipment designed to indicate operational radiation levels and releases of radioactive materials, equipment or component failures, and system malfunctions or improper operations. The PRMS includes beta and gamma radiation-sensitive detectors working in redundant channels, as required for each subsystem. The radiation detectors are capable of detecting the types and energies of radiation emitted from fuel, radioactive wastes, and process and effluent streams. Local readout and alarm modules are located in specific areas to provide information on the radiological status of plant systems and alert personnel to abnormal or accident conditions.

11.5.2 Summary of Application

Section 11.5 of the STP Units 3 and 4 COL FSAR incorporates by reference Section 11.5 of the certified ABWR DCD, Revision 4, referenced in 10 CFR Part 52, Appendix A. In addition, in FSAR Section 11.5, the applicant provides the following:

Tier 1 Departures

- STD DEP T1 2.3-1 Deletion of MSIV Closure and Scram on High Radiation

This departure deletes the Scram and main steam isolation valve (MSIV) automatic closure on the high main steam line radiation monitor signal.

- STD DEP T1 3.4-1 Safety-Related I&C Architecture

This departure updates the safety-related instrumentation and control (I&C) design architecture.

Tier 2 Departures Not Requiring Prior NRC Approval

- STD DEP 7.1-1 References to Setpoints and Allowable Values

This departure clarifies the references to setpoints and allowable values.

- STD DEP 11.5-1 Process and Effluent Radiation Monitoring and Sampling System

This departure addresses the process and effluent radiation monitoring and sampling system.

COL License Information Items

- COL License Information Item 11.4 Calculations of Radiation Release Rates

The COL applicant shall provide and describe in the operation and maintenance manual the procedures and/or methods for the conversion of the radiation measurements into release rates of gaseous discharge from the main plant stack (Section 11.5).

- COL License Information Item 11.5 Compliance with the Regulatory Shielding Design Basis

The COL applicant shall describe in the operation and maintenance manual the sampling system design of the SGTS and of the main stack effluent monitoring subsystems and show compliance with the regulatory shielding requirements for low-radiation exposure under accident conditions as stipulated in NUREG-0737 Item II.F.1, clarification 2 of Attachment 2. The requirement for the shielding design will be covered in the equipment design specifications (Section 11.5).

- COL License Information Item 11.6 Provisions for Isokinetic Sampling

The COL applicant shall describe in the operation and maintenance manual the sampling technique used for monitoring and sampling effluent gasses to assure that a representative gas

sample is extracted and that the sampling system is capable of maintaining isokinetic conditions within 20 percent of the flow rate during and following an accident as stipulated in NUREG-0737 Item II.F.1, clarification 3 of Attachment 2 (Section 11.5).

- COL License Information Item 11.7 Sampling of Radioactive Iodine and Particulates

The COL applicant shall describe in the operation and maintenance manual the collection technique used to extract representative samples of radioactive iodines and particulates during and following an accident. These measurements are used to determine the quantitative releases for dose calculations and assessments as stipulated in NUREG-0737, Table II.F.1-2 (Section 11.5).

- COL License Information Item 11.8 Calibration Frequencies and Techniques

The COL applicant shall provide in the operation and maintenance manual for the system the calibration frequencies and techniques for the radiation sensors. This information shall be based on vendor data for the equipment (Section 11.5).

Supplemental Information

The applicant provides supplemental information in Section 11.5.7S to address the ODCM, per RG 1.206 Section C.III.1.

11.5.3 Regulatory Basis

The regulatory basis of the information incorporated by reference is in NUREG-1503. In addition, the relevant requirements of Commission regulations for the PRMS, and associated acceptance criteria, are in Section 11.5 of NUREG-0800.

In particular, the regulatory basis for accepting the supplementary information for the PRMS is established in 10 CFR 20.1301, 10 CFR 20.1301(e), 10 CFR 20.1302, and 10 CFR 20.1406; 10 CFR 50.34a and 10 CFR 50.36a; Appendix A to 10 CFR Part 50, GDC 60, 63, and 64; Appendix I to 10 CFR Part 50, Sections II.A, II.B, II.C, and II.D; 10 CFR 52.80(a); and Generic Letter 89-01. Additional requirements include those of 10 CFR 50.34(f)(2)(xvii) and 10 CFR 50.34(f)(2)(xxvii) for monitoring gaseous effluents from potential accident release points, consistent with GDC 63 and 64. SRP acceptance criteria include RGs 1.21, 1.33, 1.97, 4.1, 4.8, and 4.15; industry codes and standards including ANSI/HPS N13.1 and ANS N42.18; and BTP 7-10 of SRP Section 7.5.

In addition, in accordance with Section VIII, "Processes for Changes and Departures," of "Appendix A to Part 52--Design Certification Rule for the U.S. Advanced Boiling Water Reactor," the applicant identifies Tier 1 and Tier 2 departures. Tier 1 departures require prior NRC approval and are subject to the requirements in VIII.A.4. Tier 2 departures are subject to the requirements in Section VIII.B.5, which are similar to the requirements in 10 CFR 50.59.

11.5.4 Technical Evaluation

As documented in NUREG-1503, NRC staff reviewed and approved Section 11.5 of the certified ABWR DCD. The staff reviewed Section 11.5 of the STP Units 3 and 4 COL FSAR and checked the referenced ABWR DCD to ensure that the combination of information in the COL FSAR and information in the ABWR DCD represents the complete scope of information relating

Radiation Monitoring Systems.” Under the column “Automatic Control Function (ACF) Trip,” the applicant uses a clause “Based on setpoint calculation” without a specific link to the ODCM. The applicant’s response to this RAI dated September 9, 2009 (ML092530686), states that in FSAR Subsection 11.5.3.4, the setpoints that initiate automatic functions are based on calculations developed in accordance with controlled plant procedures or, if pertaining to gaseous or liquid releases within the scope of the ODCM, in accordance with the ODCM. The applicant revised the FSAR text to state that the setpoint procedure for the “ACF Trip” in Table 11.5-1 for the reactor building vent exhaust, the fuel handling area air vent exhaust, and the radwaste liquid discharge is in the ODCM. The applicant modifies the departure description for STD DEP 7.1-1 in a similar manner. The staff found that all setpoints requiring ODCM determination were now correctly referenced to the ODCM, and the applicant’s response has adequately addressed the staff’s concerns. This also addresses COL License Information Item 11.3. The staff has verified that the FSAR has been updated to reflect the above changes, therefore RAI 11.05-2 is closed.

- STD DEP 11.5-1 Process and Effluent Radiation Monitoring and Sampling System

This departure includes a number of changes that affect numerous subsections of FSAR Section 11.5. These are discussed below.

Table 11.5-1, Table 11.5-2, and Table 11.5-3 are replaced and Table 11.5-7 is revised.

Section 5.0 of Part 7, “Departures Report,” contains tables showing departures and all affected sections, tables, and figures. This departure is classified by the applicant as not requiring prior NRC approval. In Section 3.0 of Part 7, the applicant describes and evaluates this departure per the requirements of Section VII.B.5 of Appendix A to 10 CFR Part 52. In the justification for this departure, the applicant states that the functional requirements set forth in the referenced ABWR DCD will be met, but the vendor determines the implementation of the design and specific equipment needs. The applicant states in Section 3.0 of Part 7 that the evaluation demonstrates that there is no impact on (1) the probability or consequences of an accident, (2) the malfunction of an SSC important to safety, or (3) the likelihood or consequences of a severe accident. Based on the evaluation of the changes proposed in Departure STD DEP 11.5, the applicant states that prior NRC approval of these changes is not required.

Deletion of References to Specific Types of Detectors

The applicant deletes references to specific types of detectors such as the digital gamma-sensitive Geiger-Mueller, the ionization chamber, or scintillation detectors. The applicant states that specific types of detectors will be selected based on state-of-the-art and availability. Furthermore, the referenced ABWR DCD Tier 1, Section 2.3.1, and Tier 1, Table 2.3.1, “Process Radiation Monitoring System,” specifies the type of radioactivity that will be monitored in each system.

Combination of Radiation Monitor Downscale (Low) Trip and the Inoperative Trip Function

The downscale (low) trip and the inoperative trip have been combined into one trip function (downscale/inoperative), because both trips are used for the same purpose: to detect equipment failure. Also, output signals from the radiation monitors will be expressed in units of Sievert rather than in units of Gray. Units of Sievert specifically give the absorbed radiation dose in human tissue, while units of Gray refer to the absorbed radiation dose in any material.

An example of obsolete or changing technology is the removal of “recorders” for data recording and trending. This function is now performed by trending software in digital I&C systems.

Deletion of Incinerator for Burning Low-Level Radwaste

STP Units 3 and 4 will not have an incinerator for burning low-level radwaste. Therefore subsections under FSAR Section 11.5 (i.e., Subsections 11.5.1.1.2, 11.5.2.2.11, 11.5.4.3, 11.5.5.2, and 11.5.5.2) that refer to the incinerator have been modified. Thus, the incinerator stack discharge radiation monitor is not required. References to specific calibration techniques and maintenance procedures such as calibration reproducibility, error precision, and timeliness for maintenance have been removed. These techniques and methods are specific to site procedures that are often controlled by the ODCM or supplied by the equipment vendors.

Elimination of “Warning Alarm” and the “Detector Type” Columns

The “Warning Alarm” column and the “Detector Type” column were eliminated in replacement Table 11.5-1. Warning alarms are not provided, included, discussed, listed, or just “in the text” of the specific entry for each radiation monitor. The “Warning Alarm” text in DCD Table 11.5-1 is vague. To retain the specific detector types found in DCD Table 11.5-1 could limit selection based on the state-of-the-art and availability. The “Sensitivity” column and the “Type” column were both eliminated in replacement Tables 11.5-2 and 11.5-3. Sensitivities are not included because they are vendor specific.

Correction of the Bypass Valve Closure Trip Function

To be consistent with the DCD instrument electrical diagram (Figure 7.6-5), the corrected bypass valve closure trip of the offgas post-treatment radiation monitor now states that it is initiated by the High-High Alarm. The High-High Alarm for the gland seal condenser exhaust was added to be consistent with DCD Figure 7.6-5.

Clarification of the Location of Radiation Monitors

In FSAR Subsection 11.5.2.1.5, the departure retains the location of the radiation monitor in the control room for display, recording, and annunciation. In other subsections of Section 11.5, the departure deletes the specific location of the radiation monitor. Both SRP Section 11.5 and RG 1.206 provide guidance implying that the system description should include the location of equipment. NRC staff issued RAI 11.05-3 requesting the applicant to provide the location of the radiation monitors and sufficient specific information for the staff to complete the evaluation. The RAI asked the applicant to provide an independent confirmation of compliance with NRC regulations and guidance, or to justify the deletion of the radiation monitor location. The applicant’s response to this RAI dated September 9, 2009 (ML092530686), deletes “in the control room” from FSAR Tier 2 Subsection 11.5.2.1.5 and adds a footnote in Table 11.5-1 to clarify that although the radiation monitors are not in the control room, the alarms and indication for these radiation detectors are displayed locally and in the main control room. The staff noted that Table 11.5-1 also provides the location of the radiation detectors. These modifications resolved the inconsistency. The staff has verified that the FSAR has been updated to reflect the above changes, and RAI 11.05-3 is closed.

Identification of the “main control inoperative room”

Also in Subsection 11.5.2.2.4, the departure adds the statement “These trip outputs are alarmed in the main control inoperative room.” NRC staff issued RAI 11.05-4 requesting the applicant to identify where the “main control inoperative room” is located or to correct the text. The applicant’s response to this RAI dated September 9, 2009, corrects the phrase by deleting the word “inoperative.” The staff has verified that the FSAR has been updated to reflect the above changes, RAI-11.05-4 is therefore closed.

Radiation Effluent Monitor Calibrations

The applicant also states in STD DEP 11.5-1 in FSAR Subsection 11.5.5.2 that liquid and gaseous radiation process monitors may be calibrated using two methods. The first method uses certified commercial radionuclide sources traceable to the National Institute of Standards and Technology (NIST). The second method analyzes particulate iodine or gaseous grab samples with lab instrumentation. NRC staff found it acceptable to calibrate radiation process monitors using traceable NIST radionuclide sources, but the staff required additional information regarding the use of grab samples with lab instrumentation. Accordingly, the staff issued RAI 11-05 requesting the applicant to expand or revise FSAR Subsection 11.5.5.2, which discusses the calibration of radiation effluent process monitors.

The applicant’s response to this RAI dated September 3, 2009 (ML092510039), clarifies the intent to calibrate applicable radiation effluent monitors using applicable liquid or gaseous certified commercial radionuclide sources traceable to NIST. The applicant also states that grab samples are used for periodic in-service calibrations only and may only be used as a check on the calibration of a PRMS. The applicant further specifies that grab samples do not suffice as an actual acceptable regulatory calibration per RGs 1.21 and 4.15 and includes a COL change to this effect. The staff reviewed this response and determined that the change made to eliminate the applicant’s intent to use plant samples and plant analyses as calibrations is acceptable. The staff has verified that the FSAR has been updated to reflect the above changes, RAI 11-05 is therefore closed.

The applicant’s response to RAI 11-05 changes the calibration frequency of effluent radiation monitors during plant operations, during a plant shutdown, or during the refueling outage. The applicant changed the wording in the FSAR to state that “Each continuous monitor is calibrated during plant shutdown or during the refueling outage if the detector is not accessible during power operation.” The STP FSAR had stated “plant operation” and not plant shutdown. The staff issued RAI 11.05-7 requesting the applicant to clarify this change. The applicant’s response to this RAI dated January 25, 2010 (ML100290012), modifies Subsection 11.5.5.2, “Calibrations,” to state that they do not intend to use plant samples for calibrations, and calibrations of continuous radiation monitors will take place “during plant operation or during the refueling outage if the detector is not accessible during power operation.” The staff reviewed this response and found it acceptable. The staff has verified that the FSAR has been updated to reflect the above changes, RAI 11.05-7 is therefore closed.

Supplemental Information Section 11.5.7S

Supplemental Information in Section 11.5.7S states that an NRC-approved ODCM exists for the STP Units 1 and 2 nuclear power plant on the same site. This supplement briefly summarizes the content of the STP Units 1 and 2 ODCM. The applicant states that the ODCM for STP Units 3 and 4 will be integrated into the STP Units 1 and 2 ODCM and will take into account the

appropriate differences between the existing and the new units. NRC staff issued RAI 11.05-1 and RAI 11.05-5 requesting the applicant to provide additional details on the ODCM integration and whether it will be completed before fuel loading. The applicant's clarification states the intent to maintain a site-wide ODCM and where possible, the ODCM will be aligned with NEI 07-09A (Revision 0), "Generic FSAR Template Guidance for ODCM Program Description." Upon further discussion with the staff, the applicant issued a supplemental response to RAI 11.05-1 in a letter dated November 16, 2009 (ML093220188), stating that STP Units 3 and 4 will have their own ODCM and will incorporate NEI 07-09A template relating to BWR plants. The staff determined that because the applicant has committed to use NEI 07-9A. The staff has verified that the FSAR has been updated to reflect the above changes, RAI 11.05-05 and RAI 11.05-1 are closed.

COL License Information Items

- COL License Information Item 11.4 Calculation of Radiation Release Rates

ABWR DCD Tier 2 Subsection 11.5.6.1, "COL License Information," states that the COL applicant shall provide the COL license information item that applies on a plant-specific basis. For the STP COL application, this COL license information item is addressed as follows:

DCD COL License Information Item 11.4 states, "The COL applicant shall provide and describe in the operation and maintenance manual the procedures and/or methods for the conversion of the radiation measurements into release rates of gaseous discharge from the main plant stack."

The applicant states that the ODCM contains the methodology and parameters used to calculate offsite doses resulting from gaseous and liquid effluents. The ODCM includes methods for converting radiation measurements of gaseous discharge from the main plant stack into release rates.

NRC staff found this information acceptable because the guidance in NEI 07-9A (Revision 0), "Generic FSAR Template Guidance for ODCM Program Description," addresses this issue. Therefore, the applicant has adequately addressed COL License Information Item 11.4.

- COL License Information Item 11.5 Compliance with the Regulatory Shielding Design Basis

ABWR DCD Tier 2 Subsection 11.5.6.2, "COL License Information," states that the COL applicant shall provide the COL license information item that applies on a plant-specific basis. For the STP COL application, this COL license information item is addressed as follows:

DCD COL License Information Item 11.5 states, "The COL applicant shall describe in the operation and maintenance manual the sampling system design of the Standby Gas Treatment System (SGTS) and of the main stack effluent monitoring subsystems and show compliance with the regulatory shielding requirements for low-radiation exposure under accident conditions as stipulated in NUREG-0737, Item II.F.1, clarification 2 of Attachment 2. The requirement for the shielding design will be covered in the equipment design specifications."

The applicant commits (COM 11.5-1) to implement the operation of the sampling system for the SGTS and the operation of the main stack effluent monitoring using operation and maintenance procedures that demonstrate compliance with the regulatory shielding requirements for low-radiation exposure under accident conditions, as stipulated in NUREG-0737, "Clarification

of TMI Action Plan Requirements,” Item II.F.1, clarification 2 of Attachment 2. The requirement for the shielding design will be covered in the equipment design and procurement specifications.

NRC staff found this commitment acceptable and COL License Information Item 11.5 adequately addressed because these operation and maintenance procedures will be based on recognized and proven quality guidance from NUREG-0737 that can be confirmed by both inspection of the procedures and observation of the system operations, with the results confirmed later by NRC inspections and audits.

- COL License Information Item 11.6 Provisions for Isokinetic Sampling

ABWR DCD Tier 2 Subsection 11.5.6.3, “COL License Information,” states that the COL applicant shall provide the COL license information item that applies on a plant-specific basis. For the STP COL application, this COL license information item is addressed as follows:

DCD COL License Information Item 11.6 states, “The COL applicant shall describe in the operation and maintenance manual the sampling technique used for monitoring and sampling of effluent gases to assure that a representative gas sample is extracted and that the sampling system is capable of maintaining isokinetic conditions within 20 percent of the flow rate during and following an accident as stipulated in NUREG-0737, Item II.F.1, clarification 3 of Attachment 2.”

The applicant commits (COM 11.5-2) to develop procedures before fuel loading that include the collection techniques used to extract representative samples of radioactive iodine and particulates under accident conditions. These collection and sampling procedures require a sampling system that is capable of maintaining isokinetic conditions within 20 percent of the flow rate during and following an accident, as stipulated in NUREG-0737 Item II.F.1, clarification 3 of Attachment 2. These procedures will be developed in accordance with the plant operating procedure development plan in Section 13.5.

The staff found this commitment acceptable and COL License Information Item 11.6 adequately addressed because an identifiable time period is specified, and these collection and sampling procedures will be based on recognized and proven quality guidance from NUREG-0737, which can be confirmed by both inspection of the procedures and observation of the sampling system operations, with results confirmed later by NRC inspections and audits.

- COL License Information Item 11.7 Sampling of Radioactive Iodine and Particulates

ABWR DCD Tier 2 Subsection 11.5.6.4, “COL License Information,” states that the COL applicant shall provide the COL license information item that applies on a plant-specific basis. For the STP COL application, this COL license information item is addressed as follows:

DCD COL License Information Item 11.7 states, “The COL applicant shall describe in the operation and maintenance manual the collection technique used to extract representative samples of radioactive iodine and particulates during and following an accident. These measurements are used to determine the quantitative releases for dose calculations and assessment (as stipulated in NUREG-0737, Table II.F.1-2).”

The applicant commits (COM 11.5-3) to develop procedures before fuel loading that include the collection technique used to extract representative samples of radioactive iodines and particulates during and following an accident. These measurements are used to determine the

quantitative releases for dose calculations and assessments (as stipulated in NUREG-0737, Table II.F.1-2). These procedures will be developed in accordance with the plant operating procedure development plan in Section 13.5.

NRC staff found this commitment acceptable and COL License Information Item 11.7 adequately addressed because an identifiable time period is specified, and these collection and sampling procedures will be based on recognized and proven quality guidance from NUREG-0737, which NRC inspections and audits can confirm later by inspecting the procedures.

- COL License Information Item 11.8 Calibration Frequencies and Techniques

ABWR DCD Tier 2 Section 11.5.6.5, "COL License Information," states that the COL applicant shall provide the COL license information item that applies on a plant-specific basis. For the STP COL application, this COL license information item is addressed as follows:

DCD COL License Information Item 11.8 states, "The COL applicant shall provide in the operation and maintenance manual for the system the calibration frequencies and techniques for the radiation sensors. This information shall be based on vendor data for the equipment."

The applicant commits (COM 11.5-4) to develop procedures before fuel loading that specify the calibration frequencies and techniques for the radiation sensors. This information is to be based on vendor data for the equipment. These procedures will be developed in accordance with the plant operating procedure development plan in Section 13.5.

NRC staff found this commitment acceptable and COL License Information Item 11.8 adequately addressed because an identifiable time period is specified and these operation, maintenance, and calibration procedures will be developed in accordance with the plant operating procedure development plan, which NRC inspections and audits can confirm later by inspecting the procedures.

Supplemental Information

Preoperational Tests

In COL FSAR Subsection 14.2.12.1.23, "Process Radiation Monitoring System Preoperational Tests," the applicant states that the PRMS will be tested during the Preoperational Test Program. The PRMS equipment will be performance tested to demonstrate conformance with design process capabilities. An integrity test will be performed on the system upon completion. NRC staff found that the applicant's statement conforms to the guidance in SRP 14.2.

Inspections, Tests, Analyses, and Acceptance Criteria

All ITAAC are incorporated by reference with no changes.

11.5.5 Post Combined License Activities

Verification of Compliance with COM 11.5-1 through COM 11.5-4

The applicant identifies commitments COM 11.5-1 through COM 11.5-4 as addressing COL License Information Items 11.5 through 11.8, respectively.

License Condition for the ODCM before Fuel Loading

NRC staff includes the following two license conditions:

1. "Prior to fuel loading, the licensee shall implement an operational program for process and effluent monitoring and sampling." The program shall include the following subprograms and documents:
 - a. Radiological Effluent Technical Specifications/Standard Radiological Effluent Controls
 - b. Offsite Dose Calculation Manual
 - c. Radiological Environmental Monitoring Program
 - d. Process Control Program
2. The licensee shall submit to the NRC a schedule no later than 12 months after the issuance of the combined operating license that supports planning for the conduct of NRC inspections of the four operating programs and documents listed in the above license condition (number 1). The schedule shall be updated every 6 months until 12 months before the scheduled fuel loading and every month thereafter, until either the four operational programs and documents have been fully implemented or the plant has been placed in commercial service.

11.5.6 Conclusion

The NRC staff's finding related to information incorporated by reference is in NUREG-1503. Pursuant to 10 CFR 52.63(a)(5) and Part 52 Appendix A Section VI.B.1, all nuclear safety issues relating to the PRMS that were incorporated by reference have been resolved.

The staff reviewed the STP Units 3 and 4 COL FSAR and checked the referenced ABWR DCD. The staff's review confirmed that the application has addressed the relevant information, and no outstanding information is expected to be addressed in the COL FSAR related to this section.

The staff compared the additional COL information in the application to the relevant NRC regulations, the acceptance criteria in NUREG-0800 Section 11.5, and other NRC regulatory guides. The staff concluded that the applicant is in compliance with the regulations.

The staff also concluded that the design and operation of the process and effluent radiological monitoring systems include the equipment necessary to monitor process and effluent streams and to control releases of radioactive materials associated with the operation of the LWMS, GWMS, and SWMS. The staff found that the process and effluent radiological monitoring system designs are acceptable and meet the applicable requirements of 10 CFR 20.1301, 10 CFR 20.1301(e), 10 CFR 20.1302, and 10 CFR 20.1406; 10 CFR 50.34a and 10 CFR 50.36a; 10 CFR 50.34(f)(2)(xvii) and 10 CFR 50.34(f)(2)(xxvii); 10 CFR Part 50, Appendix I design objectives; and GDC 60, 63, and 64.

This conclusion is based on the following:

1. The PRMS includes the instrumentation for monitoring and sampling radioactivity for contaminated liquid, gaseous, and solid waste process and effluent streams under a broad range of routine operations and AOOs. The staff evaluated the provisions proposed for sampling and monitoring all appropriate process streams and effluent

release points, including nonradioactive systems that could become contaminated through interfaces with radioactive systems.

2. The applicant meets the requirements of GDC 60 and 61 with respect to controlling releases of radioactive materials into the environment, and associated offsite doses to members of the public, by assuring that the PRMS design includes features to monitor and control releases of radioactive materials in liquid and gaseous effluents. As demonstrated in FSAR Subsection 12.2.2.2.2 using site-specific conditions, all releases are conducted via the LWMS and GWMS and comply with the design objectives of Appendix I to Part 50 and the requirements of 10 CFR 20.1301, 1301(e), and 1302.
3. The applicant meets the requirements of 10 CFR 50.34(f)(2)(xvii) and 10 CFR 50.34(f)(2)(xxvii), as they relate to monitoring gaseous effluents from all potential release points under a broad range of routine and accident conditions.
4. The applicant's proposed ODCM for STP Units 3 and 4, and operating procedures as they relate to controlling and monitoring effluent releases and doses to members of the public, meet the requirements of Appendix I to Part 50 and 10 CFR 20.1301, 10 CFR 20.1301(e), and 10 CFR 20.1302. The staff concluded that the endorsement of NEI ODCM Template 07-09 and the PRMS supplemental information in FSAR Section 11.5 are consistent with the requirements of Generic Letter 89-01 and Radiological Assessment BTP (Revision 1, November 1979); and the guidance of NUREG-1302 for BWR plants; NUREG-0133; and RGs 1.21, 1.33, 4.1, 4.8, and 4.15.
5. The applicant demonstrates compliance with the requirements of 10 CFR 52.80(a) for the PRMS. If the inspections, tests, and analyses are performed as required, and the respective acceptance criteria are met, there will be reasonable assurance that the PRMS has been constructed and will be operated in accordance with the license, the Atomic Energy Act, and NRC regulations.