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52-026

ND-11-0304

U.S. Nuclear Regulatory Commission
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Southern Nuclear Operating Company
Vogtle Electric Generating Plant Units 3 and 4 Combined License Application
Submittal No. 7 Roadmap

Ladies and Gentlemen:

By letter dated March 28, 2008, Southern Nuclear Operating Company (SNC) submitted an application for combined licenses (COLs) for proposed Vogtle Electric Generating Plant (VEGP) Units 3 and 4 to the U.S. Nuclear Regulatory Commission (NRC) for two Westinghouse AP1 000 reactor plants, in accordance with 10 CFR Part 52. By letter dated January 31, 2011, SNC submitted an update (COL Application Submittal No. 7) to the VEGP Units 3 and 4 COL Application. In the January 31 letter, SNC committed to providing a document identifying each COL application change, as an aid to the NRC reviewers, in a separate letter. This letter's enclosure is such a document, a "roadmap" of changes included in the Submittal No. 7 update, along with an explanation of the information contained in the roadmap.

The SNC licensing contacts for this application are W. A. Sparkman at (205) 992-5061 and A. G. Aughtman at (205) 992-5805.

Respectfully submitted,

A handwritten signature in black ink, appearing to read "B. L. Ivey", written over a horizontal line.

B. L. Ivey

BLI/DMW

Enclosure: Vogtle Electric Generating Plant Units 3 and 4 COL Application Submittal No. 7
Roadmap

DO92
NRC

cc: Southern Nuclear Operating Company

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Southern Nuclear Operating Company

ND-11-0304

Enclosure

Vogtle Electric Generating Plant Units 3 and 4

COL Application

Submittal No. 7 Roadmap

NOTE: The enclosed document contains a one (1) page format description and a twenty (20) page roadmap.

VEGP Units 3 & 4 COL Application Submittal 7 Roadmap

Format Explanation (by columns)

- Change ID # [unique identifier for tracking purposes]
- COLA Part A [Part 1 (Pt 01) through 11 (Pt 11)]
- COLA Chapter A [e.g., FSAR 01 to FSAR 19] {generally used only for Part 2}
- Section / Page A [page numbers (if identified) are specific to document to be Revised]
- Change Summary [Short description of change...]
- Basis for Change [the Source of the change...]

NuStart's COLA Tracking Management (CTM) : COLA Changes | SNC VEGP COLA Roadmap of Submittal 7 Update

FEB-11-2011 5:04 PM

SNC VEGP COLA Roadmap of Submittal 7 Update					COLA REP contains 'VEGP' AND ...
QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
Pt 01 - (empty)					4 COLA Changes
8455	Pt 01		01.00	COLA Part 1, Section 1.0, General Information, will be revised to add the following sentence at the end of the paragraph: All organizational and financial information included in this part of the COL Application is current through May 22, 2009, unless otherwise noted.	Status declaration
8350	Pt 01		01.01.04	COLA Part 1, Subsection 1.1.4, Requested Licenses and Authorized Uses, will be revised from: Throughout this application, the "referenced DCD" is the AP1000 DCD submitted by Westinghouse as Revision 17. To read: Throughout this application, the "referenced DCD" is the AP1000 DCD submitted by Westinghouse as Revision 18.	DCD Rev 18
8351	Pt 01		01.07	COLA Part 1, Section 1.7, Reference 1.7-2, will be revised from: Westinghouse Electric Company, 2008, "AP1000 Design Control Document," APP-GW-GL-700, Revision 17 [ADAMS Accession No. ML083230166]. To read: Westinghouse Electric Company, 2010, "AP1000 Design Control Document," APP-GW-GL-700, Revision 18 [ADAMS Accession No. ML103480572].	DCD Rev 18
8457	Pt 01		01D	COLA Part 1, Appendix 1D, first paragraph, will be revised to add the following sentence at the end of the paragraph: All information in this Appendix 1D of the COL Application is current through December 11, 2009.	Status declaration
Pt 02 - FSAR 01					27 COLA Changes
8221	Pt 02	FSAR 01	01.01	1. COLA Part 2, FSAR Chapter 1, Section 1.1, Introduction, will be revised to read: Unless otherwise specified, reference to the DCD refers to Tier 2 information and includes the sensitive unclassified non-safeguards information (including proprietary information), and safeguards information referenced in the AP1000 DCD. Such DCD information is included in this combined license application in the same manner as it is included in the AP1000 DCD, i.e., references in the DCD are included as references in the FSAR, and material incorporated by reference into the DCD is incorporated by reference into the FSAR. Appropriate agreements are in place to provide access to the withheld sensitive unclassified non-safeguards information (including proprietary information), and safeguards information referenced in the AP1000 DCD.	VEGP-VOL-CH01 IBR of PI & SGI response item 1 SNC Ltr ND-10-2207
8348	Pt 02	FSAR 01	01.01	COLA Part 2, FSAR Chapter 1, Section 1.1, Introduction, will be revised from: Throughout this FSAR, the "referenced DCD" is the AP1000 DCD submitted by Westinghouse as Revision 17 including any supplemental material as identified in Table 1.6-201. To read: Throughout this FSAR, the "referenced DCD" is the AP1000 DCD submitted by Westinghouse as Revision 18 including any supplemental material as identified in Table 1.6-201.	DCD Rev 18
8725	Pt 02	FSAR 01	01.02F / F1.2-201	Revise COLA Part 2, FSAR Chapter 1, Section 1.2, Figure 1.2-201, Title block to match DCD Figure 1.2-18 title block from: Annex Building General Arrangement Plant at Elevation 100'-0" and 107'-2" To read: Annex Building General Arrangement Plan at Elevation 100'-0" & 107'-2"	Editorial
8349	Pt 02	FSAR 01	01.06.T / T1.6-201	COLA Part 2, FSAR Chapter 1, Section 1.6, Table 1.6-201, will be revised from: Westinghouse/APP-GW-GL-700 AP1000 Design Control Document 17 All September 2008 ML083230868 To read: Westinghouse/APP-GW-GL-700 AP1000 Design Control Document 18 All December 2010	DCD Rev 18

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
				ML103480572	
8750	Pt 02	FSAR 01	01.06.T / T1.6-202 03.08.05	COLA Part 2, FSAR Chapter 1, Section 1.6, Table 1.6-202, item 3.8.5, will be revised from: Additionally, the first sentence of the second paragraph in ESPA SSAR Subsection 3.8.5.1.1 is not incorporated by reference. To read: Additionally, the first paragraph in ESPA SSAR Subsection 3.8.5.1.1 is not incorporated by reference.	DCD Rev 18 vs VEGP ESP
8751	Pt 02	FSAR 01	01.08.T / T1.8-201 03.04-1	COLA Part 2, FSAR Chapter 1, Table 1.8-201, Summary of FSAR Departures from the DCD, will be revised to add the following line item: VEGP DEP 3.4-1 An alternate waterproofing system for the seismic Category I structures below grade is as presented in the ESPA SSAR. 3.4.1.1.1.1	DCD Rev 18 vs VEGP ESP
8028	Pt 02	FSAR 01	01.08.T / T1.8-201 08.03-1.	3. COLA Part 2, FSAR Chapter 1, Table 1.8-201, Summary of FSAR Departures from the DCD, will be revised to add the following: Departure Number Departure Description Summary FSAR Section or Subsection STD DEP 8.3-1 The Class 1E voltage regulating transformers do not have active components to limit current. 8.3.2.2	VEGP-VOL-CH08 response to STD-VOL-08.03-002 item 3 SNC Ltr ND-10-2005
8752	Pt 02	FSAR 01	01.08.T / T1.8-202 02.05-17	COLA Part 2, FSAR Chapter 1, Section 1.8, Table 1.8-202, COL item 2.5-17, will be revised to add the following FSAR Section(s) as new cross-references: 3.4.1.1.1.1 3.8.5.1	DCD Rev 18 vs VEGP ESP
7784	Pt 02	FSAR 01	01.08.T / T1.8-202 03.08-05	1. COLA Part 2, FSAR Chapter 1, Section 1.8, Table 1.8-202, will be revised to add new COL item listing to read: 3.8-5 Structures Inspection Program 3.8.6.5 3.8.3.7 A 3.8.4.7 3.8.5.7 3.8.6.5 17.6	VEGP-VOL-CH03 SIP response to STD COL 03.08-005 item 1 SNC Ltr ND-10-1594
7935	Pt 02	FSAR 01	01.08.T / T1.8-202 03.08-06	1. COLA Part 2, FSAR Chapter 1, Section 1.8, Table 1.8-202, will be revised to add new COL item listing to read: 3.8-6 Construction Procedures Program 3.8.6.6 3.8.6.6 H	VEGP-VOL-CH03 Const Procedures response to STD-COL-03.08-006 item 1 SNC Ltr ND-10-1900
7748	Pt 02	FSAR 01	01.08.T / T1.8-202 05.02-03	1. COLA Part 2, FSAR Chapter 1, Table 1.8-202, will be revised to add a new COL information item to read: 5.2-3 Response to Unidentified Reactor Coolant System Leakage Inside Containment 5.2.6.3 5.2.6.3 A 5.2.5.3.5	VEGP-RAI-LTR-060 in response to RAI 05.02.05-001 item 1 SNC Ltr ND-10-1423
7801	Pt 02	FSAR 01	01.08.T / T1.8-202 05.03-07	1. COLA Part 2, FSAR Chapter 1, Section 1.8, Table 1.8-202, will be revised to add new COL item listing to read: 5.3-7 Quickloc Weld Build-up ISI 5.3.6.6 5.2.4.1 A 5.3.6.6	VEGP-VOL-CH05 ISI response to STD COL 05.03-007 item 1 SNC Ltr ND-10-1656
7695	Pt 02	FSAR 01	01.08.T / T1.8-202 18.02-02	4. COLA Part 2, FSAR Chapter 1, Table 1.8-202, will be revised to add FSAR Subsection 9.5.2.2.5 to COL Item 18.2-2.	VEGP-VOL-Ch 18 re EOF item 4 SNC Ltr ND-10-1371
7875	Pt 02	FSAR 01	01.08.T / T1.8-202 19.59.10-6	4. COLA Part 2, FSAR Chapter 1, Table 1.8-202, will be revised to add new COL Item Number 19.59.10-6 as shown below: COL APPLICANT (A), HOLDER (H), OR BOTH (B) COL ITEM SUBJECT DCD SUBSECTION FSAR SUBSECTION(S) 19.59.10-6 Confirm that the Seismic Margin Assessment analysis is applicable to the COL site 19.59.10.5 19.55.6.3 19.59.10.5 A	VEGP-VOL-CH19 PRA item 4 SNC Ltr ND-10-1811
8440	Pt 02	FSAR 01	01.09.T / T1.9-201 1.011	COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, is revised to include Regulatory Guide 1.11 to read: 1.11 Instrument Lines Penetrating the Primary Reactor Containment DCD discussion only; See DCD Table 1.9-1	DCD Rev 18

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
				(Rev. 1, March 2010)	
8223	Pt 02	FSAR 01	01.09.T / T1.9-201 1.028	1. COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, information for Regulatory Guide 1.28, will be revised to add the following additional FSAR section reference: 14.2.2.2	VEGP-VOL-CH14 Qualification Req response item 1 SNC Ltr ND-10-2204
7785	Pt 02	FSAR 01	01.09.T / T1.9-201 1.160	2. COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Regulatory Guide 1.160 will be revised to add the following new FSAR Subsection references prior to the existing FSAR Section reference of 17.6 (NEI 07-02A): 3.8.3.7 3.8.4.7 3.8.5.7	VEGP-VOL-CH03 SIP response to STD COL 03.08-005 item 2 SNC Ltr ND-10-1594
7786	Pt 02	FSAR 01	01.09.T / T1.9-201 1.199	3. COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Regulatory Guide 1.199 will be revised to read: DCD discussion only; See DCD Table 1.9-1	VEGP-VOL-CH03 SIP response to STD COL 03.08-005 item 3 SNC Ltr ND-10-1594
8383	Pt 02	FSAR 01	01.09.T / T1.9-204 Sh02	1. COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-204, Bulletin Number 05-01, Material Control and Accounting at Reactors and Wet Spent Fuel Storage Facilities, will be revised to read: Number Title Comment 05-01 Material Control and Accounting at Reactors and Wet Spent Fuel Storage Facilities 13.5.2.2.9	VEGP-RAI-LTR-064 response to RAI 01.05-003 item 1 SNC Ltr ND-10-2257.
8139	Pt 02	FSAR 01	01.10.02	1. COLA Part 2, FSAR Chapter 1, Subsection 1.10.2, last paragraph, will be revised from: This assessment identified administrative and managerial controls to avoid impacts to SSCs from construction. The results of the assessment are presented in Table 1.10-202. To read: The initial assessment consisted of a review of individual SSCs and LCOs to determine whether an item is applicable, or may be eliminated due to either examination or being internal and specific to an operating unit. The assessment identified the SSCs that could reasonably be expected to be impacted by construction activities unless administrative and managerial controls are established. The results of the assessment are presented in Table 1.10-202. Periodic assessment during construction is addressed in Appendix 13AA, Subsection 13AA.1.1.1.1.8	VEGP-RAI-LTR-063 response to RAI 01.05-002 item 1 SNC Ltr ND-10-2114
8140	Pt 02	FSAR 01	01.10.03	2. COLA Part 2, FSAR Chapter 1, Subsection 1.10.3, last paragraph, will be revised to read: The above discussed controls to eliminate or mitigate construction hazards that could potentially impact operating unit SSCs important to safety are in place when there is an operating nuclear unit on the site. Additional controls may be established during construction as addressed in Appendix 13AA, Subsection 13AA.1.1.1.1.8.	VEGP-RAI-LTR-063 response to RAI 01.05-002 item 2 SNC Ltr ND-10-2114
8143	Pt 02	FSAR 01	01.10.T / T1.10-201	5. COLA Part 2, FSAR Chapter 1, Subsection 1.10, Table 1.10-202, will be revised to read: Equipment and Material Laydown, Storage, Warehousing • Releases of Flammable, Hazardous or Toxic Materials	VEGP-RAI-LTR-063 response to RAI 01.05-002 item 5 SNC Ltr ND-10-2114 (Note that this change actually affects Table 1.10-201, not 202.)
8142	Pt 02	FSAR 01	01.10.T / T1.10-202	4. COLA Part 2, FSAR Chapter 1, Subsection 1.10, Table 1.10-202, will be revised to include the following new item: Impact of Local Flooding • Safety-related structures, systems, and components (SSCs)	VEGP-RAI-LTR-063 response to RAI 01.05-002 item 4 SNC Ltr ND-10-2114
8144	Pt 02	FSAR 01	01.10.T / T1.10-203	6. COLA Part 2, FSAR Chapter 1, Subsection 1.10, Table 1.10-203, will be revised to include the following new item: Impact of Local Flooding • Site grading and drainage provisions consider potential flooding impacts from local intense precipitation	VEGP-RAI-LTR-063 response to RAI 01.05-002 item 6 SNC Ltr ND-10-2114
8145	Pt 02	FSAR 01	01.10.T / T1.10-203	7. COLA Part 2, FSAR Chapter 1, Subsection 1.10, Table 1.10-203, will be revised to include the following new item: Impact of Site Groundwater Dewatering • Administrative controls address groundwater level monitoring	VEGP-RAI-LTR-063 response to RAI 01.05-002 item 7 SNC Ltr ND-10-2114
8441	Pt 02	FSAR 01	01AA 1.011	COLA Part 2, FSAR Chapter 1, Appendix 1AA, is revised to include Regulatory Guide 1.11 to read: Regulatory Guide 1.11, Rev. 1, 3/10 – Instrument Lines Penetrating the Primary Reactor Containment	DCD Rev 18

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
				Conformance with the design aspects is as stated in the DCD. This guidance is completely within the scope of the DCD.	
7787	Pt 02	FSAR 01	01AA 1.199	4. COLA Part 2, FSAR Chapter 1, Appendix 1AA, Regulatory Guide 1.199 will be revised to read: Conformance with Revision 0 of the Regulatory Guide is as stated in the DCD. This guidance is completely within the scope of the DCD.	VEGP-VOL-CH03 SIP response to STD COL 03.08-005 Item 4 SNC Ltr ND-10-1594
Pt 02 - FSAR 02					5 COLA Changes
7771	Pt 02	FSAR 02	02.00.T / T2.0-201 Sh01	COLA Part 2, FSAR Chapter 2, Table 2.0-201, will be revised to add the following additional VEGP Reference for Wind Speed - Operating Basis parameter. ESPA SSAR Figure 2.5.1-32	Provide reference for exposure and topographic factor
8541	Pt 02	FSAR 02	02.05.04.13	1. COLA Part 2, FSAR Chapter 2, Subsection 2.5.4.13 will be added (with an LMA of VEGP SUP 2.5-1) to read: 2.5.4.13 Heavy Lift Derrick Counterweight and Ring Foundation The ring foundation for the heavy lift derrick (HLD) and counterweight are abandoned in place below grade following construction of Units 3 and 4. The HLD rails are removed from the ring foundation after construction of Units 3 and 4. The (HLD) counterweight and ring foundation are shown on Figure 2.5-203. The top of the HLD counterweight and ring foundation concrete is located at approximately elevation 215 ft MSL, which is five feet below the nominal site grade of 220 ft MSL. The HLD counterweight and ring foundation are not visible following the installation of the roads, drainage provisions, and ground surface cover. The HLD counterweight and ring foundation are below the surface drainage system provisions and do not affect the runoff for the local PMP flood event discussed in Subsection 2.4.2.3. The HLD counterweight and ring foundation are located above the design ground water elevation of 165 ft MSL, and do not impact the hydrological analyses described in ESPA SSAR Subsections 2.4.12 and 2.4.13. The safety-related portion of the excavations is filled with Category 1 backfill to the NI basemat and with Category 2 backfill to grade. The side slopes are filled with engineered granular backfill (EGB), which is non-safety related and does not affect the static or seismic performance of the safety-related structures. As shown on Figure 2.5-203, the HLD counterweight and ring foundation does not extend into the safety-related backfill of either Unit 3 or Unit 4. The ring foundation does extend into the EGB backfill of the excavations for both Unit 3 and Unit 4. The counterweight overall depth is approximately 28 ft. and is below the EGB backfill of the excavation for Unit 4. Subsection 3.7.1.1.1 provides the results of the evaluation which confirms that the presence of the HLD counterweight and ring foundation has no effect on the site specific seismic analyses.	VEGP-VOL-CH02 re HLD response SNC Letter ND-11-0008
8753	Pt 02	FSAR 02	02.05.07.17	COLA Part 2, FSAR Chapter 2, Subsection 2.5.7.17, will be revised from: This COL item is addressed in Subsection 3.8.5.1. To read: This COL item is addressed in Subsections 3.8.5.1 and 3.4.1.1.1.1.	DCD Rev 18 vs VEGP ESP
8542	Pt 02	FSAR 02	02.05F / F2.5-203	2. COLA Part 2, FSAR Chapter 2, Section 2.5, will be revised (with an LMA VEGP SUP 2.5-1) to include new Figure 2.5-203. (See SNC letter for actual figure.)	VEGP-VOL-CH02 re HLD response SNC Letter ND-11-0008
8754	Pt 02	FSAR 02	03.04.01.01.01.01	COLA Part 2, FSAR Chapter 3, Subsection 3.4.1.1.1.1 will be added (with LMAs of VEGP DEP 3.4-1 and VEGP COL 2.5-17) to read: 3.4.1.1.1.1 Waterproofing Add the following text to the end of the fourth bullet of the first paragraph of DCD Subsection 3.4.1.1.1.1. An alternate waterproofing system for the seismic Category I structures below grade is as presented in ESPA SSAR Subsection 3.8.5.1.1.	DCD Rev 18 vs VEGP ESP
Pt 02 - FSAR 03					12 COLA Changes
8543	Pt 02	FSAR 03	03.07.01.01	3. COLA Part 2, FSAR Chapter 3, will be revised to add as third and fourth paragraph under Subsection 3.7.1.1.1 (with an LMA of VEGP SUP 2.5-1) to read: As discussed in Subsection 2.5.4.13, the heavy lift derrick (HLD) counterweight and ring foundation were abandoned in place after construction. The HLD counterweight is outside the defined excavation of Unit 3 and Unit 4 and therefore does not need to be evaluated. Portions of the HLD ring foundation extend over the Unit 3 and Unit 4 excavation slopes within the engineered granular backfill (EGB); but outside the Category 1 and 2 backfill. The presence of the HLD ring foundation has no effect on the VEGP site-specific 3D SASSI SSI analyses of the Nuclear Island (NI) presented in Appendix 3GG based on the following information.	VEGP-VOL-CH02 re HLD response SNC Letter ND-11-0008

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
				The VEGP site-specific 3D SASSI SSI of the NI is consistent with the accepted DCD 3D SASSI NI modeling approach of not including structure-to-structure interaction of the adjacent structures such as the Annex Building and the Turbine Building, and therefore the more distant abandoned HLD ring foundation has even less structure-to-structure effects on the NI seismic response. Additionally only a portion of the abandoned HLD ring foundation is within a limited area of the non-safety EGB over the slopes of the excavation. It has been demonstrated in the ESP as amended that a large variation of the EGB properties does not significantly affect the site-specific seismic analyses; therefore, it is concluded the abandoned portion of the HLD ring foundation in the EGB has no significant effect on the site-specific seismic analyses.	
8015	Pt 02	FSAR 03	03.07.04.04	<p>1. COLA Part 2, FSAR Chapter 3, Subsection 3.7.4.4, will be revised to read:</p> <p>VEGP COL 3.7-2 Post-earthquake operating procedures utilize the guidance of EPRI Reports NP5930, TR-100082, and NP-6695, as modified and endorsed by the NRC in Regulatory Guides 1.166 and 1.167. A response spectrum check up to 10Hz and the cumulative absolute velocity will be calculated based on the recorded motions at the free field instrument. If the operating basis earthquake ground motion is exceeded or significant plant damage occurs, the plant must be shutdown in an orderly manner.</p> <p>STD COL 3.7-2 In addition, the procedures address measurement of the post-seismic event gaps between the new fuel rack and walls of the new fuel storage pit, between the individual spent fuel racks, and from the spent fuel racks to the spent fuel pool walls, and provide for appropriate corrective actions to be taken if needed (such as repositioning the racks or analysis of the as-found condition).</p>	VEGP-VOL-CH03 Seismic Instrumentation Information response to COL-03.07-002 Item 1 SNC Ltr ND-10-2001
8016	Pt 02	FSAR 03	03.07.05.02	<p>2. COLA Part 2, FSAR Chapter 3, Subsection 3.7.5.2, will be revised to read:</p> <p>VEGP COL 3.7-2 This COL Item is addressed in Subsection 3.7.4.4.</p> <p>STD COL 3.7-2</p>	VEGP-VOL-CH03 Seismic Instrumentation Information response to COL-03.07-002 Item 2 SNC Ltr ND-10-2001
7788	Pt 02	FSAR 03	03.08.03.07	<p>5. COLA Part 2, FSAR Chapter 3, will be revised to add new Subsection 3.8.3.7 (with an LMA of STD COL 3.8-5) to read:</p> <p>-----</p> <p>3.8.3.7 In-Service Testing and Inspection Requirements</p> <p>-----</p> <p>Replace the existing DCD statement with the following:</p> <p>The inspection program for structures is identified in Section 17.6. This inspection program is consistent with the requirements of 10 CFR 50.65 and the guidance in Regulatory Guide 1.160.</p>	VEGP-VOL-Ch03 SIP response to STD COL 03.08-005 item 5 SNC Ltr ND-10-1594
7789	Pt 02	FSAR 03	03.08.04.07	<p>6. COLA Part 2, FSAR Chapter 3, will be revised to add new Subsection 3.8.4.7 (with an LMA of STD COL 3.8-5) to read:</p> <p>-----</p> <p>3.8.4.7 Testing and In-Service Inspection Requirements</p> <p>-----</p> <p>Replace the existing DCD final statement of the subsection with the following:</p> <p>The inspection program for structures is identified in Section 17.6. This inspection program is consistent with the requirements of 10 CFR 50.65 and the guidance in Regulatory Guide 1.160.</p>	VEGP-VOL-Ch03 SIP response to STD COL 03.08-005 item 6 SNC Ltr ND-10-1594
8755	Pt 02	FSAR 03	03.08.05.01	<p>COLA Part 2, FSAR Chapter 3, Subsection 3.8.5.1 (portion with LMAs of VEGP ESP VAR 1.6-2 and VEGP COL 2.5-17) will be revised to read:</p> <p>Subsection 3.8.5 of the referenced ESPA SSAR is incorporated by reference after the last paragraph of DCD Subsection 3.8.5.1 with the following variance:</p> <p>The first paragraph of ESPA SSAR Subsection 3.8.5.1, which pertains to DCD Revision 15, is not incorporated by reference.</p> <p>In addition, the first paragraph in ESPA SSAR Subsection 3.8.5.1.1 also addresses material specific to Revision 15 of the DCD. Therefore, that paragraph is not incorporated by reference.</p>	DCD Rev 18 vs VEGP ESP
7790	Pt 02	FSAR 03	03.08.05.07	<p>7. COLA Part 2, FSAR Chapter 3, will be revised to add new Subsection 3.8.5.7 (with an LMA of STD COL 3.8-5) to read:</p> <p>-----</p> <p>3.8.5.7 In-Service Testing and Inspection Requirements</p> <p>-----</p>	VEGP-VOL-Ch03 SIP response to STD COL 03.08-005 item 7 SNC Ltr ND-10-1594

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
				<p>Replace the existing DCD first statement with the following:</p> <p>The inspection program for structures is identified in Section 17.6. This inspection program is consistent with the requirements of 10 CFR 50.65 and the guidance in Regulatory Guide 1.160.</p>	
7791	Pt 02	FSAR 03	03.08.06.05	<p>8. COLA Part 2, FSAR Chapter 3, will be revised to add new Subsection 3.8.6.5 (with an LMA of STD COL 3.8-5) to read:</p> <p>-----</p> <p>3.8.6.5 Structures Inspection Program</p> <p>-----</p> <p>This item is addressed in Subsections 3.8.3.7, 3.8.4.7, 3.8.5.7, and 17.6.</p>	VEGP-VOL-CH03 SIP response to STD COL 03.08-005 item 8 SNC Ltr ND-10-1594
7936	Pt 02	FSAR 03	03.08.06.06	<p>2. COLA Part 2, FSAR Chapter 3, will be revised to add new Subsection 3.8.6.6 (with an LMA of STD COL 3.8-6) to read:</p> <p>-----</p> <p>3.8.6.6 Construction Procedures Program</p> <p>-----</p> <p>Add the following to the end of Subsection 3.8.6.6:</p> <p>Construction and inspection procedures for concrete filled steel plate modules address activities before and after concrete placement, use of construction mock-ups, and inspection of modules before and after concrete placement as discussed in DCD Subsection 3.8.4.8. The procedures will be made available to NRC inspectors prior to use.</p>	VEGP-VOL-CH03 Const Procedures response to STD-COL-03.08-006 item 2 SNC Ltr ND-10-1900
7762	Pt 02	FSAR 03	03.09.03.01.02	<p>5. COLA Part 2, FSAR Chapter 3, Subsection 3.9.3.1.2, will be revised under the heading of Locations to be Monitored, to read:</p> <p>In addition to the existing permanent plant temperature instrumentation, temperature and displacement monitoring will be included at critical locations on the surge line. The additional locations utilized for monitoring during the hot functional testing and the first fuel cycle (see Subsection 14.2.9.2.22) are selected based on the capability to provide effective monitoring.</p>	VEGP-RAI-LTR 057 S1 response to RAI 03.12-002 item 5 SNC Ltr ND-10-1501
8456	Pt 02	FSAR 03	03.09.06.02.02	<p>Add LMA of STD COL 3.9-4 to the text added in Revision 3 shown below...</p> <p>=====</p> <p>Add the paragraph below as the last paragraph of FSAR Subsection 3.9.6.2.2 prior to the subheading "Check Valve Tests":</p> <p>The attributes of the AOV testing program described above, to the extent that they apply to and can be implemented on other safety-related power-operated valves, such as electro-hydraulic valves, are applied to those other power-operated valves.</p>	Editorial addition of LMA for material added via COL-SER-OI-CH03 response to OI 03.09-04 (SNC Ltr ND-09-2015)
8146	Pt 02	FSAR 03	03GG	APP 3GG - (Refer to the response letter for the complete FSAR App 3GG Mark Up - Enclosure 2)	VEGP-RAI-LTR-018 S2 response to RAI 03.07.02-001 SNC Ltr ND-10-1723
Pt 02 - FSAR 05					4 COLA Changes
7802	Pt 02	FSAR 05	05.02.04.01	<p>2. COLA Part 2, FSAR Chapter 5, will be revised to add the following new paragraph at the end of the portion of Subsection 5.2.4.1 with an LMA of STD COL 5.3-7, to read:</p> <p>The in-service inspection program is augmented to include the performance of a 100 percent volumetric examination of the weld build-up on the reactor vessel head for the instrumentation penetrations (Quickloc) conducted once during each 120-month inspection interval in accordance with the ASME Code, Section XI. The weld build-up acceptance standards are those provided in ASME Code, Section XI, IWB-3514. Personnel performing examinations and the ultrasonic examination systems are qualified in accordance with ASME Code, Section XI, Appendix VIII. Alternatively, an alternative inspection may be developed in conjunction with the voluntary consensus standards bodies (i.e., ASME) and submitted to the NRC for approval.</p>	VEGP-VOL-CH05 ISI response to STD COL 05.03-007 item 2 SNC Ltr ND-10-1656
7749	Pt 02	FSAR 05	05.02.05.03.05	<p>2. COLA Part 2, FSAR Chapter 5, will be revised to add a new Subsection 5.2.5.3.5 (with an LMA of STD COL 5.2-3) to read:</p> <p>-----</p> <p>Add the following new subsection following DCD Subsection 5.2.5.3.4.</p> <p>5.2.5.3.5 Response to Reactor Coolant System Leakage</p> <p>Operating procedures specify operator actions in response to prolonged low level unidentified reactor coolant leakage conditions that exist above normal leakage rates and below the Technical Specification (TS) limits to provide operators sufficient time to take action before the TS limit is</p>	VEGP-RAI-LTR-060 in response to RAI 05.02.05-001 item 2 SNC Ltr ND-10-1423

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
				<p>reached. The procedures include identifying, monitoring, trending, and addressing prolonged low level leakage. The procedures for effective management of leakage, including low level leakage, are developed including the following operations related activities:</p> <ul style="list-style-type: none"> • Trends in the unidentified leakage rates are periodically analyzed. When the leakage rate increases noticeably from the baseline leakage rate, the safety significance of the leak is evaluated. The rate of increase in the leakage is determined to verify that plant actions can be taken before the plant exceeds TS limits. • Procedures are established for responding to leakage. These procedures address the following considerations to prevent adverse safety consequence results from the leakage: <ul style="list-style-type: none"> - Plant procedures specify operator actions in response to leakage rates less than the limits set forth in the Technical Specifications. The procedures include actions for confirming the existence of a leak, identifying its source, increasing the frequency of monitoring, verifying the leakage rate (through a water inventory balance), responding to trends in the leakage rate, performing a walkdown outside containment, planning a containment entry, adjusting alarm setpoints, limiting the amount of time that operation is permitted when the sources of the leakage are unknown, and determining the safety significance of the leakage. - Plant procedures specify the amount of time the leakage detection and monitoring instruments (other than those required by Technical Specifications) may be out of service to effectively monitor the leakage rate during plant operation (i.e.; hot shutdown, hot standby, startup, transients, and power operation). • The output and alarms from leakage monitoring systems are provided in the main control room. Procedures are readily available to the operators for converting the instrument output to a common leakage rate. (Alternatively, these procedures may be part of a computer program so that the operators have a real-time indication of the leakage rate as determined from the output of these monitors.) Periodic calibration and testing of leakage monitoring systems are conducted. The alarm(s), and associated setpoint(s), provide operators an early warning signal so that they can take corrective actions, as discussed above, i.e., before the plant exceeds TS limits. • During maintenance and refueling outages, actions are taken to identify the source of any unidentified leakage that was detected during plant operation. In addition, corrective action is taken to eliminate the condition resulting in the leakage. <p>The procedures described above will be available prior to fuel load.</p>	
7750	Pt 02	FSAR 05	05.02.06.03	<p>3. COLA Part 2, FSAR Chapter 5, will be revised to add a new Subsection 5.2.6.3 (with an LMA of STD COL 5.2-3) to read:</p> <p>-----</p> <p>5.2.6.3 Response to Unidentified Reactor Coolant System Leakage Inside Containment</p> <p>-----</p> <p>This COL item is addressed in Subsection 5.2.5.3.5.</p>	VEGP-RAI-LTR-060 in response to RAI 05.02.05-001 item 3 SNC Ltr ND-10-1423
7803	Pt 02	FSAR 05	05.03.06.06	<p>3. COLA Part 2, FSAR Chapter 5, will be revised to add new Subsection 5.3.6.6 (with an LMA of STD COL 5.3-7) to read:</p> <p>-----</p> <p>5.3.6.6 Quickloc Weld Build-up ISI</p> <p>-----</p> <p>This item is addressed in Subsection 5.2.4.1.</p>	VEGP-VOL-CH05 ISI response to STD COL 05.03-007 item 3 SNC Ltr ND-10-1656
Pt 02 - FSAR 06					9 COLA Changes
7782	Pt 02	FSAR 06	06.01.02.01.06	<p>COLA Part 2, FSAR Chapter 6, Section 6.1.2.1.6, will be revised to include the following new information after the existing fourth paragraph just after the Service Level I and Service Level III discussions (the LMA of STD COL 6.1-2 remains unchanged):</p> <p>Include a new second paragraph under the subsection titled "Service Level II Coatings" within DCD Subsection 6.1.2.1.6 with the following information.</p> <p>Such safety-related Service Level II coatings used inside containment are procured to the same standards as Service Level I coatings with regard to radiation tolerance and performance under design basis accident conditions as discussed below.</p>	VEGP-VOL-Ch06 Coatings in response to STD COL 06.01-002 SNC Ltr ND-10-1566
7923	Pt 02	FSAR 06	06.01.02.01.06	<p>COLA Part 2, FSAR Chapter 6, Section 6.1.2.1.6, as revised per Qb7782 will be revised to remove the term "safety-related" from: Such safety-related Service Level II coatings used inside containment are procured to the same standards as Service Level I coatings with regard to radiation tolerance and performance under design basis accident conditions as discussed below.</p> <p>To read:</p>	Correction to remove "safety-related" designation from changes in VEGP-VOL-Ch06 Coatings in response to STD COL 06.01-002

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
				Such Service Level II coatings used inside containment are procured to the same standards as Service Level I coatings with regard to radiation tolerance and performance under design basis accident conditions as discussed below.	SNC Ltr ND-10-1566. Only the procurement is per Appendix B.
8442	Pt 02	FSAR 06	06.04.07	COLA Part 2, FSAR Chapter 6, Subsection 6.4.7 is revised to include an additional LMA of STD COL 6.4-1.	Consistency to match the LMAs used in the sections where the item is addressed, particularly 6.4.4
7714	Pt 02	FSAR 06	06.04.T / T6.4-201	1. COLA Part 2, FSAR Chapter 6, Section 6.4, standard portion of table of toxic chemical evaluations (VEGP Table 6.4-201) as modified by the response to BLN-RAI-LTR-169 will be further revised in the Standard Onsite Toxic Chemicals listing for the Hydrogen Gas from "Corner of the Auxiliary and Turbine buildings" to read "Yard at turbine building" in the "Evaluated Location" column.	VEGP-VOL-Ch06 re toxic chemicals response to STD-COL-06.04-001 item 1 SNC Ltr ND-10-1473
7715	Pt 02	FSAR 06	06.04.T / T6.4-201	2. COLA Part 2, FSAR Chapter 6, Section 6.4, standard portion of table of toxic chemical evaluations (VEGP Table 6.4-201) as modified by the response to BLN-RAI-LTR-169 will be further revised in the Standard Onsite Toxic Chemicals listing for the Hydrogen Liquid from "2000 gal" to read "1500 gal" in the "Evaluated Maximum Quantity" column.	VEGP-VOL-Ch06 re toxic chemicals response to STD-COL-06.04-001 item 2 SNC Ltr ND-10-1473
7805	Pt 02	FSAR 06	06.04.T / T6.4-201	1. COLA Part 2, FSAR Chapter 6, Section 6.4, Table 6.4-201 Part A, will be revised for the standard chemicals of hydrogen (liquid), nitrogen, and carbon dioxide, to change the Evaluated Minimum Distance to MCR Intake from 814 ft to 577 ft.	VEGP-RAI LTR 061 response to RAI 06.04-005 item 1 SNC Ltr ND-10-1721
7806	Pt 02	FSAR 06	06.04.T / T6.4-201	2. COLA Part 2, FSAR Chapter 6, Section 6.4, Table 6.4-201 Part A, will be revised for the standard chemicals of hydrogen (liquid and gas), nitrogen, and carbon dioxide, to change the MCR Habitability Impact Evaluation from IH to MCR.	VEGP-RAI LTR 061 response to RAI 06.04-005 item 2 SNC Ltr ND-10-1721
7807	Pt 02	FSAR 06	06.04.T / T6.4-201	3. COLA Part 2, FSAR Chapter 6, Section 6.4, Table 6.4-201 Part B, will be revised for the chemicals Methoxypropylamine and Ammonium Bisulfite, to change the MCR Habitability Impact Evaluation from MCR to IH, MCR.	VEGP-RAI LTR 061 response to RAI 06.04-005 item 3 SNC Ltr ND-10-1721
7808	Pt 02	FSAR 06	06.04.T / T6.4-201 Notes	4. COLA Part 2, FSAR Chapter 6, Section 6.4, Table 6.4-201, will be revised to change the footnote for MCR to read: MCR -Chemicals with an Impact Evaluation designation of "MCR" indicates the evaluation of this chemical considered design details of the main control room such as volume, envelope boundaries, ventilation systems, and occupancy factor.	VEGP-RAI LTR 061 response to RAI 06.04-005 item 4 SNC Ltr ND-10-1721
Pt 02 - FSAR 08					2 COLA Changes
8026	Pt 02	FSAR 08	08.03.02.01.04	1. COLA Part 2, FSAR Chapter 8, Subsection 8.3.2.1.4, Maintenance and Testing, will be revised to add the following as the last paragraph with LMA STD COL 8.3-2: Procedures are established for periodic testing of the Class 1E battery chargers and Class 1E voltage regulating transformers in accordance with the manufacturer recommendations. • Circuit breakers in the Class 1E battery chargers and Class 1E voltage regulating transformers that are credited for an isolation function are tested through the use of breaker test equipment. This verification confirms the ability of the circuit to perform the designed coordination and corresponding isolation function between Class 1E and non-Class 1E components. Circuit breaker testing is done as part of the Maintenance Rule program and testing frequency is determined by that program. • Fuses / fuse holders that are included in the isolation circuit are visually inspected. • Class 1E battery chargers are tested to verify current limiting characteristic utilizing manufacturer recommendation and industry practices. Testing frequency is in accordance with that of the associated battery.	VEGP-VOL-CH08 response to STD-VOL-08.03-002 item 1 SNC Ltr ND-10-2005
8027	Pt 02	FSAR 08	08.03.02.02	2. COLA Part 2, FSAR Chapter 8, will be revised to add new Section 8.3.2.2 to read: 8.3.2.2 Analysis STD DEP 8.3-1 Replace the first sentence of the third paragraph of DCD Subsection 8.3.2.2 with the following: The Class 1E battery chargers are designed to limit the input (ac) current to an acceptable value under faulted conditions on the output side, however, the voltage regulating transformers do not have active components to limit current; therefore, the Class 1E voltage regulating transformer maximum current is determined by the impedance of the transformer.	VEGP-VOL-CH08 response to STD-VOL-08.03-002 item 2 SNC Ltr ND-10-2005
Pt 02 - FSAR 09					4 COLA Changes
7694	Pt 02	FSAR 09	09.05.02.02.05	3. COLA Part 2, FSAR Chapter 9, Subsection 9.5.2.2.5, will be revised to add LMA VEGP COL 18.2-2.	VEGP-VOL-Ch 18 re EOF item 3 SNC Ltr ND-10-1371

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
7870	Pt 02	FSAR 09	09.05.04.05.02	COLA Part 2, FSAR Chapter 9, Subsection 9.5.4.5.2, second paragraph, the word "kinetic" will be revised to "kinematic" to match the required ASTM testing.	Editorial - Provide appropriate description of the testing
7804	Pt 02	FSAR 09	09.05.T / T9.5-201 033	COLA Part 2, FSAR Chapter 9, Table 9.5-201, item 33, will be revised under the column Remarks from: Subsection 9.5.1.8.2.2 and DCD Subsection 6.4.3.1 address these requirements. To read: Subsection 9.5.1.8.2.2 and DCD Subsections 6.4.2.3 and 6.4.4 address these requirements.	Provide appropriate reference for control room personnel breathing air source
8726	Pt 02	FSAR 09	09AF / F9A-201	Revise COLA Part 2, FSAR Chapter 9, Appendix 9A, Figure 9A-201, Title block to match DCD Figure 9A-3, Sheet 1 of 3, title block from: Annex I and II Building Fire Areas Plan at Elevation 100'-0" and 107'-2" To read (in italics): [Annex I and II Building Fire Areas Plan at Elevation 100'-0" & 107'-2"]* and to include the * footnote from the DCD.	DCD Rev 18 and Editorial
Pt 02 - FSAR 11				5 COLA Changes	
7812	Pt 02	FSAR 11	11.02.01.02.04	1. COLA Part 2, FSAR Chapter 11, Subsection 11.2.1.2.4, Controlled Release of Radioactivity, third paragraph with LMA of VEGP SUP 11.2-1 will be revised to read: The liquid radwaste system (WLS) exterior discharge piping from the Units 3 and 4 Radwaste Building is buried, stainless steel, enclosed within a guard pipe and monitored for leakage to comply with 10 CFR 20.1406. The WLS discharge lines connect to the Waste Water System (WWS) plant outfall pipe within the Exclusion Area Boundary for dilution below the release limits of 10 CFR Part 20, Appendix B, Table II, Column 2. Dilution at this point, downstream of the WWS blowdown sump, is primarily supplied by the circulating water blowdown flow. The blowdown sump and plant outfall are described in Subsection 9.2.9.2.2. The WWS blowdown line to the plant outfall at the Savannah River is a high density polyethylene single-walled buried pipe. There are no valves, vacuum breakers, or pumps along the WWS blowdown line between the point where the WLS connects and the plant outfall. Monitoring for leakage downstream of the WLS radwaste discharge line connection is per NEI 08-08A (Reference 201) as described in Appendix 12AA. This monitoring will be implemented as part of the Units 3 and 4 groundwater monitoring program.	VEGP-VOL-CH11 item 1 SNC Ltr ND-10-1378
8591	Pt 02	FSAR 11	11.02.01.02.04	COLA Part 2, FSAR Chapter 11, Subsection 11.2.1.2.4, will be revised to remove the DCD info from: 11.2.1.2.4 Controlled Release of Radioactivity Replace the last paragraph in DCD Subsection 11.2.1.2.4 with the following information: The monitored radwaste discharge pipeline is engineered to preclude leakage to the environment. This pipe is routed from the auxiliary building to the radwaste building (the short section of pipe between the two buildings is fully available for visual inspection as noted above) and then out of the radwaste building to the licensed release point for dilution and discharge. The discharge radiation monitor and isolation valve are located inside the auxiliary building. The exterior piping is designed to preclude inadvertent or unidentified releases to the environment. No valves, vacuum breakers, or other fittings are incorporated outside of buildings. This greatly reduces the potential for undetected leakage from this discharge to the environment at a non-licensed release point, and supports compliance with 10 CFR 20.1406 (Reference 5). {current plant specific supplemental information...} To read: 11.2.1.2.4 Controlled Release of Radioactivity Add the following to the end of DCD Subsection 11.2.1.2.4: {current plant specific supplemental information...}	DCD Rev 18
7813	Pt 02	FSAR 11	11.02.06	2. COLA Part 2, FSAR Chapter 11, Subsection 11.2.6, References, will be revised to include the following reference. 201. NEI 08-08A, Generic FSAR Template Guidance for Life Cycle Minimization of Contamination, Revision 0, October 2009 (ML093220445).	VEGP-VOL-CH11 item 2 SNC Ltr ND-10-1378
8443	Pt 02	FSAR 11	11.04.07	COLA Part 2, FSAR Chapter 11, Subsection 11.4.7, Reference 201, is revised to include the ADAMS number in the reference from: ...March 2009. To read: ...March 2009 (ML091460627).	Editorial addition
8444	Pt 02	FSAR 11	11.05.09	COLA Part 2, FSAR Chapter 11, Subsection 11.5.9, Reference 202, is revised to include the ADAMS number in the reference from: ...March 2009. To read: ...March 2009 (ML091050234).	Editorial addition
Pt 02 - FSAR 12				4 COLA Changes	
8018	Pt 02	FSAR 12	12.02.01.01.10	2. COLA Part 2, FSAR Chapter 12, Subsection 12.2.1.1.10, Miscellaneous Sources, will be revised to include a new final paragraph to read: During	VEGP-RAI-LTR-062 response to

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change												
				<p>the period prior to the implementation of the Emergency Plan (in preparation for the initial fuel loading following the 52.103(g) finding), no specific materials related emergency plan will be necessary because:</p> <p>a) No byproduct material will be received, possessed, or used in a physical form that is "in unsealed form, on foils or plated sources, or sealed in glass," that exceeds the quantities in Schedule C in 10 CFR 30.72, and</p> <p>b) The source material to be received, possessed, or used does not involve uranium hexafluoride in excess of 50 kilograms in a single container or 1000 kilograms total.</p>	RAI 01.05-001 item 2 SNC Ltr ND-10-2002												
8727	Pt 02	FSAR 12	12.03F / F12.3-201	<p>Revise COLA Part 2, FSAR Chapter 12, Section 12.3, Figure 12.3-201, Title block to match DCD Figure 12.3-1 Sh11 of 16 title block from: Radiation Zones, Normal Operations /Shutdown Annex Building Elevation 100'-00" & 107'-2"</p> <p>To read: Radiation Zones, Normal Operations/Shutdown Annex Building, Elevation 100'-0" & 107'-2"</p>	Editorial												
8728	Pt 02	FSAR 12	12.03F / F12.3-202	<p>Revise COLA Part 2, FSAR Chapter 12, Section 12.3, Figure 12.3-202, Title block to match DCD Figure 12.3-2 Sh11 of 15 title block from: Radiation Zones, Post-Accident Annex Building, Elevation 100'-0" & 107'-2"</p> <p>To read: Radiation Zones, Post-Accident Annex Building, Elevation 100'-0" & 107'-2"</p>	Editorial												
8729	Pt 02	FSAR 12	12.03F / F12.3-203	<p>Revise COLA Part 2, FSAR Chapter 12, Section 12.3, Figure 12.3-203, Title block to match DCD Figure 12.3-3 Sh11 of 16 title block from: Radiological Access Controls, Normal Operations/Shutdown Annex Building Elevation 100'-00" & 107'-2"</p> <p>To read: Radiological Access Controls, Normal Operations/Shutdown Annex Building, Elevation 100'-00" & 107'-2"</p>	Editorial												
Pt 02 - FSAR 13					6 COLA Changes												
8017	Pt 02	FSAR 13	13.04.T / T13.4-201 14	<p>1. COLA Part 2, FSAR Chapter 13, Section 13.4, Table 13.4-201, item 14, Emergency Planning, will be revised to remove the following information related to materials.</p> <p>(portions applicable to radioactive material) 10 CFR 30.32(i)(3) Prior to initial receipt of 10 CFR 30.32(i)(1) byproduct, source, or 10 CFR 40.310(3) special nuclear materials 10 CFR 40.310(1) (excluding Exempt Quantities as described in 10 CFR 30.18) 70.22(i)(1)</p>	VEGP-RAI-LTR-062 response to RAI 01.05-001 item 1 SNC Ltr ND-10-2002												
8121	Pt 02	FSAR 13	13.04.T / T13.4-201 15	<p>1. COLA Part 2, FSAR, Chapter 13, Section 13.4, Table 13.4-201, item 15, Security Program, will be revised.</p> <p>Refer to the response letter for the complete change.</p>	VEGP-RAI-LTR 051 S2 item 1 SNC Ltr ND-10-2040												
8122	Pt 02	FSAR 13	13.04.T / T13.4-201 21	<p>2. COLA Part 2, FSAR, Chapter 13, Section 13.4, Table 13.4-201, item 21, Cyber Security Program, will be revised to read:</p> <table border="1"> <thead> <tr> <th>Item</th> <th>Program Title</th> <th>Program Source (Required by)</th> <th>FSAR Section</th> <th>Implementation Milestone</th> <th>Requirement</th> </tr> </thead> <tbody> <tr> <td>21.</td> <td>Cyber Security Program</td> <td>10 CFR 73.54(b)</td> <td>13.6</td> <td>Prior to receipt of 10 CFR 73.55(a)(4)</td> <td>fuel onsite</td> </tr> </tbody> </table> <p>10 CFR 73.55(b)(8); (protected area) 10 CFR 73.55(c)(6)</p>	Item	Program Title	Program Source (Required by)	FSAR Section	Implementation Milestone	Requirement	21.	Cyber Security Program	10 CFR 73.54(b)	13.6	Prior to receipt of 10 CFR 73.55(a)(4)	fuel onsite	VEGP-RAI-LTR 051 S2 item 2 SNC Ltr ND-10-2040
Item	Program Title	Program Source (Required by)	FSAR Section	Implementation Milestone	Requirement												
21.	Cyber Security Program	10 CFR 73.54(b)	13.6	Prior to receipt of 10 CFR 73.55(a)(4)	fuel onsite												
8384	Pt 02	FSAR 13	13.04.T / T13.4-201 22	<p>2. COLA Part 2, FSAR Chapter 13, Section 13.4, Table 13.4-201, will be revised by adding a new Item # (where # is the next appropriate number designation) with a left margin annotation (LMA) of STD COL 13.4-1, as follows:</p> <p>Program Title: SNM Material Control and Accounting Program</p> <p>Program Source (Required by): 10 CFR 74, Subpart B (§§ 74.11 - 74.19, excl. § 74.17)</p> <p>FSAR Section: 13.5.2.2.9</p> <p>Implementation Milestone: Prior to receipt of special nuclear material</p> <p>Implementation Requirement: License Condition</p>	VEGP-RAI-LTR-064 response to RAI 01.05-003 item 2 SNC Ltr ND-10-2257												

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
8385	Pt 02	FSAR 13	13.05.02.02.09	<p>3. COLA Part 2, FSAR Chapter 13, Section 13.5, will be revised to add a new subsection 13.5.2.2.9 with an LMA of STD COL 13.5-1, as follows:</p> <p>13.5.2.2.9 Special Nuclear Material (SNM) Material Control and Accounting Procedures</p> <p>A material control and accounting system consisting of special nuclear material accounting procedures is utilized to delineate the requirements, responsibilities, and methods of special nuclear material control from the time special nuclear material is received until it is shipped from the plant. These procedures provide detailed steps for SNM shipping and receiving, inventory, accounting, and preparing records and reports. The Special Nuclear Material (SNM) Material Control and Accounting (MC&A) Program description is submitted to the Nuclear Regulatory Commission as a separate licensing basis document.</p>	VEGP-RAI-LTR-064 response to RAI 01.05-003 item 3 SNC Ltr ND-10-2257
8141	Pt 02	FSAR 13	13AA.01.01.01.01.08	<p>3. COLA Part 2, FSAR Chapter 13, Appendix 13AA, Subsection 13AA.1.1.1.1.8, will be revised to add a new last paragraph to read:</p> <p>Periodic assessment involving both the construction and operations organizations continues to identify SSCs that could reasonably be expected to be impacted by scheduled construction activities. Appropriate administrative and managerial controls are then established as necessary. Specific hazards, impacted SSCs, and managerial and administrative controls are reviewed on a recurring basis and, if necessary, controls are revised/developed and implemented and maintained current as work progresses on site. For example, prior to construction activities that involve the use of large construction equipment such as cranes, managerial and administrative controls are in place to prevent adverse impacts on any operating unit(s) overhead power lines, switchyard, security boundary, etc., by providing the necessary restrictions on the use of large construction equipment.</p>	VEGP-RAI-LTR-063 response to RAI 01.05-002 item 3 SNC Ltr ND-10-2114
Pt 02 - FSAR 14					2 COLA Changes
8224	Pt 02	FSAR 14	14.02.02.02	<p>2. COLA Part 2, FSAR Chapter 14, Subsection 14.2.2.2, PT&O Organization Personnel Qualifications and Training, will be revised to add the following new second paragraph:</p> <p>Acceptable qualifications of non-supervisory test engineers follow the guidance provided in Regulatory Guide 1.28 as discussed in Appendix 1AA, i.e., ASME NQA-1-1994, Appendix 2A-1, Nonmandatory Guidance on the Qualification of Inspection and Test Personnel.</p>	VEGP-VOL-CH14 Qualification Req response item 2 SNC Ltr ND-10-2204
7938	Pt 02	FSAR 14	14A	<p>COLA Part 2, FSAR Chapter 14, new Appendix 14A, will be added (to incorporate new DCD Appendix) to read:</p> <p>APPENDIX 14A DESIGN ACCEPTANCE CRITERIA/ITAAC CLOSURE PROCESS</p> <p>This section of the referenced DCD is incorporated by reference with no departures or supplements.</p>	DCD Rev 18
Pt 02 - FSAR 15					2 COLA Changes
8124	Pt 02	FSAR 15	15.00.03.02	<p>1. COLA Part 2 (Rev. 3), FSAR Chapter 15, Section 15.0.3.2, will be revised to read:</p> <p>The plant operating instrumentation selected for feedwater flow measurement is a Caldon [Cameron] LEFM CheckPlus™ System (Reference 201). This selected plant operating instrumentation has documented instrumentation uncertainties to calculate a power calorimetric uncertainty that confirms the 1% uncertainty assumed for the initial reactor power in the safety analysis bounds the calculated calorimetric power measurement uncertainty values. This calculated calorimetric is done in accordance with a previously accepted Westinghouse methodology (Reference 202). Administrative controls implement maintenance and contingency activities related to the power calorimetric instrumentation.</p>	COL-SER-OI-CH15 S3 response to SER-OI-15.00-001 item 1 SNC Ltr ND-10-2091
8125	Pt 02	FSAR 15	15.00.16	<p>2. COLA Part 2 (Rev. 3), FSAR Chapter 15, Section 15.0, will be revised to add the following new subsection:</p> <p>15.0.16 References</p> <p>Add the following text to the end of DCD Subsection 15.0.16.</p> <p>201. Final Safety Evaluation for Cameron Measurement Systems Engineering Report ER-157P, Revision 8, "Caldon Ultrasonics Engineering Report ER-157P, "Supplement to Topical Report ER-80P: Basis for a Power Uprate with the LEFM Check or Checkplus™ System", (TAC No. ME1321). August 16, 2010. ADAMS Accession No. ML102160694.</p> <p>202. Final Safety Evaluation for Beaver Valley Power Station, Unit Nos. 1 and 2 (BVPS-1 and 2) Issuance of Amendment re: 1.4-Percent Power Uprate and Revised BVPS-2 Heatup and Cooldown Curves. September 24, 2001, ADAMS Accession No. ML012490569.</p>	COL-SER-OI-CH15 S3 response to SER-OI-15.00-001 item 2 SNC Ltr ND-10-2091
Pt 02 - FSAR 17					1 COLA Change
7792	Pt 02	FSAR 17	17.06	<p>9. COLA Part 2, FSAR Chapter 17, Section 17.6, will be revised to add a new LMA of STD COL 3.8-5 to the first paragraph.</p>	VEGP-VOL-Ch03 SIP response to STD COL 03.08-005 item 9 SNC Ltr ND-10-1594
Pt 02 - FSAR 18					3 COLA Changes

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
7692	Pt 02	FSAR 18	18.02.01.03	<p>1. COLA Part 2, FSAR Chapter 18, Subsection 18.2.1.3, will be revised from:</p> <p>A common EOF is located in Birmingham, Alabama, and it supports all of SNC's units including VEGP Units 3 and 4. The EOF communications strategy is described in the Emergency Plan.</p> <p>The scope of functions and tasks to which the AP1000 Human Factors Engineering Program Plan will be applied to the TSC and the EOF are identified and addressed in Westinghouse Technical Report APP-GW-GLR-136 (Reference 201).</p> <p>To read:</p> <p>The EOF and TSC communications strategies, as well as the EOF and TSC Human Factors attributes, are described in the Emergency Plan. FSAR 9.5.2.2.5 provides additional information related to offsite interfaces.</p>	VEGP-VOL-Ch 18 re EOF item 1 SNC Ltr ND-10-1371
8407	Pt 02	FSAR 18	18.02.01.03	<p>COLA Part 2, FSAR Chapter 18, Subsection 18.2.1.3, will be revised from:</p> <p>FSAR 9.5.2.2.5 provides additional information related to offsite interfaces.</p> <p>To read:</p> <p>Subsection 9.5.2.2.5 provides additional information related to offsite interfaces.</p>	Editorial revision to VEGP-VOL-Ch 18 re EOF item 1 SNC Ltr ND-10-1371
7693	Pt 02	FSAR 18	18.02.07	<p>2. COLA Part 2, FSAR Chapter 18, will be revised to delete Subsection 18.2.7 and Reference 201.</p>	VEGP-VOL-Ch 18 re EOF item 2 SNC Ltr ND-10-1371
Pt 02 - FSAR 19				7 COLA Changes	
7873	Pt 02	FSAR 19	19.55	<p>2. COLA Part 2, FSAR Chapter 19, Section 19.55, Seismic Margin Analysis, will be revised to read:</p> <p>19.55 SEISMIC MARGIN ANALYSIS</p> <p>This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.</p>	VEGP-VOL-CH19 PRA item 2 SNC Ltr ND-10-1811
7874	Pt 02	FSAR 19	19.55.06	<p>3. COLA Part 2, FSAR Chapter 19, Subsection 19.55.6 will be revised to add the following subsection with LMA VEGP COL 19.59.10-6:</p> <p>19.55.6.3 Site Specific Seismic Margin Analysis</p> <p>The VEGP GMRS exceeds the AP1000 CSDRS over certain frequency ranges, site-specific seismic soil-structure interaction analyses of the AP1000 Nuclear Island (NI) using the VEGP GMRS ground motion and VEGP soil profile demonstrated that the resulting VEGP in-structure response spectra (ISRS) at the six key locations are enveloped by the AP1000 CSDRS broadened ISRS by a significant margin except for minor exceedances at very low frequency ranges. These slight exceedances have been shown to have no impact on the NI structures, systems, and components. Therefore, the Tier 1 criteria for SSE have been satisfied. This evaluation is presented in Section 3.7.</p> <p>In regards to seismic demand for the NI structures, systems, and components, it can be concluded that the Seismic Margin Assessment analysis documented in Section 19.55 is applicable to the VEGP site.</p> <p>For seismic stability of the NI with regards to sliding and overturning, it was demonstrated that VEGP NI margins against sliding and overturning were greater than the limiting margins calculated for the standard AP1000 design cases. For seismic stability, it can be concluded that the Seismic Margin Assessment analysis documented in DCD Section 19.55 is applicable to the VEGP site.</p> <p>For site specific conditions relating to soil-related failure modes, the demonstration of adequate seismic margin of the AP1000 design at the VEGP site is performed for a review level earthquake of 1.67 x VEGP GMRS, where the VEGP site-specific review level earthquake seismic responses and seismic loads are defined as 1.67 x VEGP GMRS seismic responses and seismic loads.</p> <p>Potential for soil liquefaction was evaluated at 1.67 x VEGP GMRS which produces a peak ground acceleration of 0.44g. The liquefaction potential factor of safety was found to be high such that liquefaction potential was screened out as a contributor to design-specific plant-level HCLPF capacity. Similarly, bearing pressure capacity to demand still demonstrated sufficient margin so this potential failure mode was screened out as a contributor to design specific plant-level HCLPF capacity.</p>	VEGP-VOL-CH19 PRA item 3 SNC Ltr ND-10-1811
8740	Pt 02	FSAR 19	19.55.06.03	<p>COLA Part 2, FSAR Chapter 19, Subsection 19.55.6.3 will be revised from:</p> <p>The VEGP GMRS exceeds the AP1000 CSDRS over certain frequency ranges, site-specific seismic soil-structure interaction analyses...</p> <p>To read:</p> <p>The VEGP GMRS exceeds the AP1000 CSDRS over certain frequency ranges. Site-specific seismic soil-structure interaction analyses...</p>	Editorial revision to VEGP-VOL-CH19 PRA item 3 SNC Ltr ND-10-1811
8946	Pt 02	FSAR 19	19.58.T / T19.58-201	<p>COLA Part 2, FSAR Chapter 19, Section 19.58, Table 19.58-201 (Sheet 9 of 10), under External Fires, last sentence from: APPGW-GLR-101</p> <p>To read: APP-GW-GLR-101</p>	Editorial to match original submittal via letter ND-09-0975.
7872	Pt 02	FSAR 19	19.59.10.05	<p>1. COLA Part 2, FSAR Chapter 19, Subsection 19.59.10.5, Combined License Information, will be revised to add the following as the last paragraph with LMAs STD COL 19.59.10-6 and VEGP COL 19.59.10-6:</p> <p>As discussed in Subsection 19.55.6.3, it has been confirmed that the Seismic Margin Analysis (SMA) documented in DCD Section 19.55 is</p>	VEGP-VOL-CH19 PRA item 1 SNC Ltr ND-10-1811

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
				applicable to the site. The site-specific effects (i.e., soil-related failure modes, etc.) have been evaluated and it was concluded that the plant-specific plant-level HCLPF value is equal to or greater than 1.67 times the site-specific GMRS peak ground acceleration.	
7876	Pt 02	FSAR 19	19.59.10.05	5. COLA Part 2, FSAR Chapter 19, Subsection 19.59.10.5, Combined License Information 19.59.10-1, Item 1 will be revised to read: 1. Specific minimum seismic requirements consistent with those used to define the AP1000 DCD Table 19.55-1 HCLPF values. This includes the known frequency range used to define the HCLPF by comparing the required response spectrum (RRS) and test response spectrum (TRS). The test response spectra are chosen so as to demonstrate that no more than one percent rate of failure is expected when the equipment is subjected to the applicable seismic margin ground motion for the equipment identified to be applicable in the seismic margin insights of the site-specific PRA. The range of frequency response that is required for the equipment with its structural support is defined.	VEGP-VOL-CH19 PRA item 5 SNC Ltr ND-10-1811
8445	Pt 02	FSAR 19	19.59.10.05	COLA Part 2, FSAR Chapter 19, Subsection 19.59.10.5, first paragraph, is revised to add the LMA STD COL 19.59.10-6 along with the existing LMA of STD COL 19.59.10-1.	Consistency with changes made in VEGP-VOL-CH19 PRA item 1 SNC Ltr ND-10-1811
Pt 04 - (empty)					7 COLA Changes
5900	Pt 04		A, A.2	1) At the end of the Justification sentence for GTS 3.3.1, 3.3.2 and 3.6.4 add the words "the plant specific" in front of "technical specifications". 2) At the end of the Justification sentence for the first GTS 5.2.2 and in 5.3 add the word "the" in front of "plant specific technical specifications".	Editorial
6525	Pt 04		B, 00 All	COLA Part 4, Technical Specifications, will be revised to incorporate the AP1000 GTS changes identified in the WEC response to the AP1000 DCD SER Open Item, OI-SRP3.9.6-CIB1-05.	INCLUDED in QB 8358 COL-SER-OI-Ch03 response to OI 03.09-06 (SNC Ltr ND-09-2015)
7265	Pt 04		B, 00 All	See attachments to Westinghouse letter DCP/NRC2864 (dated May 6,2010) -the same changes identified in the Westinghouse letter for the DCD Generic Technical Specifications and Bases will be directly incorporated into the COL plant-specific Technical Specifications and Bases (with the exception that the bracketed Reviewer's Notes will be removed).	INCLUDED in QB 8358 COL-SER-OI-CH16 S2 response to OI 16.01-001 SNC Ltr ND-10-0996
8358	Pt 04		B, 00 All	COLA Part 4, Technical Specifications, will be revised to incorporate the AP1000 GTS changes identified by WEC in numerous responses to AP1000 DCD RAIs and SER Open Items, and other sources as incorporated into DCD Revision 18.	DCD Rev 18
7256	Pt 04		B, B02.01.02	Revise header for Bases of TS 2.1.2 from "Reactor Core SLs" to read "RCS Pressure SL".	INCLUDED in QB 8358 Consistency with DCD
5099	Pt 04		B, B03.03.02 / B3.3.2-38	Delete the blank line in the first paragraph under Item 18.a on Bases Page B 3.3.2-38.	INCLUDED in QB 8358 Editorial
5091	Pt 04		B, B03.03.02 / B3.3.2-63	For SR 3.3.2.5 change "every 9 days" to read "every 92 days" to correct a typographical error created during COLA Revision 1.	INCLUDED in QB 8358 DCD Rev 17 conformance
Pt 05 - (empty)					4 COLA Changes
8136	Pt 05		B.00 / Pg02	1. Add the following supplements after the second paragraph of COLA Part 5: On Table B-2, Emergency Response Organization Assignments, of the Base Plan, revise the table row for Emergency Position "TSC Engineering Supervisor" to read: TSC Engineering Supervisor VEGP 1 and 2 Supervision from onsite staff as designated in emergency implementing procedures TSC Engineering Supervisor VEGP 3 and 4 Supervision from onsite staff as designated in emergency implementing procedures On Table B-2, Emergency Response Organization Assignments, of the Base Plan, revise the table row for Emergency Position "TSC Operations Supervisor" to read: TSC Operations Supervisor VEGP 1 and 2 Supervision from onsite staff as designated in emergency implementing procedures TSC Operations Supervisor VEGP 3 and 4 Supervision from onsite staff as designated in emergency implementing procedures	VEGP-P10-VOL-CH18 TSC ACRS item 1 SNC Ltr ND-10-2086
8137	Pt 05		O.00 / Pg07	2. Add the following supplements after the seventh paragraph of COLA Part 5: On Table O-2, Training Course Description, of the Base Plan, revise the table row for Training Course "Emergency Plan Overview (EPO)" to read:	VEGP-P10-VOL-CH18 TSC ACRS item 2 SNC Ltr ND-10-2086

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change				
				<p>Emergency Plan Overview (EPO) (1) This course covers an overview of the emergency plan with special attention to emergency planning zones (EPZs); emergency classification systems; onsite emergency response organizations; responsibilities of emergency response personnel; and site accountability and site dismissal.</p> <p>On Table O-2, Training Course Description, of the Base Plan, revise the table row for Training Course "Management of Radiological Emergencies (MREs)" to read:</p> <p>Management of Radiological Emergencies (MREs) This course covers classification of emergencies using classification systems used by VEGP 1 and 2 and VEGP 3 and 4 as appropriate; emergency notification of onsite and offsite emergency response personnel and agencies; activation and staffing of emergency response facilities; protective action recommendation decision making based on EPA PAGs; retrieval of available plant computer data; reentry and repair operations; and communications and information management; and recovery.</p>					
8394	Pt 05		O.00 / Pg07	Restore footnote of COLA Part 5 on Table O-2, Training Course Description, of the Base Plan, for the table row for Training Course "Emergency Plan Overview (EPO)" to read (a), not (1).	Editorial restoration of unintentional change included in VEGP-P10-VOL-CH18 TSC ACRS item 2 SNC Ltr ND-10-2086				
7696	Pt 05		wa7.A / Pg08	<p>5. COLA Part 5, Emergency Plan, will be revised to add language to Appendix 7 of the VEGP 3 and 4 Emergency Plan as described below:</p> <p>Add the following sentence at the end of Subsection A7A.1:</p> <p>"The Emergency Operations Facility has been established consistent with NUREG 0696 guidelines."</p>	VEGP-VOL-Ch 18 re EOF item 5 SNC Ltr ND-10-1371				
Pt 07 - (empty)					15 COLA Changes				
8756	Pt 07		A / DEP 3.4-1	<p>COLA Part 7, Section A, STD and VEGP Departures, will be revised to add the following departure:</p> <p>VEGP DEP 3.4-1 Waterproofing membrane material</p>	DCD Rev 18 vs VEGP ESP				
8029	Pt 07		A / DEP 8.3-1	<p>4. COLA Part 7, Section A, STD and VEGP Departures, will be revised to add the following departure:</p> <table border="1"> <thead> <tr> <th>Departure Number</th> <th>Description</th> </tr> </thead> <tbody> <tr> <td>STD DEP 8.3-1</td> <td>Class 1E voltage regulating transformer current limiting features</td> </tr> </tbody> </table>	Departure Number	Description	STD DEP 8.3-1	Class 1E voltage regulating transformer current limiting features	VEGP-VOL-CH08 response to STD-VOL-08.03-002 item 4 SNC Ltr ND-10-2005
Departure Number	Description								
STD DEP 8.3-1	Class 1E voltage regulating transformer current limiting features								
8757	Pt 07		A.1 / DEP 3.4-1	<p>COLA Part 7, Section A.1, STD and VEGP Departures, will be revised to add the following departure to the table:</p> <p>VEGP DEP 3.4-1 Waterproofing membrane material</p> <p>and to the appropriate location in the text... [See complete change description for DEP text.]</p>	DCD Rev 18 vs VEGP ESP				
8030	Pt 07		A.1 / DEP 8.3-1	5. COLA Part 7, Section A.1, Departures that Can Be Implemented Without Prior NRC Approval, will be revised to add STD DEP 8.3-1 related to the Class 1E voltage regulating transformer current limiting features. [See letter for full text of departure.]	VEGP-VOL-CH08 response to STD-VOL-08.03-002 item 5 SNC Ltr ND-10-2005				
8902	Pt 07		A.1 / DEP 8.3-1	<p>COLA Part 7, Section A.1, STD DEP 8.3-1, will be revised from: ...FSAR Section 8.3.2.2.</p> <p>To read: ...FSAR Subsection 8.3.2.2.</p> <p>and from: DCD section 8.3.2.2 states... To read: DCD Subsection 8.3.2.2 states...</p>	Editorial revision to VEGP-VOL-CH08 response to STD-VOL-08.03-002 item 5 SNC Ltr ND-10-2005				
8730	Pt 07		A.1 / DEP 9.2-1	COLA Part 7, Section A.1, VEGP DEP 9.2-1, will be revised in the Departure Evaluation, from: This Tier 2 departure is associated with a non safety-related system.	DCD Rev 18 vs VEGP ESP				

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
				To read: This Tier 2 departure is associated with the nonsafety-related RWS supply to the PWS.	
8386	Pt 07		B / EXM 3	4. COLA Part 7, Departures, Exemptions, and Variances, Part B, will be revised to add the following exemption request (where # is the next appropriate exemption request number): # Special Nuclear Material (SNM) Material Control and Accounting Program Description	VEGP-RAI-LTR-064 response to RAI 01.05-003 item 4 SNC Ltr ND-10-2257
8387	Pt 07		B / EXM 3	5. COLA Part 7, Departures, Exemptions, and Variances, Part B, will be revised to add the discussion and justification for Exemption related to the Special Nuclear Material (SNM) Material Control and Accounting (MC&A) Program. [See letter for full text of the exemption request.]	VEGP-RAI-LTR-064 response to RAI 01.05-003 item 5 SNC Ltr ND-10-2257
8355	Pt 07		C. VAR 1.2-1	COLA Part 7, VEGP VAR 1.2-1, will be revised to delete the sentence: This information is currently under review by the NRC.	DCD Rev 18
8352	Pt 07		C. VAR 1.6-1	COLA Part 7, VEGP VAR 1.6-1, will be revised to read: Section 1.6 of the ESPA SSAR contains a reference to Revision 15 of the AP1000 DCD. This information has been superseded by a later revision of the AP1000 DCD, Section 1.6, which is incorporated by reference into the VEGP COLA FSAR. The later revision of the AP1000 DCD contains the most updated information. Therefore, a variance is required to not incorporate Section 1.6 of the ESPA SSAR because it contains superseded information. The VEGP COLA incorporates by reference the updated information from Section 1.6 of the most recent revision of the DCD.	DCD Rev 18
8353	Pt 07		C. VAR 1.6-2	COLA Part 7, VEGP VAR 1.6-2, will be revised to read: The first paragraph of Section 3.8.5 of the ESPA SSAR contains a reference to Revision 15 of the AP1000 DCD. This information has been superseded by a later revision of Section 3.8.5 of the AP1000 DCD, which is incorporated by reference into the VEGP COLA FSAR. The later revision of the AP1000 DCD contains the most updated information. Therefore, a variance is required to not incorporate the first paragraph of ESPA SSAR Section 3.8.5. The VEGP COLA incorporates by reference the updated information from the most recent revision of DCD Section 3.8.5. The first sentence of the second paragraph in ESPA SSAR Subsection 3.8.5.1.1 contains a reference to a departure from the AP1000 DCD that is no longer needed since the DCD was revised to include this waterproofing option in a later revision. Therefore, a variance is required to not incorporate this statement.	DCD Rev 18
8761	Pt 07		C. VAR 1.6-2	COLA Part 7, VEGP VAR 1.6-2, will be revised to read (includes Qb 8353): Variance Number: VEGP VAR 1.6-2 Summary of Variance: The first paragraph of ESPA SSAR Subsection 3.8.5.1, Description of Foundations, is not incorporated by reference into the COLA FSAR. The first paragraph in ESPA SSAR Subsection 3.8.5.1.1 is also not incorporated by reference into the COLA FSAR. Justification for Variance: The first paragraph of Subsection 3.8.5.1 of the ESPA SSAR contains a reference to Revision 15 of the AP1000 DCD. This information has been superseded by a later revision of Subsection 3.8.5 of the AP1000 DCD, which is incorporated by reference into the VEGP COLA FSAR. The later revision of the AP1000 DCD contains the most updated information. Therefore, a variance is required to not incorporate the first paragraph of ESPA SSAR Subsection 3.8.5.1. The VEGP COLA incorporates by reference the updated information from the most recent revision of DCD Subsection 3.8.5. In addition, the first paragraph in ESPA SSAR Subsection 3.8.5.1.1 contains a reference to waterproofing systems described in the current AP1000 DCD. Therefore, a variance is required to not incorporate this paragraph.	DCD Rev 18 vs VEGP ESP
8354	Pt 07		C. VAR 1.6-3	COLA Part 7, VEGP VAR 1.6-3, will be revised to read: Chapter 15 of the ESPA SSAR contains accident release information based upon Revision 15 of the AP1000 DCD. This information has been superseded by the associated section of a later revision of the AP1000 DCD, which is incorporated by reference into the VEGP COLA FSAR. The later revision of the AP1000 DCD contains the most updated information. A variance is required to not incorporate by reference ESPA SSAR Chapter 15.	DCD Rev 18
8356	Pt 07		C. VAR 2.2-1	COLA Part 7, VEGP VAR 2.2-1, will be revised to delete the phrase: ", which is currently under review by the NRC"	DCD Rev 18
8357	Pt 07		C. VAR 2.3-1	COLA Part 7, VEGP VAR 2.3-1, will be revised from: These changes are provided to conform to AP1000 DCD Revision 17 changes. To read: These changes are provided to conform to AP1000 DCD revisions.	DCD Rev 18
Pt 09 - (empty)					25 COLA Changes
7770	Pt 09		Index	Revise Index to Part 9 to include: 11 11B Mitigative Strategies Description and Plans	Editorial
8446	Pt 09		Index	Revise Index to Part 9 from: 11C Plan Cyber Security Plan To read:	Editorial

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
				11 11C Cyber Security Plan	
7224	Pt 09		02-01.02F / F1.2-201	Revise COLA Part 9, FSAR Chapter 1, Section 1.2, Figure 1.2-201, per updated DCD Figure 1.2-18. Remove reference to OSC to make it plant-specific.	DCD Rev 18
8458	Pt 09		02-01.02F / F1.2-201	Revise COLA Part 9, FSAR Chapter 1, Section 1.2, Figure 1.2-201, Title block to match DCD Figure 1.2-18 title block from: Annex Building General Arrangement Plant at Elevation 100'-0" and 107'-2" To read: Annex Building General Arrangement Plan at Elevation 100'-0" & 107'-2"	Editorial
7225	Pt 09		02-09AF / F9A-201	Revise COLA Part 9, FSAR Chapter 9, Appendix 9A, Figure 9A-201, per updated DCD Figure 9A-3. Remove reference to OSC to make it plant-specific.	DCD Rev 18
8459	Pt 09		02-09AF / F9A-201	Revise COLA Part 9, FSAR Chapter 9, Appendix 9A, Figure 9A-201, Title block to match DCD Figure 9A-3, Sheet 1 of 3, title block from: Annex I and II Building Fire Areas Plan at Elevation 100'-0" and 107'-2" To read (in italics): [Annex I and II Building Fire Areas Plan at Elevation 100'-0" & 107'-2"]* and to include the * footnote from the DCD.	DCD Rev 18 and Editorial
7226	Pt 09		02-12.03F / F12.3-201	Revise COLA Part 9, FSAR Chapter 12, Section 12.3, Figure 12.3-201, per updated DCD Figure 12.3-1, Sheet 11 of 16. Remove reference to OSC to make it plant-specific.	DCD Rev 18
8460	Pt 09		02-12.03F / F12.3-201	Revise COLA Part 9, FSAR Chapter 12, Section 12.3, Figure 12.3-201, Title block to match DCD Figure 12.3-1 Sh11 of 16 title block from: Radiation Zones, Normal Operations /Shutdown Annex Building Elevation 100'-00" & 107'-2" To read: Radiation Zones, Normal Operations/Shutdown Annex Building, Elevation 100'-0" & 107'-2"	Editorial
7227	Pt 09		02-12.03F / F12.3-202	Revise COLA Part 9, FSAR Chapter 12, Section 12.3, Figure 12.3-202, per updated DCD Figure 12.3-2, Sheet 11 of 15. Remove reference to OSC to make it plant-specific.	DCD Rev 18
8461	Pt 09		02-12.03F / F12.3-202	Revise COLA Part 9, FSAR Chapter 12, Section 12.3, Figure 12.3-202, Title block to match DCD Figure 12.3-2 Sh11 of 15 title block from: Radiation Zones, Post-Accident Annex Building, Elevation 100'-0" & 107'-2" To read: Radiation Zones, Post-Accident Annex Building, Elevation 100'-0" & 107'-2"	Editorial
7228	Pt 09		02-12.03F / F12.3-203	Revise COLA Part 9, FSAR Chapter 12, Section 12.3, Figure 12.3-203, per updated DCD Figure 12.3-3, Sheet 11 of 16. Remove reference to OSC to make it plant-specific.	DCD Rev 18
8462	Pt 09		02-12.03F / F12.3-203	Revise COLA Part 9, FSAR Chapter 12, Section 12.3, Figure 12.3-203, Title block to match DCD Figure 12.3-3 Sh11 of 16 title block from: Radiological Access Controls, Normal Operations/Shutdown Annex Building Elevation 100'-00" & 107'-2" To read: Radiological Access Controls, Normal Operations/Shutdown Annex Building, Elevation 100'-00" & 107'-2"	Editorial
5851	Pt 09		11B-MSDP	1. Change COLA Part 9, Withheld Information, to include the new Mitigative Strategies Description and Plans (as provided in Attachment A), which contains security-related information, and therefore, should be withheld, in accordance with 10 CFR 2.390(d).	SNC Letter ND-10-2192 (includes RAI responses) Commitment made in response to LOLA Mitigative Strategies Description and Plans (see SNC Letter ND-09-0886)
8473	Pt 09		11B-MSDP 02.00	This COLA change contains Security-Related Information associated with the LOLA MSD required by 10 CFR 52.80(d). The change to COLA SubPart 11.B, Loss of Large Areas of the Plant due to Fire or Explosions Mitigative Strategies Description and Plans, Section 2.0, revises the level of the SFP that is maintained during normal operation to conform with the normal SFP level range specified in the design basis information.	Conforms with updated design basis information. Normal SFP level range is provided in APP-SFS-M3-001, Spent Fuel Pool Cooling System - System Specific Document, Rev. 2, Table 6-1.
8474	Pt 09		11B-MSDP 06.02	This COLA change contains Security-Related Information associated with the LOLA MSD required by 10 CFR 52.80(d). This changes COLA Subpart 11.B, Loss of Large Areas of the Plant due to Fire or Explosions Mitigative Strategies Description and Plans, Section 6.2.1, Table 6-2, and Section 6.2.2, Table 6-3, by correcting the room numbers that are identified for two valves in these tables.	Corrects the room number for a room containing two valves, which was erroneously identified on the system Piping and Instrumentation Diagram.

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
8475	Pt 09		11B-MSDP 06.02	This COLA change contains Security-Related Information associated with the LOLA MSD required by 10 CFR 52.80(d). This changes COLA SubPart 11.B, Loss of Large Areas of the Plant due to Fire or Explosions Mitigative Strategies Description and Plans, Section 6.2.3, fifth through seventh paragraphs, to identify the flow rates as estimated values, rather than approximate values.	Conforms with statements regarding other potential makeup flow paths in MSD Section 6.2.3. These makeup rates are estimated based on available system and equipment descriptions, which is sufficient for non-standard flow paths such as those described in Section 6.2.3.
8476	Pt 09		11B-MSDP 06.02	This COLA change contains Security-Related Information associated with the LOLA MSD required by 10 CFR 52.80(d). This changes COLA SubPart 11.B, Loss of Large Areas of the Plant due to Fire or Explosions Mitigative Strategies Description and Plans, Section 6.4, second paragraph, by deleting the degree sign used in the discussion of restriction of the maximum cladding temperature.	Editorial correction. Deletes degree symbol in conformance with convention provided by U.S. Metric Association, Inc.
9065	Pt 09		11B-MSDP 06.02	1. COLA Part 11, LOLA Mitigative Strategies Description, Section 6.2, first paragraph, will be revised. This COLA change contains Security-Related Information associated with the LOLA MSD required by 10 CFR 52.80(d). Refer to SNC Letter ND-10-1570 for the RAI and the associated RAI response.	VEGP-RAI-LTR-042 S4 response to RAI 19-069 item 1 SNC Ltr ND-10-1570
9066	Pt 09		11B-MSDP 06.02.01	2. COLA Part 11, LOLA Mitigative Strategies Description, Section 6.2.1, second paragraph, first sentence will be revised. This COLA change contains Security-Related Information associated with the LOLA MSD required by 10 CFR 52.80(d). Refer to SNC Letter ND-10-1570 for the RAI and the associated RAI response.	VEGP-RAI-LTR-042 S4 response to RAI 19-069 item 2 SNC Ltr ND-10-1570
9067	Pt 09		11B-MSDP 06.02.02	3. COLA Part 11, LOLA Mitigative Strategies Description, Section 6.2.2, will be revised. This COLA change contains Security-Related Information associated with the LOLA MSD required by 10 CFR 52.80(d). Refer to SNC Letter ND-10-1570 for the RAI and the associated RAI response.	VEGP-RAI-LTR-042 S4 response to RAI 19-069 item 3 SNC Ltr ND-10-1570
9068	Pt 09		11B-MSDP 06.03.T / T 6.3	4. COLA Part 11, LOLA Mitigative Strategies Description, Table 6-3, will be revised. This COLA change contains Security-Related Information associated with the LOLA MSD required by 10 CFR 52.80(d). Refer to SNC Letter ND-10-1570 for the RAI and the associated RAI response.	VEGP-RAI-LTR-042 S4 response to RAI 19-069 item 4 SNC Ltr ND-10-1570
9069	Pt 09		11B-MSDP 06.04	5. COLA Part 11, LOLA Mitigative Strategies Description, will be revised by adding a new Section 6.4, Spent Fuel Pool Loading Strategies. This COLA change contains Security-Related Information associated with the LOLA MSD required by 10 CFR 52.80(d). Refer to SNC Letter ND-10-1570 for the RAI and the associated RAI response.	VEGP-RAI-LTR-042 S4 response to RAI 19-069 item 5 SNC Ltr ND-10-1570
9064	Pt 09		11B-MSDP 07.19	COLA Part 11, Loss of Large Areas of the Plant Due to Explosions or Fire, Mitigative Strategies Description (MSD), Section 7.19. This COLA change contains Security-Related Information associated with the LOLA MSD required by 10 CFR 52.80(d). Refer to SNC Letter ND-10-1897 for the RAI and the associated RAI response.	VEGP-RAI-LTR-054 S2 response to RAI 19-102 (SNC Ltr ND-10-1897)
9070	Pt 09		11B-MSDP MST Phase 1	6. COLA Part 11, LOLA Mitigative Strategies Description, MST, Phase 1, Plant Operations to Mitigate Fuel Damage, Item 13, will be revised. This COLA change contains Security-Related Information associated with the LOLA MSD required by 10 CFR 52.80(d). Refer to SNC Letter ND-10-1570 for the RAI and the associated RAI response.	VEGP-RAI-LTR-042 S4 response to RAI 19-069 item 6 SNC Ltr ND-10-1570
9071	Pt 09		11B-MSDP MST Phase 2, Item 3	7. COLA Part 11, LOLA Mitigative Strategies Description, MST, Phase 2, Item 3, will be revised. This COLA change contains Security-Related Information associated with the LOLA MSD required by 10 CFR 52.80(d). Refer to SNC Letter ND-10-1570 for the RAI and the associated RAI response.	VEGP-RAI-LTR-042 S4 response to RAI 19-069 item 7 SNC Ltr ND-10-1570
Pt 10 - (empty)					19 COLA Changes
8222	Pt 10		LC#01	2. COLA Part 10, Proposed License Condition #1, ITAAC, introductory statements will be revised to read: There are several ITAAC identified in the COLA. Once incorporated into the COL, the regulations identify the requirements that must be met. The incorporation below includes the sensitive unclassified non-safeguards information (including proprietary information), and safeguards information referenced in the AP1000 DCD. Such DCD information is included in this combined license application in the same manner as it is included in the AP1000 DCD, i.e., references in the DCD are included as references in the FSAR, and material incorporated by reference into the DCD is incorporated by reference into the FSAR. Appropriate agreements are in place to provide access to the withheld sensitive unclassified non-safeguards information (including proprietary information), and safeguards information referenced in the AP1000 DCD.	VEGP-VOL-CH01 IBR of PI & SGI response item 2 SNC Ltr ND-10-2207
8020	Pt 10		LC#02, 14.04-03	1. COLA Part 10, Proposed License Conditions, including ITAAC, proposed License Condition #2, item 14.4-3, Conduct of Test Program, will be revised to read:	VEGP-VOL-CH14 response to item 1 SNC Ltr ND-10-1993

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
				14.4-3 Conduct of Test Program 14.4.3 NOTE -addressed by proposed License Conditions #3 and #6.	
8021	Pt 10		LC#02, 14.04-04	2. COLA Part 10, Proposed License Conditions, including ITAAC, proposed License Condition #2, item 14.4-4, Review and Evaluation of Test Results, will be revised to read: 14.4-4 Review and Evaluation of Test Results 14.4.4 NOTE - addressed by proposed License Condition #9.	VEGP-VOL-CH14 response to item 2 SNC Ltr ND-10-1993
8022	Pt 10		LC#02, 14.04-06	3. COLA Part 10, Proposed License Conditions, including ITAAC, proposed License Condition #2, item 14.4-6, First-Plant-Only and Three-Plant-Only Tests, will be revised from: 14.4-6 First-Plant-Only and Three-Plant-Only Tests 14.4.6 Prior to preoperational testing The COL holder for the first plant and the first three plants will perform the tests listed in subsection 14.2.5. For subsequent plants, either tests listed in subsection 14.2.5 shall be performed, or the COL applicant shall provide a justification that the results of the first-plant-only tests or firstthree-plant tests are applicable to the subsequent plant. The Combined License holder(s) for the first AP 1000 plant (or first three plants) available for testing will perform the tests defined during preoperational and startup testing as identified in subsections 14.2.9 and 14.2.10. Combined License holders referencing the results of the tests will provide the report as necessary. The schedule for providing this information will be provided prior to preoperational testing. To read: 14.4-6 First-Plant-Only and Three-Plant-Only Tests 14.4.6 NOTE -addressed by proposed License Conditions #7 and #9.	VEGP-VOL-CH14 response to item 3 SNC Ltr ND-10-1993
7765	Pt 10		LC#02, 15.0-1	3. COLA Part 10, Proposed License Conditions, LC#2, will be revised to include a new line item for COL item 15.0-1 as follows: 15.0-1 Documentation of Plant Calorimetric 15.0.15.1 Prior to initial fuel load Uncertainty Methodology Confirm the plant operating instrumentation installed for feedwater flow measurement is a Caldon [Cameron] LEFM CheckPlus™ System. Additionally, confirm that administrative controls are in place to implement maintenance and contingency activities related to the system.	SUPERSEDED by Qb 8126, COL-SER-OI-Ch15 S2 response to SER OI 15.00-001 item 3 SNC Ltr ND-10-1527
8126	Pt 10		LC#02, 15.0-1	3. COLA Part 10 (Rev. 3), Proposed License Conditions, LC#2, COL Holder Items, COL Item No. 15.0-1, will be revised to read: 15.0-1 Documentation of Plant Calorimetric 15.0.15.1 Uncertainty Methodology Note -addressed by proposed ITAAC Table 2.5.4-2, item 4.	COL-SER-OI-CH15 S3 response to SER-OI-15.00-001 item 3 SNC Ltr ND-10-2091
7877	Pt 10		LC#02, 19.59.10-01	6. COLA Part 10, License Conditions and ITAAC, Section 2, COL Item No. 19.59.10-1, Item 1 will be revised to read: 1. Specific minimum seismic requirements consistent with those used to define the Table 19.55-1 HCLPF values. This includes the known frequency range used to define the HCLPF by comparing the required response spectrum (RRS) and test response spectrum (TRS). The test response spectra are chosen so as to demonstrate that no more than one percent rate of failure is expected when the equipment is subjected to the applicable seismic margin ground motion for the equipment identified to be applicable in the seismic margin insights of the site-specific PRA. The range of frequency response that is required for the equipment with its structural support is defined.	VEGP-VOL-CH19 PRA item 6 SNC Ltr ND-10-1811
8019	Pt 10		LC#03 C.4	3. COLA Part 10, proposed License Condition 3, Operational Program Implementation, item C, Receipt of Materials, will be revised to read: C.4 -Deleted	VEGP-RAI-LTR-062 response to RAI 01.05-001 item 3 SNC Ltr ND-10-2002
8388	Pt 10		LC#03 C.7	6. COLA Part 10, Proposed License Conditions, LC#3.C, Operational Program Implementation, Receipt of Materials, will be revised to include a new line item for implementation of an SNM material control and accounting program, as follows (where # is the next appropriate letter designation): C.#. SNM Material Control and Accounting Program	VEGP-RAI-LTR-064 response to RAI 01.05-003 item 6 SNC Ltr ND-10-2257
7937	Pt 10		LC#06	3. COLA Part 10, Proposed License Conditions, will be revised to add a new standard item to proposed license condition 6 to read (where # is the next appropriate letter): #. the implementation of construction and inspection procedures for concrete filled steel plate modules activities before and after concrete placement, use of construction mock-ups, and inspection of modules before and after concrete placement as discussed in DCD Subsection 3.8.4.8.	VEGP-VOL-CH03 Const Procedures response to STD-COL-03.08-006 item 3 SNC Ltr ND-10-1900
8023	Pt 10		LC#06	4. COLA Part 10, Proposed License Conditions, including ITAAC, proposed License Condition #6, Operational Program Readiness, will be revised to	VEGP-VOL-CH14 response to

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
				read: c. the approved preoperational and startup test procedures (including the site-specific startup administration manual (procedure) prior to initiating the plant initial test program) in accordance with FSAR Subsection 14.2.3.	item 4 SNC Ltr ND-10-1993
8127	Pt 10		LC#06	4. COLA Part 10, Proposed License Conditions, LC#6, Operational Program Readiness, will be revised to include a new line item for availability of documentation of plant calorimetric uncertainty methodology as follows (where # is the next appropriate letter designation): #. the availability of documented instrumentation uncertainties to calculate a power calorimetric uncertainty, prior to initial fuel load. #. the availability of administrative controls to implement maintenance and contingency activities related to the power calorimetric uncertainty instrumentation, prior to initial fuel load.	COL-SER-OI-CH15 S3 response to SER-OI-15.00-001 item 4 SNC Ltr ND-10-2091
8024	Pt 10		LC#07	5. COLA Part 10, Proposed License Conditions, including ITAAC, proposed License Condition #7, First-Plant-Only and First-Three-Plant-Only Testing, will be revised to read: 7. First-Plant-Only and First-Three-Plant-Only Testing Certain design features of the AP1000 plant will be subjected to special tests to establish unique phenomenological performance parameters of the AP1000 design. Because of the standardization of the AP1000 design, these special tests (designated as first-plant-only tests and first-three-plant-only tests) are not required on subsequent plants. Once these tests are completed by the first plant (or first three plants) and appropriate documentation identified, the subsequent plants need only reference the applicable documentation to show that the first plant (or first three plants) completed the required testing. Accordingly, the following license condition is proposed: First-Plant-Only and First-Three-Plant-Only Testing A licensee shall provide written identification of the applicable references for documentation for the completion of the testing to the Director of the Office of New Reactors (or equivalent NRC management) within thirty (30) calendar days of the licensee confirmation of acceptable test results. Subsequent plant licensees crediting completion of testing by the first-plant or by the first-three plants shall provide a report referencing the applicable documentation identified by the first (or first three) plant(s) confirming the testing to the Director of the Office of New Reactors (or equivalent NRC management). This report shall be provided to NRC either prior to initiation of pre-operational testing, or within sixty (60) days of the identification of the documentation for the completion of the testing by the first plant (or third plant, as appropriate), whichever is later.	VEGP-VOL-CH14 response to item 5 SNC Ltr ND-10-1993
8539	Pt 10		LC#07	COLA Part 10, Proposed License Conditions, including ITAAC, the title of the proposed License Condition #7, First-Plant-Only and First-Three-Plant-Only Testing, will be revised to read (with BOLD text): 7. FIRST-PLANT-ONLY AND FIRST-THREE-PLANT-ONLY TESTING:	Editorial formatting of VEGP-VOL-CH14 response to item 5 SNC Ltr ND-10-1993
8025	Pt 10		LC#09	6. COLA Part 10, Proposed License Conditions, including ITAAC, proposed License Condition #9, Power-Ascension Test Phase, will be revised to address the complete startup testing program, with additions for pre-operational testing, and for above 5% up to and including 100% RTP to read: 9. Startup Program Test Results Certain milestones within the startup testing phase of the initial test program (i.e., pre-critical testing, criticality testing, and low-power <5% RTP testing) are controlled through license conditions to ensure that relevant test results are reviewed, evaluated, and approved by the designated licensee management before proceeding with the power ascension test phase. Accordingly, the following license conditions are proposed: Pre-operational Testing Following completion of pre-operational testing, the licensee shall review and evaluate individual test results. Test exceptions or results which do not meet acceptance criteria are identified to the affected and responsible organizations, and corrective actions and retests, as required, are performed. Pre-critical and Criticality Testing 1. Following completion of pre-critical and criticality testing, the licensee shall review and evaluate individual test results. Test exceptions or results which do not meet acceptance criteria are identified to the affected and responsible organizations, and corrective actions and retests, as required, are performed. 2. The licensee shall provide written notification to the Director of the Office of New Reactors (or equivalent NRC management) within fourteen (14) calendar days of completion of the pre-critical and criticality testing. Low-Power <5% RTP Testing 1. Following completion of low-power testing <5% RTP, the licensee shall review and evaluate individual test results. Test exceptions or results which do not meet acceptance criteria are identified to the affected and responsible organizations, and corrective actions and retests, as required, are performed. 2. The licensee shall provide written notification to the Director of the Office of New Reactors (or equivalent NRC management) within fourteen (14) calendar days of completion of the low-power testing.	VEGP-VOL-CH14 response to item 6 SNC Ltr ND-10-1993

QB Change ID#	COLA Part A	Chapter A	Section / Page A	Change Summary	Basis for Change
				At-Power (5%-100% RTP) Testing 1. Following completion of at-power testing (at or above 5% RTP up to and including testing at 100% RTP), the licensee shall review and evaluate individual test results. Test exceptions or results which do not meet acceptance criteria are identified to the affected and responsible organizations, and corrective actions and retests, as required, are performed. 2. The licensee shall provide written notification to the Director of the Office of New Reactors (or equivalent NRC management) within fourteen (14) calendar days of completion of the at-power testing.	
8540	Pt 10		LC#09	COLA Part 10, Proposed License Conditions, including ITAAC, the title of the proposed License Condition #9, Power-Ascension Test Phase, will be revised To read (with BOLD text): 9. STARTUP PROGRAM TEST RESULTS:	Editorial formatting of VEGP-VOL-CH14 response to item 6 SNC Ltr ND-10-1993
8128	Pt 10		LC#AppB PS ITAAC 2.5.4	5. COLA Part 10, Appendix B, will be revised to include a new Plant-Specific ITAAC line item for COL item 15.0-1 as follows: Add the following information to the information provided in the referenced DCD Tier 1 Section 2.5.4, as a new item 4 under the Design Description section: 4. The plant operating instrumentation installed for feedwater flow measurement is one that has been specifically approved by the NRC; the power calorimetric uncertainty calculation includes uncertainties for the associated instrumentation based on an NRC approved methodology; and the calculated calorimetric values are bounded by the uncertainty value assumed for the initial reactor power in the safety analysis.	COL-SER-OI-CH15 S3 response to SER-OI-15.00-001 item 5 SNC Ltr ND-10-2091
8129	Pt 10		LC#AppB PS ITAAC 2.5.4, T2.5.4-2	6. COLA Part 10, Appendix B, will be revised to include a new Plant-Specific ITAAC line item for COL item 15.0-1 as follows: Add the following information to the information provided in the referenced DCD Tier 1 Section 2.5.4, as a new, final line item in Table 2.5.4-2. Refer to the response letter for the complete change.	COL-SER-OI-CH15 S3 response to SER-OI-15.00-001 item 6 SNC Ltr ND-10-2091
8138	Pt 10		LC#AppB- EmPl	3. COLA Part 10, Appendix B, Inspections, Tests, Analyses and Acceptance Criteria, Emergency Planning ITAAC, will be modified by adding a new entry as shown below: Add the following emergency planning acceptance criteria to item 8.1.1.D.2, Exercises and Drills, of the Unit 3 emergency planning ITAAC included in Appendix E of Early Site Permit ESP-004: d. Demonstrate the capability of TSC and EOF equipment and data displays to clearly identify and reflect the affected unit	VEGP-P10-VOL-CH18 TSC ACRS item 3 SNC Ltr ND-10-2086
Pt 11 - (empty)					4 COLA Changes
5852	Pt 11		B- MSDP	2. Change COLA Part 11, Enclosures, by including the new Mitigative Strategies Description and Plans (as provided in Attachment B). Note: The actual document should be withheld from disclosure in accordance with 10 CFR 2.390(d), because it contains security-related information. Consequently, the version of this Mitigative Strategies Description and Plans in Part 11 is redacted, and the full document is provided in COLA Part 9, Withheld Information	Commitment made in response to LOLA Mitigative Strategies Description and Plans (see SNC Letter ND-09-0886)
8389	Pt 11		D- MC&A	7. COLA Part 11, Enclosures, will be revised by the addition of a new Enclosure 11.#, Special Nuclear Material Control and Accounting Program Description (where # is the next appropriate Appendix 11 letter designation). Enclosure 11.# is provided as Enclosure 2 to this letter. Include 11.# on Index Sheet for Part 11. (Refer to the final response letter for Enclosure 2)	VEGP-RAI-LTR-064 response to RAI 01.05-003 item 7 SNC Ltr ND-10-2257
8412	Pt 11		D- MC&A	Brackets removed from SNM MC&A Program Description in COLA Part 11D.	Southern letter ND-10-2257, dated November 23, 2010
8447	Pt 11		D- MC&A	Add Revision 0 to each page of the SNM MC&A Program Description in COLA Part 11D.	Editorial revision to submittal via SNC letter ND-10-2257, dated November 23, 2010