



Alfred M. Paglia, Jr.
Manager
Nuclear Licensing
New Nuclear Deployment

February 7, 2011
NND-11-0042

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

ATTN: Document Control Desk

Subject: Virgil C. Summer Nuclear Station (VCSNS) Units 2 and 3 Combined License Application (COLA) - Docket Numbers 52-027 and 52-028 Submittal of Roadmap of Changes for the February 7, 2011 Submittal of the COLA.

Reference: Letter from Ronald B. Clary (SCE&G) to Document Control Desk (NRC), February 7, 2011 Submittal of the Combined License Application for V.C. Summer Nuclear Station Units 2 and 3, Including Revision 4 of the Final Safety Analysis Report (FSAR).

By letter dated March 27, 2008, South Carolina Electric & Gas Company (SCE&G) submitted a combined license application (COLA) for two Westinghouse AP1000 units, designated V.C. Summer Nuclear Station (VCSNS) Units 2 and 3, to be located at the existing VCSNS site in Fairfield County, South Carolina. By letter dated February 7, 2011, SCE&G submitted an update to the VCSNS Units 2 and 3 COLA. Enclosed is a 'roadmap' of changes included in the February 7, 2011 update, which is being provided as a reviewer aid. Also provided is an explanation of the roadmap column headings.

Should you have any questions, please contact me by telephone at (803) 345-4191, or by email at apaglia@scana.com.

Very truly yours,

Alfred M. Paglia
Manager, Nuclear Licensing
New Nuclear Deployment

JEF/AMP/jf

D083
NRD

Document Control Desk
Page 2 of 2
NND-11-0042

Enclosure – COLA February 7, 2011 Submittal Roadmap

c: Joseph M. Sebrosky
Amy M. Monroe
Daniel Patton
Joel Hjelseth
William E. Hutchins
William A. Fox, III
Ron Wittschen
VCSummer2&3ProjectMail@Shawgrp.com
vcsummer2&3project@westinghouse.com
DCRM-EDMS@SCANA.COM

South Carolina Electric & Gas Company

NND-11-0042

Enclosure

VCSNS Units 2 and 3 COL Application

February 7, 2011 Submittal Roadmap

(31 Pages Including Coversheet and Column Index Page)

**VCSNS Units 2 & 3 COL Application February 7, 2011 Submittal Roadmap Format
Explanation (by columns)**

| Column Label | Explanation |
|-----------------------------|---|
| Change ID # | Unique internal identifier for tracking purposes |
| COLA REP | Identifies the change as STD (standard) or VCSNS (VCS) specific |
| COLA Part REP | Part 1 (PT01) through Part 16 (PT16) |
| Chapter REP | FSAR Chapter |
| Section / Page REP | Identifies specific location of the change in the COLA. Page numbers, if identified, are specific to the document that was revised, i.e., Revision 4. |
| Complete Change Description | Short description of the change |
| Basis for Change | The source of the change |

NuStart's COLA Tracking Management (CTM) : COLA Changes | Summer Rev 4

FEB-07-2011 10:56 AM

Summer Rev 4

Due Date is '03-31-2011' AND ...

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|-----------------------|----------|---------------|-------------|-------------------------------|---|---|
| PT01 - (empty) | | | | | | 3 COLA Changes |
| 8402 | VCS | PT01 | | COLA Part 1 Section 1.0 | COLA Part 1, Section 1.0, will be revised to add new COLA Part 16 as follows: Part 16 - COLA Enclosure 6 - Special Nuclear Material (SNM) Material Control and Accounting Program Description | SCE&G Letter NND-10-0448 |
| 8403 | VCS | PT01 | | COLA Part 1 Section 1.0 | COLA Part 1, Section 1.0, last sentence in the second paragraph will be revised to read (note, update to Revision 18 of the DCD): "This application presents descriptions and analyses of the final station design and incorporates by reference, Appendix D to 10 CFR Part 52 as required by Section III.B of that Appendix as amended by Revision 18." | DCD Revision 18 conforming change. Updated Revision 17 to Revision 18. |
| 9073 | VCS | PT01 | | COLA Part 1 Table 1 (Sheet 3) | COLA Part 1, Table 1 Sheet 3, will be revised to update the position titles for the following Santee Cooper Officials: 1) "Executive Vice President Chief Financial Officer" to "Executive Vice President and Chief Financial officer" 2) "Senior Vice President General Counsel" to "Executive Vice President and General Counsel" 3) "Senior Vice President Corporate Services" to "Executive Vice President of Corporate Services" | Updated position titles for Santee Cooper Officials |
| PT02 - FSAR01 | | | | | | 25 COLA Changes |
| 8275 | STD,VCS | PT02 | FSAR01 | 01.01 | COLA Part 2, FSAR Chapter 1, Section 1.1, Introduction, will be revised from: Unless otherwise specified, reference to the DCD refers to Tier 2 information. To read: Unless otherwise specified, reference to the DCD refers to Tier 2 information and includes the sensitive unclassified non-safeguards information (including proprietary information), and safeguards information referenced in the AP1000 DCD. Such DCD information is included in this combined license application in the same manner as it is included in the AP1000 DCD, i.e., references in the DCD are included as references in the FSAR, and material incorporated by reference into the DCD is incorporated by reference into the FSAR. Appropriate agreements are in place to provide access to the withheld sensitive unclassified non-safeguards information (including proprietary information), and safeguards information referenced in the AP1000 DCD. | VEGP-VOL-CH01 IBR of PI & SGI response item 1 SNC Ltr ND-10-2207 (SCE&G Endorsement Letter NND-10-0429) |
| 8379 | STD,VCS | PT02 | FSAR01 | 01.01 | COLA Part 2, FSAR Chapter 1, Section 1.1, Introduction, will be revised from: Throughout this FSAR, the "referenced DCD" is the AP1000 DCD submitted by Westinghouse as Revision 17 including any supplemental material as identified in Table 1.6-201. To read: Throughout this FSAR, the "referenced DCD" is the AP1000 DCD submitted by Westinghouse as Revision 18 including any supplemental material as identified in Table 1.6-201. | DCD Rev 18 |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change | | | | | | | | | | | | | | | | | | | | |
|--|--|----------------------------|-------------|-----------------------------|---|--|-------------------------------|----------------------------|---------------|---|---------|--|--|---|--|---------|--|--|--|---------|--|--|--|------|--|--|
| 8378 | STD,VCS | PT02 | FSAR01 | 01.06.T / T1.6-201 | COLA Part 2, FSAR Chapter 1, Section 1.6, Table 1.6-201, will be revised from: Westinghouse/APP-GW-GL-700 AP1000 Design Control Document 17 All September 2008 ML083230868 To read: Westinghouse/APP-GW-GL-700 AP1000 Design Control Document 18 All December 2010 ML103480572 | DCD Rev 18 | | | | | | | | | | | | | | | | | | | | |
| 8416 | VCS | PT02 | FSAR01 | 01.06.T/1.6-201 | In Table 1.6-201, change the revision of the VCSNS Emergency Plan from "Rev 3" to "Rev 4" and the submitted date to "February 2011". | Consistency with revised Part 5 update for COLA Rev 4 submittal | | | | | | | | | | | | | | | | | | | | |
| 8039 | STD,VCS | PT02 | FSAR01 | 01.08.T / T1.8-201 | COLA Part 2, FSAR Chapter 1, Table 1.8-201, Summary of FSAR Departures from the DCD, will be revised to add the following: <table border="0" style="width: 100%; border-collapse: collapse;"> <thead> <tr> <th style="text-align: left;">Departure Number</th> <th style="text-align: left;">Departure Description Summary</th> <th style="text-align: left;">FSAR Section or Subsection</th> </tr> </thead> <tbody> <tr> <td>STD DEP 8.3-1</td> <td>The Class 1E voltage regulating transformers do not have active components to limit current.</td> <td>8.3.2.2</td> </tr> </tbody> </table> | Departure Number | Departure Description Summary | FSAR Section or Subsection | STD DEP 8.3-1 | The Class 1E voltage regulating transformers do not have active components to limit current. | 8.3.2.2 | VEGP-VOL-CH08 response to STD-VOL-08.03-002 item 3 SNC Ltr ND-10-2005 (SCE&G Endorsement Letter NND-10-0381) | | | | | | | | | | | | | | |
| Departure Number | Departure Description Summary | FSAR Section or Subsection | | | | | | | | | | | | | | | | | | | | | | | | |
| STD DEP 8.3-1 | The Class 1E voltage regulating transformers do not have active components to limit current. | 8.3.2.2 | | | | | | | | | | | | | | | | | | | | | | | | |
| 7971 | STD,VCS | PT02 | FSAR01 | 01.08.T / T1.8-202 03.08-05 | COLA Part 2, FSAR Chapter 1, Section 1.8, Table 1.8-202, will be revised to add new COL item listing to read: <table border="0" style="width: 100%; border-collapse: collapse;"> <tbody> <tr> <td style="width: 30%;">3.8-5 Structures Inspection Program</td> <td style="width: 30%;">3.8.6.5</td> <td style="width: 10%;">3.8.3.7</td> <td style="width: 10%;">A</td> </tr> <tr> <td></td> <td></td> <td>3.8.4.7</td> <td></td> </tr> <tr> <td></td> <td></td> <td>3.8.5.7</td> <td></td> </tr> <tr> <td></td> <td></td> <td>3.8.6.5</td> <td></td> </tr> <tr> <td></td> <td></td> <td>17.6</td> <td></td> </tr> </tbody> </table> | 3.8-5 Structures Inspection Program | 3.8.6.5 | 3.8.3.7 | A | | | 3.8.4.7 | | | | 3.8.5.7 | | | | 3.8.6.5 | | | | 17.6 | | VEGP-VOL-Ch03 SIP response to STD COL 03.08-005 item 1 SNC Ltr ND-10-1594 (SCE&G Endorsement Letter NND-10-0340) |
| 3.8-5 Structures Inspection Program | 3.8.6.5 | 3.8.3.7 | A | | | | | | | | | | | | | | | | | | | | | | | |
| | | 3.8.4.7 | | | | | | | | | | | | | | | | | | | | | | | | |
| | | 3.8.5.7 | | | | | | | | | | | | | | | | | | | | | | | | |
| | | 3.8.6.5 | | | | | | | | | | | | | | | | | | | | | | | | |
| | | 17.6 | | | | | | | | | | | | | | | | | | | | | | | | |
| 8102 | STD,VCS | PT02 | FSAR01 | 01.08.T / T1.8-202 03.08-06 | COLA Part 2, FSAR Chapter 1, Section 1.8, Table 1.8-202, will be revised to add new COL item listing to read: <table border="0" style="width: 100%; border-collapse: collapse;"> <tbody> <tr> <td style="width: 30%;">3.8-6 Construction Procedures Program</td> <td style="width: 30%;">3.8.6.6</td> <td style="width: 10%;">3.8.6.6</td> <td style="width: 10%;">H</td> </tr> </tbody> </table> | 3.8-6 Construction Procedures Program | 3.8.6.6 | 3.8.6.6 | H | VEGP-VOL-CH03 Const Procedures response to STD-COL-03.08-006 item 1 SNC Ltr ND-10-1900 (SCE&G Endorsement Letter NND-10-0379) | | | | | | | | | | | | | | | | |
| 3.8-6 Construction Procedures Program | 3.8.6.6 | 3.8.6.6 | H | | | | | | | | | | | | | | | | | | | | | | | |
| 7962 | STD,VCS | PT02 | FSAR01 | 01.08.T / T1.8-202 05.02-03 | COLA Part 2, FSAR Chapter 1, Table 1.8-202, will be revised to add a new COL information item to read: <table border="0" style="width: 100%; border-collapse: collapse;"> <tbody> <tr> <td style="width: 30%;">5.2-3 Response to Unidentified Reactor Coolant System Leakage Inside Containment</td> <td style="width: 30%;">5.2.6.3</td> <td style="width: 10%;">5.2.6.3</td> <td style="width: 10%;">A</td> </tr> <tr> <td></td> <td></td> <td>5.2.5.3.5</td> <td></td> </tr> </tbody> </table> | 5.2-3 Response to Unidentified Reactor Coolant System Leakage Inside Containment | 5.2.6.3 | 5.2.6.3 | A | | | 5.2.5.3.5 | | VEGP-RAI-LTR-060 in response to RAI 05.02.05-001 item 1 SNC Ltr ND-10-1423 (SCE&G Endorsement Letter NND-10-0329) | | | | | | | | | | | | |
| 5.2-3 Response to Unidentified Reactor Coolant System Leakage Inside Containment | 5.2.6.3 | 5.2.6.3 | A | | | | | | | | | | | | | | | | | | | | | | | |
| | | 5.2.5.3.5 | | | | | | | | | | | | | | | | | | | | | | | | |
| 7967 | STD,VCS | PT02 | FSAR01 | 01.08.T / T1.8-202 05.03-07 | COLA Part 2, FSAR Chapter 1, Section 1.8, Table 1.8-202, will be revised to add new COL item listing to read: <table border="0" style="width: 100%; border-collapse: collapse;"> <tbody> <tr> <td style="width: 30%;">5.3-7 Quickloc Weld Build-up ISI</td> <td style="width: 30%;">5.3.6.6</td> <td style="width: 10%;">5.2.4.1</td> <td style="width: 10%;">A</td> </tr> <tr> <td></td> <td></td> <td>5.3.6.6</td> <td></td> </tr> </tbody> </table> | 5.3-7 Quickloc Weld Build-up ISI | 5.3.6.6 | 5.2.4.1 | A | | | 5.3.6.6 | | VEGP-VOL-CH05 ISI response to STD COL 05.03-007 item 1 SNC Ltr ND-10-1656 (SCE&G Endorsement Letter NND-10-0337) | | | | | | | | | | | | |
| 5.3-7 Quickloc Weld Build-up ISI | 5.3.6.6 | 5.2.4.1 | A | | | | | | | | | | | | | | | | | | | | | | | |
| | | 5.3.6.6 | | | | | | | | | | | | | | | | | | | | | | | | |
| 8466 | VCS | PT02 | FSAR01 | 01.08.T / T1.8- | COLA Part 2, FSAR Chapter 1, Table 1.8-202, will be revised to add new COL Item Number 19.59.10-6 | DCD Rev 18 and SCE&G Letter | | | | | | | | | | | | | | | | | | | | |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change | | | | | | | | | | |
|---------------|---|----------------|-------------------------|--|---|--|---------|----------------|--------------------|---|------------|--|------------|-------------------------|---|-------------|
| | | | | 202 19.59.10-6 | <p>as shown below:</p> <table border="1"> <thead> <tr> <th>COL ITEM</th> <th>SUBJECT</th> <th>DCD SUBSECTION</th> <th>FSAR SUBSECTION(S)</th> <th>COL APPLICANT (A), HOLDER (H), OR BOTH (B)</th> </tr> </thead> <tbody> <tr> <td>19.59.10-6</td> <td>Confirm that the Seismic Margin Assessment analysis is applicable to the COL site</td> <td>19.59.10.5</td> <td>19.55.6.3 19.59.10.5</td> <td>A</td> </tr> </tbody> </table> | COL ITEM | SUBJECT | DCD SUBSECTION | FSAR SUBSECTION(S) | COL APPLICANT (A), HOLDER (H), OR BOTH (B) | 19.59.10-6 | Confirm that the Seismic Margin Assessment analysis is applicable to the COL site | 19.59.10.5 | 19.55.6.3 19.59.10.5 | A | NND-10-0362 |
| COL ITEM | SUBJECT | DCD SUBSECTION | FSAR SUBSECTION(S) | COL APPLICANT (A), HOLDER (H), OR BOTH (B) | | | | | | | | | | | | |
| 19.59.10-6 | Confirm that the Seismic Margin Assessment analysis is applicable to the COL site | 19.59.10.5 | 19.55.6.3 19.59.10.5 | A | | | | | | | | | | | | |
| 8448 | STD,VCS | PT02 | FSAR01 | 01.09.T / T1.9-201 1.011 | <p>COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, is revised to include Regulatory Guide 1.11 to read:</p> <p>1.11 Instrument Lines Penetrating the Primary Reactor Containment (Rev. 1, March 2010) DCD discussion only; See DCD Table 1.9-1</p> | DCD Rev 18 | | | | | | | | | | |
| 7972 | STD,VCS | PT02 | FSAR01 | 01.09.T / T1.9-201 1.160 | <p>COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Regulatory Guide 1.160 will be revised to add the following new FSAR Subsection references prior to the existing FSAR Section reference of 17.6 (NEI 07-02A):</p> <p>3.8.3.7 3.8.4.7 3.8.5.7</p> | VEGP-VOL-Ch03 SIP response to STD COL 03.08-005 item 2 SNC Ltr ND-10-1594 (SCE&G Endorsement Letter NND-10-0340) | | | | | | | | | | |
| 7973 | STD,VCS | PT02 | FSAR01 | 01.09.T / T1.9-201 1.199 | <p>COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, Regulatory Guide 1.199 will be revised from:</p> <p>Not referenced; See Appendix 1AA</p> <p>To read:</p> <p>DCD discussion only; See DCD Table 1.9-1</p> | VEGP-VOL-Ch03 SIP response to STD COL 03.08-005 item 3 SNC Ltr ND-10-1594 (SCE&G Endorsement Letter NND-10-0340) | | | | | | | | | | |
| 8226 | STD,VCS | PT02 | FSAR01 | 01.09.T / T1.9-201 1.028 | <p>COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-201, information for Regulatory Guide 1.28, will be revised to add the following additional FSAR section reference:</p> <p>14.2.2.2</p> | VEGP-VOL-CH14 Qualification Req response item 1 SNC Ltr ND-10-2204 (SCE&G Endorsement Letter NND-10-0428) | | | | | | | | | | |
| 8736 | VCS | PT02 | FSAR01 | 01.09.T / T1.9-201 1.076 | In Table 1.9-201, Remove Subsection 3.5.1.4 from Regulatory Guide 1.76 | Consistency with RAI 03.05.01.04-2 (NRC Letter 054 Response per SCE&G Letter NND-10-0378 dated October 11, 2010) | | | | | | | | | | |
| 8395 | STD,VCS | PT02 | FSAR01 | 01.09.T / T1.9-204 | <p>COLA Part 2, FSAR Chapter 1, Section 1.9, Table 1.9-204, Bulletin Number 05-01, Material Control and Accounting at Reactors and Wet Spent Fuel Storage Facilities, will be revised from:</p> <table border="1"> <thead> <tr> <th>Number</th> <th>Title</th> <th>Comment</th> </tr> </thead> <tbody> <tr> <td>05-01</td> <td>Material Control and Accounting at Reactors and Wet Spent Fuel Storage Facilities</td> <td>13.6</td> </tr> </tbody> </table> <p>To read:</p> | Number | Title | Comment | 05-01 | Material Control and Accounting at Reactors and Wet Spent Fuel Storage Facilities | 13.6 | VEGP-RAI-LTR-064 response to RAI 01.05-003 item 1 SNC Ltr ND-10-2257 (SCE&G Endorsement NND-10-0448) | | | | |
| Number | Title | Comment | | | | | | | | | | | | | | |
| 05-01 | Material Control and Accounting at Reactors and Wet Spent Fuel Storage Facilities | 13.6 | | | | | | | | | | | | | | |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|---------------|----------|---------------|-------------|---------------------|--|---|
| | | | | | <p>Number Title Comment</p> <p>05-01 Material Control and Accounting at Reactors and Wet Spent Fuel Storage Facilities 13.5.2.2.9</p> | |
| 8289 | STD,VCS | PT02 | FSAR01 | 01.10.02 | <p>COLA Part 2, FSAR Chapter 1, Subsection 1.10.2, last paragraph, will be revised from:</p> <p>This assessment identified administrative and managerial controls to avoid impacts to SSCs from construction. The results of the assessment are presented in Table 1.10-202.</p> <p>To read:</p> <p>The initial assessment consisted of a review of individual SSCs and LCOs to determine whether an item is applicable, or may be eliminated due to either examination or being internal and specific to an operating unit. The assessment identified the SSCs that could reasonably be expected to be impacted by construction activities unless administrative and managerial controls are established. The results of the assessment are presented in Table 1.10-202. Periodic assessment during construction is addressed in Appendix 13AA, Subsection 13AA.1.1.1.1.8</p> | VEGP-RAI-LTR-063 response to RAI 01.05-002 item 1 SNC Ltr ND-10-2114 (SCE&G Endorsement Letter NND-10-0422) |
| 8290 | STD,VCS | PT02 | FSAR01 | 01.10.03 | <p>COLA Part 2, FSAR Chapter 1, Subsection 1.10.3, last paragraph, will be revised from:</p> <p>The above discussed controls to eliminate or mitigate construction hazards that could potentially impact operating unit SSCs important to safety are in place when there is an operating nuclear unit on the site.</p> <p>To read:</p> <p>The above discussed controls to eliminate or mitigate construction hazards that could potentially impact operating unit SSCs important to safety are in place when there is an operating nuclear unit on the site. Additional controls may be established during construction as addressed in Appendix 13AA, Subsection 13AA.1.1.1.1.8.</p> | VEGP-RAI-LTR-063 response to RAI 01.05-002 item 2 SNC Ltr ND-10-2114 (SCE&G Endorsement Letter NND-10-0422) |
| 8294 | STD,VCS | PT02 | FSAR01 | 01.10.T / T1.10-201 | <p>COLA Part 2, FSAR Chapter 1, Subsection 1.10, Table 1.10-201, will be revised from:</p> <p>Equipment and Material Laydown, Storage, Warehousing</p> <ul style="list-style-type: none"> • Releases of Stored Flammable, Hazardous or Toxic Materials <p>To read:</p> <p>Equipment and Material Laydown, Storage, Warehousing</p> <ul style="list-style-type: none"> • Releases of Flammable, Hazardous or Toxic Materials | VEGP-RAI-LTR-063 response to RAI 01.05-002 item 5 SNC Ltr ND-10-2114 (SCE&G Endorsement Letter NND-10-0422) |
| 8292 | STD,VCS | PT02 | FSAR01 | 01.10.T / T1.10-202 | <p>COLA Part 2, FSAR Chapter 1, Subsection 1.10, Table 1.10-202, will be revised to include the following new item:</p> <p>Impact of Local Flooding</p> <ul style="list-style-type: none"> • Safety-related structures, systems, and components (SSCs) | VEGP-RAI-LTR-063 response to RAI 01.05-002 item 4 SNC Ltr ND-10-2114 (SCE&G Endorsement Letter NND-10-0422) |
| 8295 | STD,VCS | PT02 | FSAR01 | 01.10.T / T1.10-203 | <p>COLA Part 2, FSAR Chapter 1, Subsection 1.10, Table 1.10-203, will be revised to include the following new item:</p> <p>Impact of Local Flooding</p> <ul style="list-style-type: none"> • Site grading and drainage provisions consider | VEGP-RAI-LTR-063 response to RAI 01.05-002 item 6 SNC Ltr ND-10-2114 (SCE&G Endorsement Letter NND-10- |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|----------------------|----------|---------------|-------------|---------------------|--|--|
| | | | | | potential flooding impacts from local intense precipitation | 0422) |
| 8296 | STD,VCS | PT02 | FSAR01 | 01.10.T / T1.10-203 | COLA Part 2, FSAR Chapter 1, Subsection 1.10, Table 1.10-203, will be revised to include the following new item: Impact of Site Groundwater Dewatering • Administrative controls address groundwater level monitoring | VEGP-RAI-LTR-063 response to RAI 01.05-002 item 7 SNC Ltr ND-10-2114 (SCE&G Endorsement Letter NND-10-0422) |
| 8449 | STD,VCS | PT02 | FSAR01 | 01AA 1.011 | COLA Part 2, FSAR Chapter 1, Appendix 1AA, is revised to include Regulatory Guide 1.11 to read: Regulatory Guide 1.11, Rev. 1, 3/10 – Instrument Lines Penetrating the Primary Reactor Containment Conformance with the design aspects is as stated in the DCD. This guidance is completely within the scope of the DCD. | DCD Rev 18 |
| 8712 | VCS | PT02 | FSAR01 | 01AA 1.028 | Due to table pagination, Revise entry for Regulatory Guide 1.28 Revision 3 to include LMA of VCS COL 1.9-1. | Consistency with placement of site specific LMAs on standard table entries. |
| 7974 | STD,VCS | PT02 | FSAR01 | 01AA 1.199 | 4. COLA Part 2, FSAR Chapter 1, Appendix 1AA, Regulatory Guide 1.199 will be revised from: NA This Regulatory Guide is not applicable to the AP1000 certified design. To read: Conformance with Revision 0 of the Regulatory Guide is as stated in the DCD. This guidance is completely within the scope of the DCD. | VEGP-VOL-Ch03 SIP response to STD COL 03.08-005 item 4 SNC Ltr ND-10-1594 (SCE&G Endorsement Letter NND-10-0340) |
| PT02 - FSAR03 | | | | | | 11 COLA Changes |
| 8467 | VCS | PT02 | FSAR03 | 03.05.01.04 | Based on the additional information and proposed AP1000 DCD changes provided to the NRC by WEC in RAI-COL03.05.01.04-1 R2, delete Section 3.5.1.4. | RAI 03.05.01.04-2 (NRC Letter 054 Response per SCE&G Letter NND-10-0378 dated October 11, 2010) |
| 7975 | STD,VCS | PT02 | FSAR03 | 03.08.03.07 | COLA Part 2, FSAR Chapter 3, will be revised to add new Subsection 3.8.3.7 (with an LMA of STD COL 3.8-5) to read: ----- 3.8.3.7 In-Service Testing and Inspection Requirements ----- Replace the existing DCD statement with the following: The inspection program for structures is identified in Section 17.6. This inspection program is consistent with the requirements of 10 CFR 50.65 and the guidance in Regulatory Guide 1.160. | VEGP-VOL-Ch03 SIP response to STD COL 03.08-005 item 5 SNC Ltr ND-10-1594 (SCE&G Endorsement Letter NND-10-0340) |
| 7976 | STD,VCS | PT02 | FSAR03 | 03.08.04.07 | COLA Part 2, FSAR Chapter 3, will be revised to add new Subsection 3.8.4.7 (with an LMA of STD COL 3.8-5) to read: ----- 3.8.4.7 Testing and In-Service Inspection Requirements | VEGP-VOL-Ch03 SIP response to STD COL 03.08-005 item 6 SNC Ltr ND-10-1594 (SCE&G Endorsement Letter NND-10-0340) |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|---------------|----------|---------------|-------------|------------------|---|--|
| | | | | | <p>-----</p> <p>Replace the existing DCD final statement of the subsection with the following:</p> <p>The inspection program for structures is identified in Section 17.6. This inspection program is consistent with the requirements of 10 CFR 50.65 and the guidance in Regulatory Guide 1.160.</p> | |
| 7977 | STD,VCS | PT02 | FSAR03 | 03.08.05.07 | <p>COLA Part 2, FSAR Chapter 3, will be revised to add new Subsection 3.8.5.7 (with an LMA of STD COL 3.8-5) to read:</p> <p>-----</p> <p>3.8.5.7 In-Service Testing and Inspection Requirements</p> <p>-----</p> <p>Replace the existing DCD first statement with the following:</p> <p>The inspection program for structures is identified in Section 17.6. This inspection program is consistent with the requirements of 10 CFR 50.65 and the guidance in Regulatory Guide 1.160.</p> | VEGP-VOL-Ch03 SIP response to STD COL 03.08-005 item 7 SNC Ltr ND-10-1594 (SCE&G Endorsement Letter NND-10-0340) |
| 7978 | STD,VCS | PT02 | FSAR03 | 03.08.06.05 | <p>COLA Part 2, FSAR Chapter 3, will be revised to add new Subsection 3.8.6.5 (with an LMA of STD COL 3.8-5) to read:</p> <p>-----</p> <p>3.8.6.5 Structures Inspection Program</p> <p>-----</p> <p>This item is addressed in Subsections 3.8.3.7, 3.8.4.7, 3.8.5.7, and 17.6.</p> | VEGP-VOL-Ch03 SIP response to STD COL 03.08-005 item 8 SNC Ltr ND-10-1594 (SCE&G Endorsement Letter NND-10-0340) |
| 8103 | STD,VCS | PT02 | FSAR03 | 03.08.06.06 | <p>COLA Part 2, FSAR Chapter 3, will be revised to add new Subsection 3.8.6.6 (with an LMA of STD COL 3.8-6) to read:</p> <p>-----</p> <p>3.8.6.6 Construction Procedures Program</p> <p>-----</p> <p>Add the following to the end of Subsection 3.8.6.6:</p> <p>Construction and inspection procedures for concrete filled steel plate modules address activities before and after concrete placement, use of construction mock-ups, and inspection of modules before and after concrete placement as discussed in DCD Subsection 3.8.4.8. The procedures will be made available to NRC inspectors prior to use.</p> | VEGP-VOL-CH03 Const Procedures response to STD-COL-03.08-006 item 2 SNC Ltr ND-10-1900 (SCE&G Endorsement Letter NND-10-0379) |
| 8468 | VCS | PT02 | FSAR03 | 03.08.06.06 | <p>Editorial Correction to QB8103: Add "DCD" to the lead-in statement to read:</p> <p>Add the following to the end of DCD Subsection 3.8.6.6:</p> | Editorial Correction to QB8103 and VEGP-VOL-CH03 Const Procedures response to STD-COL-03.08-006 item 2 SNC Ltr ND-10-1900 (SCE&G Endorsement Letter NND-10-0379) |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|----------------------|----------|---------------|-------------|------------------|---|---|
| 7961 | STD,VCS | PT02 | FSAR03 | 03.09.03.01.02 | <p>COLA Part 2, FSAR Chapter 3, Subsection 3.9.3.1.2, will be revised under the heading of Locations to be Monitored, from:</p> <p>In addition to the existing permanent plant temperature instrumentation, temperature and displacement monitoring will be included at critical locations on the surge line.</p> <p>To read:</p> <p>In addition to the existing permanent plant temperature instrumentation, temperature and displacement monitoring will be included at critical locations on the surge line. The additional locations utilized for monitoring during the hot functional testing and the first fuel cycle (see Subsection 14.2.9.2.22) are selected based on the capability to provide effective monitoring.</p> | VEGP-RAI-LTR 057 S1 response to RAI 03.12-002 item 5 SNC Ltr ND-10-1501 (SCE&G Endorsement Letter NND-10-0328) |
| 8747 | VCS | PT02 | FSAR03 | 3.09.06.02.02 | Revise the 7th insertion statement to remove the bullet from the paragraph beginning "Active MOV Test Frequency Determination" | Consistency with formatting of standard content |
| 8748 | VCS | PT02 | FSAR03 | 3.09.06.02.02 | Revise the 8th insertion section to remove the lead-in statement, LMA, and horizontal spacer bar above and below the section. | Consistency with formatting of standard content |
| 9024 | VCS | PT02 | FSAR03 | 3.09.06.02.05 | Remove the LMA STD COL 3.9-4 from Subsection 3.9.6.2.5. The LMA is not needed since it is established in Subsection 3.9.6.2.4 | Consistency with formatting of standard content |
| PT02 - FSAR05 | | | | | | 4 COLA Changes |
| 7968 | STD,VCS | PT02 | FSAR05 | 05.02.04.01 | <p>COLA Part 2, FSAR Chapter 5, will be revised to add the following new paragraph at the end of the portion of Subsection 5.2.4.1 with an LMA of STD COL 5.3-7, to read:</p> <p>The in-service inspection program is augmented to include the performance of a 100 percent volumetric examination of the weld build-up on the reactor vessel head for the instrumentation penetrations (Quickloc) conducted once during each 120-month inspection interval in accordance with the ASME Code, Section XI. The weld build-up acceptance standards are those provided in ASME Code, Section XI, IWB-3514. Personnel performing examinations and the ultrasonic examination systems are qualified in accordance with ASME Code, Section XI, Appendix VIII. Alternatively, an alternative inspection may be developed in conjunction with the voluntary consensus standards bodies (i.e., ASME) and submitted to the NRC for approval.</p> | VEGP-VOL-CH05 ISI response to STD COL 05.03-007 item 2 SNC Ltr ND-10-1656 (SCE&G Endorsement Letter NND-10-0337) |
| 7963 | STD,VCS | PT02 | FSAR05 | 05.02.05.03.05 | <p>COLA Part 2, FSAR Chapter 5, will be revised to add a new Subsection 5.2.5.3.5 (with an LMA of STD COL 5.2-3) to read:</p> <p>-----</p> <p>Add the following new subsection following DCD Subsection 5.2.5.3.4.</p> <p>5.2.5.3.5 Response to Reactor Coolant System Leakage</p> <p>Operating procedures specify operator actions in response to prolonged low level unidentified reactor coolant leakage conditions that exist above normal leakage rates and below the Technical Specification (TS) limits to provide operators sufficient time to take action before the TS limit is reached. The procedures include identifying, monitoring, trending, and addressing prolonged low level leakage. The procedures for effective management of leakage, including low level leakage, are developed including the following operations related activities:</p> <ul style="list-style-type: none"> • Trends in the unidentified leakage rates are periodically analyzed. When the leakage rate increases noticeably from the baseline leakage rate, the safety significance of the leak is evaluated. The rate of increase in the leakage is determined to verify that plant actions can be taken before the plant exceeds TS limits. | VEGP-RAI-LTR-060 in response to RAI 05.02.05-001 item 2 SNC Ltr ND-10-1423 (SCE&G Endorsement Letter NND-10-0329) |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|----------------------|----------|---------------|-------------|------------------|--|--|
| | | | | | <ul style="list-style-type: none"> Procedures are established for responding to leakage. These procedures address the following considerations to prevent adverse safety consequence results from the leakage: <ul style="list-style-type: none"> Plant procedures specify operator actions in response to leakage rates less than the limits set forth in the Technical Specifications. The procedures include actions for confirming the existence of a leak, identifying its source, increasing the frequency of monitoring, verifying the leakage rate (through a water inventory balance), responding to trends in the leakage rate, performing a walkdown outside containment, planning a containment entry, adjusting alarm setpoints, limiting the amount of time that operation is permitted when the sources of the leakage are unknown, and determining the safety significance of the leakage. Plant procedures specify the amount of time the leakage detection and monitoring instruments (other than those required by Technical Specifications) may be out of service to effectively monitor the leakage rate during plant operation (i.e., hot shutdown, hot standby, startup, transients, and power operation). The output and alarms from leakage monitoring systems are provided in the main control room. Procedures are readily available to the operators for converting the instrument output to a common leakage rate. (Alternatively, these procedures may be part of a computer program so that the operators have a real-time indication of the leakage rate as determined from the output of these monitors.) Periodic calibration and testing of leakage monitoring systems are conducted. The alarm(s), and associated setpoint(s), provide operators an early warning signal so that they can take corrective actions, as discussed above, i.e., before the plant exceeds TS limits. During maintenance and refueling outages, actions are taken to identify the source of any unidentified leakage that was detected during plant operation. In addition, corrective action is taken to eliminate the condition resulting in the leakage. <p>The procedures described above will be available prior to fuel load.</p> | |
| 7964 | STD,VCS | PT02 | FSAR05 | 05.02.06.03 | <p>COLA Part 2, FSAR Chapter 5, will be revised to add a new Subsection 5.2.6.3 (with an LMA of STD COL 5.2-3) to read:</p> <p>-----</p> <p>5.2.6.3 Response to Unidentified Reactor Coolant System Leakage Inside Containment</p> <p>-----</p> <p>This COL item is addressed in Subsection 5.2.5.3.5.</p> | <p>VEGP-RAI-LTR-060 in response to RAI 05.02.05-001 item 3 SNC Ltr ND-10-1423 (SCE&G Endorsement Letter NND-10-0329)</p> |
| 7969 | STD,VCS | PT02 | FSAR05 | 05.03.06.06 | <p>COLA Part 2, FSAR Chapter 5, will be revised to add new Subsection 5.3.6.6 (with an LMA of STD COL 5.3-7) to read:</p> <p>-----</p> <p>5.3.6.6 Quickloc Weld Build-up ISI</p> <p>-----</p> <p>This item is addressed in Subsection 5.2.4.1.</p> | <p>VEGP-VOL-CH05 ISI response to STD COL 05.03-007 item 3 SNC Ltr ND-10-1656 (SCE&G Endorsement Letter NND-10-0337)</p> |
| PT02 - FSAR06 | | | | | | 9 COLA Changes |
| 7958 | STD,VCS | PT02 | FSAR06 | 06.01.02.01.06 | <p>COLA Part 2, FSAR Chapter 6, Section 6.1.2.1.6, will be revised to include the following new information after the existing fourth paragraph just after the Service Level I and Service Level III</p> | <p>VEGP-VOL-Ch06 Coatings in response to STD COL 06.01-</p> |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|---------------|----------|---------------|-------------|----------------------------|--|--|
| | | | | | discussions (the LMA of STD COL 6.1-2 remains unchanged): Include a new second paragraph under the subsection titled "Service Level II Coatings" within DCD Subsection 6.1.2.1.6 with the following information. Such safety-related Service Level II coatings used inside containment are procured to the same standards as Service Level I coatings with regard to radiation tolerance and performance under design basis accident conditions as discussed below. | 002 SNC Ltr ND-10-1566 (SCE&G Endorsement Letter NND-10-0338) |
| 7959 | STD,VCS | PT02 | FSAR06 | 06.01.02.01.06 | COLA Part 2, FSAR Chapter 6, Section 6.1.2.1.6, as revised per Qb7782 will be revised to remove the term "safety related" from: Such safety-related Service Level II coatings used inside containment are procured to the same standards as Service Level I coatings with regard to radiation tolerance and performance under design basis accident conditions as discussed below. To read: Such Service Level II coatings used inside containment are procured to the same standards as Service Level I coatings with regard to radiation tolerance and performance under design basis accident conditions as discussed below. | Correction to remove "safety-related" designation from changes in VEGP-VOL-Ch06 Coatings in response to STD COL 06.01-002 SNC Ltr ND-10-1566. Only the procurement is per Appendix B. (SCE&G Endorsement Letter NND-10-0338) |
| 8687 | VCS | PT02 | FSAR06 | 06.01.02.01.06 | COLA Part 2, FSAR Chapter 6, Subsection 6.1.2.1.6 is revised to move the LMA STD COL 6.1-2 from the lead in statement to the first paragraph of inserted text. | Editorial consistency with placement of LMAs on inserted text and not on lead-in statements |
| 8450 | STD,VCS | PT02 | FSAR06 | 06.04.07 | COLA Part 2, FSAR Chapter 6, Subsection 6.4.7 is revised to include an additional LMA of STD COL 6.4-1. | Consistency to match the LMAs used in the sections where the item is addressed, particularly 6.4.4 |
| 8009 | STD,VCS | PT02 | FSAR06 | 06.04.T / 6.4-201 Footnote | COLA Part 2, FSAR Chapter 6, Section 6.4, Table 6.4-201, will be revised to change the footnote for MCR 'from: MCR -Chemicals with an Impact Evaluation designation of "MCR" indicates the evaluation of this chemical considered additional design details of the main control room (beyond IH) such as volume, envelope boundaries, ventilation systems, and occupancy factor. To read MCR -Chemicals with an Impact Evaluation designation of "MCR" indicates the evaluation of this chemical considered design details of the main control room such as volume, envelope boundaries, ventilation systems, and occupancy factor. | VEGP-RAI LTR 061 response to RAI 06.04-005 item 4 SNC Ltr ND-10-1721 (SCE&G Endorsement Letter NND-10-0376) |
| 8002 | STD,VCS | PT02 | FSAR06 | 06.04.T / 6.4-201 Part A | COLA Part 2, FSAR Chapter 6, Section 6.4, standard portion of table of toxic chemical evaluations (VCS Table 6.4-201) as modified by the response to BLN-RAI-LTR-169 will be further revised in the Standard Onsite Toxic Chemicals listing for the Hydrogen Gas from "Corner of the Auxiliary and Turbine buildings" to read "Yard at turbine building" in the "Evaluated Location" column. | VEGP-VOL-Ch06 re toxic chemicals response to STD-COL-06.04-001 item 1 SNC Ltr ND-10-1473 (SCE&G Endorsement Letter NND-10-0331) |
| 8003 | STD,VCS | PT02 | FSAR06 | 06.04.T / 6.4-201 Part A | COLA Part 2, FSAR Chapter 6, Section 6.4, standard portion of table of toxic chemical evaluations (VCS Table 6.4-201) as modified by the response to BLN-RAI-LTR-169 will be further revised in the Standard Onsite Toxic Chemicals listing for the Hydrogen Liquid from "2000 gal" to read "1500 gal" in the "Evaluated Maximum Quantity" column. | VEGP-VOL-Ch06 re toxic chemicals response to STD-COL-06.04-001 item 2 SNC Ltr ND-10-1473 (SCE&G Endorsement Letter NND-10-0331) |
| 8007 | STD,VCS | PT02 | FSAR06 | 06.04.T / 6.4-201 | COLA Part 2, FSAR Chapter 6, Section 6.4, Table 6.4-201 Part A, will be revised for the standard | VEGP-RAI LTR 061 response |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|----------------------|----------|---------------|-------------|---------------------------|--|--|
| | | | | Part A | chemicals of hydrogen (liquid and gas), nitrogen, and carbon dioxide, to change the MCR Habitability Impact Evaluation from IH to MCR. | to RAI 06.04-005 item 2 SNC Ltr ND-10-1721 (SCE&G Endorsement Letter NND-10-0376) |
| 8004 | STD,VCS | PT02 | FSAR06 | 06.04.T / T6.4-201 Part A | COLA Part 2, FSAR Chapter 6, Section 6.4, Table 6.4-201 Part A, will be revised for the standard chemicals of hydrogen (liquid), nitrogen, and carbon dioxide, to change the Evaluated Minimum Distance to MCR Intake from 814 ft to 577 ft. | VEGP-RAI LTR 061 response to RAI 06.04-005 item 1 SNC Ltr ND-10-1721 (SCE&G Endorsement Letter NND-10-0376) |
| PT02 - FSAR07 | | | | | | 1 COLA Change |
| 8688 | VCS | PT02 | FSAR07 | 7.5 | COLA Part 2, FSAR Chapter 7, Subsection 7.5 is revised to add horizontal separator bars after the section headings of Subsections 7.52, 7.535, and 7.55. | Editorial consistency with placement of separator bars above and below DCD section headings. |
| PT02 - FSAR08 | | | | | | 2 COLA Changes |
| 8037 | STD,VCS | PT02 | FSAR08 | 08.03.02.01.04 | COLA Part 2, FSAR Chapter 8, Subsection 8.3.2.1.4, Maintenance and Testing, will be revised to add the following as the last paragraph with LMA STD COL 8.3-2: Procedures are established for periodic testing of the Class 1E battery chargers and Class 1E voltage regulating transformers in accordance with the manufacturer recommendations. <ul style="list-style-type: none"> • Circuit breakers in the Class 1E battery chargers and Class 1E voltage regulating transformers that are credited for an isolation function are tested through the use of breaker test equipment. This verification confirms the ability of the circuit to perform the designed coordination and corresponding isolation function between Class 1E and non-Class 1E components. Circuit breaker testing is done as part of the Maintenance Rule program and testing frequency is determined by that program. • Fuses / fuse holders that are included in the isolation circuit are visually inspected. • Class 1E battery chargers are tested to verify current limiting characteristic utilizing manufacturer recommendation and industry practices. Testing frequency is in accordance with that of the associated battery. | VEGP-VOL-CH08 response to STD-VOL-08.03-002 item 1 SNC Ltr ND-10-2005 (SCE&G Endorsement Letter NND-10-0381) |
| 8038 | STD,VCS | PT02 | FSAR08 | 08.03.02.02 | COLA Part 2, FSAR Chapter 8, will be revised to add new Section 8.3.2.2 to read: 8.3.2.2 Analysis STD DEP 8.3-1 Replace the first sentence of the third paragraph of DCD Subsection 8.3.2.2 with the following: The Class 1E battery chargers are designed to limit the input (ac) current to an acceptable value under faulted conditions on the output side, however, the voltage regulating transformers do not have active components to limit current; therefore, the Class 1E voltage regulating transformer maximum current is determined by the impedance of the transformer. | VEGP-VOL-CH08 response to STD-VOL-08.03-002 item 2 SNC Ltr ND-10-2005 (SCE&G Endorsement Letter NND-10-0381) |
| PT02 - FSAR09 | | | | | | 3 COLA Changes |
| 8104 | STD,VCS | PT02 | FSAR09 | 09.05.04.05.02 | COLA Part 2, FSAR Chapter 9, Subsection 9.5.4.5.2, second paragraph, the word "kinetic" will be revised to "kinematic" to match the required ASTM testing. | Editorial - Provide appropriate description of the testing |
| 8741 | VCS | PT02 | FSAR09 | 09.05.T / T9.5-201 033 | COLA Part 2, FSAR Chapter 9, Table 9.5-201, item 33, will be revised under the column Remarks from: Subsection 9.5.1.8.2.2 and DCD Subsection 6.4.3.1 address these requirements. | Provide appropriate DCD references for control room |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change | | | | | | | | | | |
|----------------------|------------------------------|---------------|------------------|-----------------------------|--|---|------------------------------|--------------|-----------|-------------|--------------------|------------|--|------------------|--------|--------------------------|
| | | | | | To read: Subsection 9.5.1.8.2.2 and DCD Subsections 6.4.2.3 and 6.4.4 address these requirements. | personnel breathing air source | | | | | | | | | | |
| 8732 | VCS | PT02 | FSAR09 | App A | Revise Appendix 9A Fire Protection to include the following new subsection with LMA VCS DEP 18.8-1: 9A.2 Fire Protection Methodology ----- 9A.2.1 Fire Area Description Add the following information at the end of the first paragraph in DCD Subsection 9A.2.1: Figure 9A-201 replaces DCD Figure 9A-3 (Sheet 1), to reflect the relocation of the Operations Support Center. | Consistency with replacement of DCD Figure 9A-3 with Figure 9A-201. | | | | | | | | | | |
| PT02 - FSAR11 | | | | | | 2 COLA Changes | | | | | | | | | | |
| 8451 | STD,VCS | PT02 | FSAR11 | 11.04.07 | COLA Part 2, FSAR Chapter 11, Subsection 11.4.7, Reference 201, is revised to include the ADAMS number in the reference from: ...March 2009. To read: ...March 2009 (ML091460627). | Editorial addition | | | | | | | | | | |
| 8452 | STD,VCS | PT02 | FSAR11 | 11.05.09 | COLA Part 2, FSAR Chapter 11, Subsection 11.5.9, Reference 202, is revised to include the ADAMS number in the reference from: ...March 2009. To read: ...March 2009 (ML091050234). | Editorial addition | | | | | | | | | | |
| PT02 - FSAR12 | | | | | | 2 COLA Changes | | | | | | | | | | |
| 8119 | STD,VCS | PT02 | FSAR12 | 12.02.01.01.10 | COLA Part 2, FSAR Chapter 12, Subsection 12.2.1.1.10, Miscellaneous Sources, will be revised to include a new final paragraph to read: During the period prior to the implementation of the Emergency Plan (in preparation for the initial fuel loading following the 52.103(g) finding), no specific materials related emergency plan will be necessary because: a) No byproduct material will be received, possessed, or used in a physical form that is "in unsealed form, on foils or plated sources, or sealed in glass," that exceeds the quantities in Schedule C in 10 CFR 30.72, and b) The source material to be received, possessed, or used does not involve uranium hexafluoride in excess of 50 kilograms in a single container or 1000 kilograms total. | VEGP-RAI-LTR-062 response to RAI 01.05-001 item 2 SNC Ltr ND-10-2002 (SCE&G Endorsement Letter NND-10-0440) | | | | | | | | | | |
| 7685 | VCS | PT02 | FSAR12 | 12.04.01.09.02 | Revise the 3rd sentence of the 2nd paragraph of Subsection 12.4.1.9.2 to read: "Small quantities of monitored airborne effluents are normally released from the Unit 1 waste gas decay tank, reactor building purges, and oil incineration." Note removal of extra "from the" after "Unit 1" | Grammatical Correction to sentence. Note removal of extra "from the" after "Unit 1" | | | | | | | | | | |
| PT02 - FSAR13 | | | | | | 8 COLA Changes | | | | | | | | | | |
| 7989 | VCS | PT02 | FSAR13 | 13.04.T / T13.4-201 Item 14 | VCSNS Part 2, FSAR, Section 13.4, Table 13.4-201, Item 14 will be revised to add a new entry (with a left margin annotation of VCS SUP 13.4-1) as follows: <table border="1"> <thead> <tr> <th>Program Title</th> <th>Program Source (Required By)</th> <th>FSAR Section</th> <th>Milestone</th> <th>Requirement</th> </tr> </thead> <tbody> <tr> <td>Emergency Response</td> <td>10 CFR 50,</td> <td></td> <td>Applicant's ERDS</td> <td>10 CFR</td> </tr> </tbody> </table> | Program Title | Program Source (Required By) | FSAR Section | Milestone | Requirement | Emergency Response | 10 CFR 50, | | Applicant's ERDS | 10 CFR | SCE&G Letter NND-10-0312 |
| Program Title | Program Source (Required By) | FSAR Section | Milestone | Requirement | | | | | | | | | | | | |
| Emergency Response | 10 CFR 50, | | Applicant's ERDS | 10 CFR | | | | | | | | | | | | |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|---------------|----------|---------------|-------------|-----------------------------|--|---|
| | | | | | Data System (ERDS) Appendix E implementation Part 50, Implementation Program at least 180 days prior to scheduled date for initial loading of fuel Appendix E, Section VI.4.a | |
| 8120 | STD,VCS | PT02 | FSAR13 | 13.04.T / T13.4-201 Item 14 | COLA Part 2, FSAR Chapter 13, Section 13.4, Table 13.4-201, item 14, Emergency Planning, will be revised to remove the following information related to materials. (portions applicable to radioactive material) 10 CFR 30.32(i)(3) Prior to initial receipt of 10 CFR 30.32(i)(1) byproduct, source, or special nuclear materials 10 CFR 40.310(3) (excluding Exempt Quantities as described in 10 CFR 70.22(i)(3)) 10 CFR 70.22(i)(1) | VEGP-RAI-LTR-062 response to RAI 01.05-001 item 1 SNC Ltr ND-10-2002 (SCE&G Endorsement Letter NND-10-0440) |
| 8272 | STD,VCS | PT02 | FSAR13 | 13.04.T / T13.4-201 Item 15 | COLA Part 2, FSAR, Chapter 13, Section 13.4, Table 13.4-201, Item 15, Security Program, will be revised. Item Program Title Program Source FSAR Milestone Requirement Section 15. Security Program Physical Security Program 10 CFR 73.55(b); 13.6 Prior to Receipt of 10CFR 73.55(a)(4) 10 CFR 73.55(c)(3); fuel onsite 10 CFR 73.56; (protected area) 10 CFR 73.57; (portions applicable to radioactive material) 10 CFR 30.34 Prior to initial receipt of byproduct, source, or special nuclear materials (excluding Exempt Quantities as described in 10CFR 30.18) 10 CFR 40.41 10CFR40.31(a) 10 CFR 73.1 Safeguards Contingency Program 10CFR73.55(c)(5); 13.6 Prior to receipt of fuel onsite 10CFR73.55(a)(4) 10CFR73.55(k); (protected area) 10 CFR Part 73, Appendix C Training and Qualification Program 10CFR73.55(c)(4); 13.6 Prior to receipt of fuel onsite (protected area) 10CFR73.55(d)(3); 10CFR Part 73, Appendix B | VEGP-RAI-LTR 051 S2 item 1 SNC Ltr ND-10-2040 (SCE&G Endorsement Letter NND-10-0431) |
| 8273 | STD,VCS | PT02 | FSAR13 | 13.04.T / T13.4-201 Item 21 | COLA Part 2, FSAR, Chapter 13, Section 13.4, Table 13.4-201, item 21, Cyber Security Program, will be revised from: Item Program Title Program Source FSAR Milestone Requirement Section 21. Cyber Security Program 10 CFR 73.54 13.6 Prior to receipt of fuel onsite License Condition | VEGP-RAI-LTR 051 S2 item 2 SNC Ltr ND-10-2040 (SCE&G Endorsement Letter NND-10-0431) |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change | | | | | | |
|---------------|---------------|------------------------------|--------------|-----------------------------|--|---|---------------|------------------------------|--------------|--------------------------|-------------|--|
| | | | | | <p>(protected area)</p> <p>To read:</p> <table border="0" data-bbox="730 327 1495 381"> <tr> <td>Item</td> <td>Program Title</td> <td>Program Source (Required by)</td> <td>FSAR Section</td> <td>Implementation Milestone</td> <td>Requirement</td> </tr> </table> <p>21. Cyber Security Program 10 CFR 73.54(b) 13.6 Prior to receipt of 10 CFR 73.55(a)(4)</p> <p>fuel onsite 10 CFR 73.55(b)(8); (protected area) 10 CFR 73.55(c)(6)</p> | Item | Program Title | Program Source (Required by) | FSAR Section | Implementation Milestone | Requirement | |
| Item | Program Title | Program Source (Required by) | FSAR Section | Implementation Milestone | Requirement | | | | | | | |
| 8396 | STD,VCS | PT02 | FSAR13 | 13.04.T / T13.4-201 Item 22 | <p>COLA Part 2, FSAR Chapter 13, Section 13.4, Table 13.4-201, will be revised by adding a new Item # (where # is the next appropriate number designation) with a left margin annotation (LMA) of STD COL 13.4-1, as follows:</p> <p>Program Title: SNM Material Control and Accounting Program</p> <p>Program Source (Required by): 10 CFR 74, Subpart B (§§ 74.11 – 74.19, excl. § 74.17)</p> <p>FSAR Section: 13.5.2.2.9</p> <p>Implementation Milestone: Prior to receipt of special nuclear material</p> <p>Implementation Requirement: License Condition</p> | VEGP-RAI-LTR-064 response to RAI 01.05-003 item 2 SNC Ltr ND-10-2257 (SCE&G Endorsement NND-10-0448) | | | | | | |
| 8397 | STD,VCS | PT02 | FSAR13 | 13.05 | <p>COLA Part 2, FSAR Chapter 13, Section 13.5, will be revised to add a new subsection 13.5.2.2.9 with an LMA of STD COL 13.5-1, as follows:</p> <p>13.5.2.2.9 Special Nuclear Material (SNM) Material Control and Accounting Procedures</p> <p>A material control and accounting system consisting of special nuclear material accounting procedures is utilized to delineate the requirements, responsibilities, and methods of special nuclear material control from the time special nuclear material is received until it is shipped from the plant. These procedures provide detailed steps for SNM shipping and receiving, inventory, accounting, and preparing records and reports. The Special Nuclear Material (SNM) Material Control and Accounting (MC&A) Program description is submitted to the Nuclear Regulatory Commission as a separate licensing basis document.</p> | VEGP-RAI-LTR-064 response to RAI 01.05-003 item 3 SNC Ltr ND-10-2257 (SCE&G Endorsement NND-10-0448) | | | | | | |
| 8689 | VCS | PT02 | FSAR13 | 13.05 | Remove the horizontal separator line and LMA STD COL 13.5-1 from QB Item 8397. | Separator line and LMA are not needed because previous section is a part of LMA STD COL 13.5-1 | | | | | | |
| 8291 | STD,VCS | PT02 | FSAR13 | APP 13AA / 13AA.1.1.1.1.8 | <p>COLA Part 2, FSAR Chapter 13, Appendix 13AA, Subsection 13AA.1.1.1.1.8, will be revised to add a new last paragraph to read:</p> <p>Periodic assessment involving both the construction and operations organizations continues to identify SSCs that could reasonably be expected to be impacted by scheduled construction activities. Appropriate administrative and managerial controls are then established as necessary. Specific hazards, impacted SSCs, and managerial and administrative controls are reviewed on a recurring basis and, if necessary, controls are revised/developed and implemented and maintained current as work progresses on site. For example, prior to construction activities that involve the use of large construction equipment such as cranes, managerial and administrative controls are in place to prevent adverse impacts on any operating unit(s) overhead power lines, switchyard, security boundary, etc., by providing the necessary restrictions on the use of large construction equipment.</p> | VEGP-RAI-LTR-063 response to RAI 01.05-002 item 3 SNC Ltr ND-10-2114 (SCE&G Endorsement Letter NND-10-0422) | | | | | | |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|----------------------|----------|---------------|-------------|----------------------|---|---|
| PT02 - FSAR14 | | | | | 4 COLA Changes | |
| 8700 | VCS | PT02 | FSAR14 | 14.02.01 | Add a horizontal spacer bar above the section heading for Subsection 14.2.1 | Consistency with placement of spacer bars above and below DCD section titles. |
| 8227 | STD,VCS | PT02 | FSAR14 | 14.02.02.02 | COLA Part 2, FSAR Chapter 14, Subsection 14.2.2.2, PT&O Organization Personnel Qualifications and Training, will be revised to add the following new second paragraph: Acceptable qualifications of non-supervisory test engineers follow the guidance provided in Regulatory Guide 1.28 as discussed in Appendix 1AA, i.e., ASME NQA-1-1994, Appendix 2A-1, Nonmandatory Guidance on the Qualification of Inspection and Test Personnel. | VEGP-VOL-CH14 Qualification Req response item 2 SNC Ltr ND-10-2204 (SCE&G Endorsement Letter NND-10-0428) |
| 8703 | VCS | PT02 | FSAR14 | 14.03.02.03 / 14.3-1 | Revise Subsection 14.3.2.3 to remove the horizontal spacer bar above the ITAAC table description. | Consistency with R-COLA for formatting of standard content material |
| 8105 | STD,VCS | PT02 | FSAR14 | 14A | COLA Part 2, FSAR Chapter 14, new Appendix 14A, will be added (to incorporate new DCD Appendix) to read: APPENDIX 14A DESIGN ACCEPTANCE CRITERIA/ITAAC CLOSURE PROCESS This section of the referenced DCD is incorporated by reference with no departures or supplements. | WEC DCD Rev 18 per WEC response to OI-SRP3.12-EMB-4 R1 via DCP/NRC2845 as revised by response to RAI-SRP3.12-EMB-04 R2 via DCP/NRC3020 with errata for numbering. |
| PT02 - FSAR15 | | | | | 2 COLA Changes | |
| 8233 | STD,VCS | PT02 | FSAR15 | 15.00.03.02 | COLA Part 2 (Rev. 3), FSAR Chapter 15, Section 15.0.3.2, will be revised from: The actual selected plant operating instrumentation has documented instrumentation uncertainties to calculate a primary power calorimetric uncertainty that confirms the uncertainty assumed for the initial reactor power in the safety analysis bounds the calculated calorimetric values. To read: The plant operating instrumentation selected for feedwater flow measurement is a Caldon [Cameron] LEFM CheckPlus™ System (Reference 201). This selected plant operating instrumentation has documented instrumentation uncertainties to calculate a power calorimetric uncertainty that confirms the 1% uncertainty assumed for the initial reactor power in the safety analysis bounds the calculated calorimetric power measurement uncertainty values. This calculated calorimetric is done in accordance with a previously accepted Westinghouse methodology (Reference 202). Administrative controls implement maintenance and contingency activities related to the power calorimetric instrumentation. | COL-SER-OI-CH15 S3 response to SER-OI-15.00-001 item 1 SNC Ltr ND-10-2091 (SCE&G Endorsement Letter NND-10-0407) |
| 8234 | STD,VCS | PT02 | FSAR15 | 15.00.16 | COLA Part 2 (Rev. 3), FSAR Chapter 15, Section 15.0, will be revised to add the following new subsection: 15.0.16 References Add the following text to the end of DCD Subsection 15.0.16. 201. Final Safety Evaluation for Cameron Measurement Systems Engineering Report ER-157P, Revision 8, "Caldon Ultrasonics Engineering Report ER-157P, 'Supplement to Topical Report ER-80P: Basis for a Power Uprate with the LEFM Check or Checkplus™ System,'" (TAC No. ME1321). August 16, 2010. ADAMS Accession No. ML102160694. 202. Final Safety Evaluation for Beaver Valley Power Station, Unit Nos. 1 and 2 (BVPS-1 and 2)- | COL-SER-OI-CH15 S3 response to SER-OI-15.00-001 item 2 SNC Ltr ND-10-2091 (SCE&G Endorsement Letter NND-10-0407) |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|----------------------|----------|---------------|-------------|------------------|--|--|
| | | | | | Issuance of Amendment re: 1 .4-Percent Power Uprate and Revised BVPS-2 Heatup and Cooldown Curves. September 24, 2001, ADAMS Accession No. ML012490569. | |
| PT02 - FSAR17 | | | | | 3 COLA Changes | |
| 8942 | STD,VCS | PT02 | FSAR17 | 17.05 | COLA Part 2, FSAR Chapter 17, Section 17.5, will be revised to remove all LMAs except for "STD COL 17.5-4" from the sentence "Table 13.4-201 provides milestones for operational quality assurance program implementation." | Consistency of Standard Content LMAs |
| 7970 | STD,VCS | PT02 | FSAR17 | 17.06 | COLA Part 2, FSAR Chapter 17, Section 17.6, will be revised to add a new LMA of STD COL 3.8-5 to the first paragraph. | VEGP-VOL-Ch03 response to STD COL 03.08-005 item 9 SNC Ltr ND-10-1594 (SCE&G Endorsement Letter NND-10-0340) |
| 8710 | VCS | PT02 | FSAR17 | 17.06./ 17.6-1 | Revise Subsection 17.6 to add a horizontal separator bar after the paragraphs covered by LMA STD SUP 17.6-1 and before the paragraph covered by LMA STD SUP 17.6-2. | Consistency with placement of separator bars between different LMA Sections. |
| PT02 - FSAR18 | | | | | 2 COLA Changes | |
| 7990 | VCS | PT02 | FSAR18 | 18.02.01.03 | COLA Part 2, FSAR Subsection 18.2.1.3, will be revised to read as follows: Add the following information at the end of DCD Subsection 18.2.1.3: The EOF and TSC communications strategies, as well as the EOF and TSC Human Factors attributes, are described in the Emergency Plan. | Voluntary Submittal per SCE&G Letter NND-10-0336 to incorporate WEC DCD Revision 18 (DCP_NRC_002577). |
| 8713 | VCS | PT02 | FSAR18 | 18.08.03.05 | Revise Subsection 18.8.3.5 to remove Tier2* designation from TSC description. | DCD Rev 18 conforming change and consistency with COLA Part 7 departure for TSC location. |
| PT02 - FSAR19 | | | | | 6 COLA Changes | |
| 8463 | VCS | PT02 | FSAR19 | 19.55 | COLA Part 2, FSAR Chapter 19, Subsection 19.55, Seismic Margin Analysis, will be revised from: 19.55 SEISMIC MARGIN ANALYSIS This section of the referenced DCD is incorporated by reference with no departures or supplements. To read: 19.55 SEISMIC MARGIN ANALYSIS This section of the referenced DCD is incorporated by reference with the following departures and/or supplements. | DCD Rev 18 and SCE&G Letter NND-10-0362 |
| 8464 | VCS | PT02 | FSAR19 | 19.55.6.3 | COLA Part 2, FSAR Chapter 19, Subsection 19.55.6.3 will be added with LMA VCS COL 19.59.10-6 19.55.6.3 Site Specific Seismic Margin Analysis The VCSNS site seismic demand based on the site-specific Ground Motion Response Spectra (GMRS) is enveloped by a seismic demand which combines both the Certified Seismic Design Response Spectra (CSDRS) and Hard Rock High Frequency (HRHF) design response spectra as defined by the Tier 1 criteria for SSE. Therefore, it can be concluded that the Seismic Margin Assessment analysis | DCD Rev 18 and SCE&G Letter NND-10-0362 |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|-----------------------|----------|---------------|-------------|----------------------|--|---|
| | | | | | documented in FSAR Section 19.55 is applicable to the VCSNS Units 2 and 3 site. The VCSNS Nuclear Island (NI) is founded on hard (sound) rock which eliminates any potential for site specific effects such as seismically induced liquefaction settlements, slope stability, foundation failure or relative displacements which would lower the HCLPF values calculated for the certified design. For non-safety related structures and foundations adjacent to the NI, these site specific effects are evaluated in FSAR Section 2.5.4 and shown to have no effect on the NI; therefore, having no potential to lower the HCLPF values calculated for the certified design. | |
| 8036 | VCS | PT02 | FSAR19 | 19.58.T/ T 19.58-201 | Revise FSAR Table 19.58-201 as shown in response to RAIs 19-78 and 19-81 in SCE&G Letter NND-10-0078. | RAIs 19-78 and 19-81 (Letter 071 Response per NND-10-0078) |
| 8106 | STD,VCS | PT02 | FSAR19 | 19.59.10.05 | COLA Part 2, FSAR Chapter 19, Subsection 19.59.10.5, Combined License Information, will be revised to add the following as the last paragraph with LMAs STD COL 19.59.10-6 and VCS COL 19.59.10-6: As discussed in Subsection 19.55.6.3, it has been confirmed that the Seismic Margin Analysis (SMA) documented in DCD Section 19.55 is applicable to the site. The site-specific effects (i.e., soil-related failure modes, etc.) have been evaluated and it was concluded that the plant-specific plant-level HCLPF value is equal to or greater than 1.67 times the site-specific GMRS peak ground acceleration. | VEGP-VOL-CH19 PRA item 1 SNC Ltr ND-10-1811 (SCE&G Endorsement Letter NND-10-0362) |
| 8453 | STD,VCS | PT02 | FSAR19 | 19.59.10.05 | COLA Part 2, FSAR Chapter 19, Subsection 19.59.10.5, first paragraph, is revised to add the LMA STD COL 19.59.10-6 along with the existing LMA of STD COL 19.59.10-1. | Consistency with changes made in VEGP-VOL-CH19 PRA item 1 SNC Ltr ND-10-1811 |
| 8465 | VCS | PT02 | FSAR19 | 19.59.10.5 | COLA Part 2, FSAR Chapter 19, Subsection 19.59.10.5, Combined License Information 19.59.10-1, Item 1 will be revised from: 1. Specific minimum seismic requirements consistent with those used to define the Table 19.55-1 HCLPF values. This includes the known frequency range used to define the HCLPF by comparing the required response spectrum (RRS) and test response spectrum (TRS). The range of frequency response that is required for the equipment with its structural support is defined. To Read: 1. Specific minimum seismic requirements consistent with those used to define the AP1000 DCD Table 19.55-1 HCLPF values. This includes the known frequency range used to define the HCLPF by comparing the required response spectrum (RRS) and test response spectrum (TRS). The test response spectra are chosen so as to demonstrate that no more than one percent rate of failure is expected when the equipment is subjected to the applicable seismic margin ground motion for the equipment identified to be applicable in the seismic margin insights of the site-specific PRA. The range of frequency response that is required for the equipment with its structural support is defined. | DCD Rev 18 and SCE&G Letter NND-10-0362 |
| PT04 - (empty) | | | | | | 2 COLA Changes |
| 7979 | STD,VCS | PT04 | | | See attachments to Westinghouse letter DCP/NRC2864 (dated May 6,2010) -the same changes identified in the Westinghouse letter for the DCD Generic Technical Specifications and Bases will be directly incorporated into the COL plant-specific Technical Specifications and Bases (with the exception that the bracketed Reviewer's Notes will be removed). | COL-SER-OI-CH16 S2 response to OI 16.01-001 SNC Ltr ND-10-0996 (SCE&G Endorsement Letter NND-10-0296) |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|-----------------------|----------|---------------|-------------|----------------------|--|--|
| 8382 | STD,VCS | PT04 | | TS | COLA Part 4, Technical Specifications, will be revised to incorporate the AP1000 GTS changes identified by WEC in numerous responses to AP1000 DCD RAIs and SER Open Items, and other sources as incorporated into DCD Revision 18. | DCD Rev 18 |
| PT05 - (empty) | | | | | | 3 COLA Changes |
| 9074 | VCS | PT05 | | Annex 1, Section 4.2 | VCSNS Part 5, Emergency Plan, Annex 1, Section 4.2 will be revised to add new paragraph as follows: G. Emergency Response Data System (ERDS) The Emergency Response Data System (ERDS) is supported via a data link to the NRC. In accordance with 10 CFR 50, Appendix E, Section VI, the appropriate variables listed in the FSAR including plant equipment status and parameter information for reactor core and coolant system conditions, reactor containment conditions, radioactivity release conditions, and plant meteorological conditions will be transmitted as required. | Consistency with Annex 2 and 3 modifications made in SCE&G Letter NND-10-0312 |
| 7987 | VCS | PT05 | | Annex 2, Section 4.2 | VCSNS Part 5, Emergency Plan, Annex 2, Section 4.2 will be revised to add new paragraph as follows: G. Emergency Response Data System (ERDS) The Emergency Response Data System (ERDS) is supported via a data link to the NRC. In accordance with 10 CFR 50, Appendix E, Section VI, the appropriate variables listed in DCD Table 7.5-1 and FSAR Table 7.5-201 including plant equipment status and parameter information for reactor core and coolant system conditions, reactor containment conditions, radioactivity release conditions, and plant meteorological conditions will be transmitted as required. | SCE&G Letter NND-10-0312 |
| 7988 | VCS | PT05 | | Annex 3, Section 4.2 | VCSNS Part 5, Emergency Plan, Annex 3, Section 4.2 will be revised to add new paragraph as follows: G. Emergency Response Data System (ERDS) The Emergency Response Data System (ERDS) is supported via a data link to the NRC. In accordance with 10 CFR 50, Appendix E, Section VI, the appropriate variables listed in DCD Table 7.5-1 and FSAR Table 7.5-201 including plant equipment status and parameter information for reactor core and coolant system conditions, reactor containment conditions, radioactivity release conditions, and plant meteorological conditions will be transmitted as required. | SCE&G Letter NND-10-0312 |
| PT07 - (empty) | | | | | | 5 COLA Changes |
| 8399 | STD,VCS | PT07 | | B / EXM 3 | COLA Part 7, Departures & Exemptions, Part B, will be revised to add the following discussion and justification for Exemption 4: 4) Special Nuclear Material (SNM) Material Control and Accounting (MC&A) Program Description [Part 70, Subpart D and Part 74, Subparts C, D, and E] Applicable Regulation(s): 10 CFR §§ 70.22(b), 70.32(c), 74.31, 74.41, and 74.51 Specific wording from which exemption is requested: 10 CFR 70.22(b), Contents of applications: (b) Each application for a license to possess special nuclear material, to possess | VEGP-RAI-LTR-064 response to RAI 01.05-003 item 5 SNC Ltr ND-10-2257 (SCE&G Endorsement NND-10-0448) |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|---------------------|-------------|---------------------|----------------|---------------------|--|------------------|
| | | | | | <p>equipment capable of enriching uranium, to operate an uranium enrichment facility, to possess and use at any one time and location special nuclear material in a quantity exceeding one effective kilogram, except for applications for use as sealed sources and for those uses involved in the operation of a nuclear reactor licensed pursuant to part 50 of this chapter and those involved in a waste disposal operation, must contain a full description of the applicant's program for control and accounting of such special nuclear material or enrichment equipment that will be in the applicant's possession under license to show how compliance with the requirements of §§ 74.31, 74.33, 74.41, or 74.51 of this chapter, as applicable, will be accomplished.</p> <p>10 CFR 70.32, Conditions of licenses:</p> <p>(c) (1) Each license authorizing the possession and use at any one time and location of uranium source material at an uranium enrichment facility or special nuclear material in a quantity exceeding one effective kilogram, except for use as sealed sources and those uses involved in the operation of a nuclear reactor licensed pursuant to part 50 of this chapter and those involved in a waste disposal operation, shall contain and be subject to a condition requiring the licensee to maintain and follow:</p> <p>(i) The program for control and accounting of uranium source material at an uranium enrichment facility and special nuclear material at all applicable facilities as implemented pursuant to § 70.22(b), or §§ 74.31(b), 74.33(b), 74.41(b), or 74.51(c) of this chapter, as appropriate;</p> <p>(ii) The measurement control program for uranium source material at an uranium enrichment facility and for special nuclear material at all applicable facilities as implemented pursuant to §§ 74.31(b), 74.33(b), 74.45(c), or 74.59(e) of this chapter, as appropriate; and</p> <p>(iii) Other material control procedures as the Commission determines to be essential for the safeguarding of uranium source material at an uranium enrichment facility or of special nuclear material and providing that the licensee shall make no change that would decrease the effectiveness of the material control and accounting program implemented pursuant to § 70.22(b), or §§ 74.31(b), 74.33(b), 74.41(b), or 74.51(c) of this chapter, and the measurement control program implemented pursuant to §§ 74.31(b), 74.33(b), 74.41(b), or 74.59(e) of this chapter without the prior approval of the Commission. A licensee desiring to make changes that would decrease the effectiveness of its material control and accounting program or its measurement control program shall submit an application for amendment to its license pursuant to § 70.34.</p> <p>10 CFR 74.31, Nuclear material control and accounting for special nuclear material of low strategic significance:</p> <p>(a) General performance objectives. Each licensee who is authorized to possess and use more than one effective kilogram of special nuclear material of low strategic significance, excluding sealed sources, at any site or contiguous sites subject to control by the licensee, other than a production or utilization facility licensed pursuant to part 50 or 70 of this chapter, or operations involved in waste disposal, shall implement and maintain a Commission approved material control and accounting system that will achieve the following objectives:</p> <p>10 CFR 74.41, Nuclear material control and accounting for special nuclear material of moderate strategic significance:</p> <p>(a) General performance objectives. Each licensee who is authorized to possess special nuclear material (SNM) of moderate strategic significance or SNM in a quantity exceeding one effective kilogram of</p> | |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|---------------------|-------------|---------------------|----------------|---------------------|--|------------------|
| | | | | | <p>strategic special nuclear material in irradiated fuel reprocessing operations other than as sealed sources and to use this material at any site other than a nuclear reactor licensed pursuant to part 50 of this chapter; or as reactor irradiated fuels involved in research, development, and evaluation programs in facilities other than irradiated fuel reprocessing plants; or an operation involved with waste disposal, shall establish, implement, and maintain a Commission-approved material control and accounting (MC&A) system that will achieve the following performance objectives:</p> <p>10 CFR 74.51, Nuclear material control and accounting for strategic special nuclear material:</p> <p>(a) General performance objectives. Each licensee who is authorized to possess five or more formula kilograms of strategic special nuclear material (SSNM) and to use such material at any site, other than a nuclear reactor licensed pursuant to part 50 of this chapter, an irradiated fuel reprocessing plant, an operation involved with waste disposal, or an independent spent fuel storage facility licensed pursuant to part 72 of this chapter shall establish, implement, and maintain a Commission-approved material control and accounting (MC&A) system that will achieve the following objectives:</p> <p>Discussion:</p> <p>South Carolina Electric and Gas (SCE&G) requests an exemption from the requirements of 10 CFR § 70.22(b) and, in turn, §§ 70.32(c), 74.31, 74.41, and 74.51. Section 70.22(b) requires an application for a license for special nuclear material to contain a full description of the applicant's program for material control and accounting (MC&A) of special nuclear material under §§ 74.31, 74.33, 74.41, and 74.51. Section 70.32(c) requires a license authorizing the use of special nuclear material to contain and be subject to a condition requiring the licensee to maintain and follow a special nuclear material control and accounting program, measurement control program, and other material control procedures, including the corresponding records management requirements. However, §§ 70.22(b), 70.32(c), 74.31, 74.41, and 74.51 contain exceptions for nuclear reactors licensed under 10 CFR Part 50. The regulations applicable to the MC&A of special nuclear material for nuclear reactors licensed under 10 CFR Part 50 are provided in 10 CFR Part 74, Subpart B, §§ 74.11 through 74.19, excluding § 74.17. The purpose of this exemption request is to seek a similar exception for this combined license (COL) under 10 CFR Part 52, such that the same regulations will be applied to the special nuclear material MC&A program as nuclear reactors licensed under 10 CFR Part 50.</p> <p>Nuclear reactors licensed under Part 50 are explicitly excepted from the requirements of §§ 70.22(b), 70.32(c), 74.31, 74.41, and 74.51. There is no technical or regulatory reason to treat nuclear reactors licensed under Part 52 differently than reactors licensed under Part 50 with respect to the MC&A provisions in 10 CFR Part 74. As indicated in the Statement of Considerations for 10 CFR § 52.0(b) (72 Fed. Reg. 49352, 49372, 49436 (Aug. 28, 2007)), applicants and licensees under Part 52 are subject to all of the applicable requirements in 10 CFR Chapter I, whether or not those provisions explicitly mention a COL under Part 52.</p> <p>This regulation clearly indicates that plants licensed under Part 52 are to be treated no differently than plants licensed under Part 50 with respect to the substantive provisions in 10 CFR Chapter I (which includes Parts 70 and 74). In particular, the exception for nuclear reactors licensed under Part 50, as contained in §§ 70.22(b), 70.32(c), 74.31, 74.41, or 74.51, should also be applied to reactors licensed under Part 52.</p> <p>An exemption from the requirements of §§ 70.22(b), 70.32(c), 74.31, 74.41, and 74.51 would not mean that a MC&A program would be unnecessary or that the COL application would be silent regarding MC&A. To the contrary, the MC&A requirements in Subpart B to Part 74 would still be applicable to the COL just as they are to licenses issued under Part 50. Additionally, the COL application will describe the MC&A program for satisfying Subpart B to Part 74.</p> <p>This exemption request is evaluated under 10 CFR § 52.7, which incorporates the requirements of § 50.12. That section allows the Commission to grant an exemption if 1) the exemption</p> | |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change | | |
|------------------|-------------|---------------|-------------|------------------|--|---|-------------|--|
| | | | | | <p>is authorized by law, 2) will not present an undue risk to the public health and safety, 3) is consistent with the common defense and security, and 4) special circumstances are present as specified in 10 CFR § 50.12(a)(2). The criteria in § 50.12 encompass the criteria for an exemption in 10 CFR §§ 70.17(a) and 74.7, the specific exemption requirements for Parts 70 and 74, respectively. Therefore, by demonstrating that the exemption criteria in § 50.12 are satisfied, this request also demonstrates that the exemption criteria in §§ 52.7, 70.17(a) and 74.7 are satisfied.</p> <p>While not containing an explicit exception for Part 50 reactors, § 74.33 applies only to uranium enrichment facilities and thus is not directly implicated in this exemption request.</p> <p>Evaluation Against Exemption Criteria</p> <p>1) This exemption is not inconsistent with the Atomic Energy Act or any other statute and is therefore authorized by law.</p> <p>2) An exemption from the requirements of 10 CFR §§ 70.22(b), 70.32(c), 74.31, 74.41, and 74.51 would not present an undue risk to public health and safety. The exemption would treat the COL applicant similarly to Part 50 license applicants, who are excepted from the regulations in question. Furthermore, the COL application will contain a description of the applicant's MC&A program under Subpart B to Part 74. Therefore, the exemption from 10 CFR §§ 70.22(b), 70.32(c), 74.31, 74.41, and 74.51 would not present an undue risk to public health and safety.</p> <p>3) An exemption from the requirements of 10 CFR §§ 70.22(b), 70.32(c), 74.31, 74.41, and 74.51 would not be inconsistent with the common defense and security. The exemption would treat the COL applicant similarly to Part 50 license applicants, who are excepted from the regulations in question. Furthermore, the COL application will contain a description of the applicant's MC&A program under Subpart B to Part 74.</p> <p>Therefore, the exemption from §§ 70.22(b), 70.32(c), 74.31, 74.41, and 74.51 is consistent with the common defense and security.</p> <p>4) The exemption request involves special circumstances under 10 CFR § 50.12(a)(2)(ii).</p> <p>That subsection defines special circumstances as when "[a]pplication of the regulation in the particular circumstances would not serve the underlying purpose of the rule or is not necessary to achieve the underlying purpose of the rule." Since the Commission determined that the requirements in 10 CFR §§ 70.22(b), 70.32(c), 74.31, 74.41, and 74.51 are unnecessary for Part 50 applicants, those requirements are also unnecessary for Part 52 applicants.</p> <p>As demonstrated above, the exemption complies with the requirements of 10 CFR §§ 50.12, 52.7, 70.17, and 74.7. For these reasons, approval of the requested exemption is requested from the regulations of 10 CFR §§ 70.22(b), 70.32(c), 74.31, 74.41, and 74.51, as described herein.</p> | | | |
| 8398 | STD,VCS | PT07 | | B / EXM 4 | <p>COLA Part 7, Departures, Exemptions, and Variances, Part B, will be revised to add the following exemption request (where # is the next appropriate exemption request number):</p> <p>4. Special Nuclear Material (SNM) Material Control and Accounting Program Description</p> | <p>VEGP-RAI-LTR-064 response to RAI 01.05-003 item 4 SNC Ltr ND-10-2257 (SCE&G Endorsement NND-10-0448)</p> | | |
| 7774 | VCS | PT07 | | DEP 18.08-1 | <p>Revise Part 7, VCS DEP 18.08-01 as indicated in SCE&G Letter NND-10-0304, dated August 11, 2010.</p> | <p>DCD Revision 18, DCP_NRC_002754 and SCE&G Letter NND-10-0304</p> | | |
| 8040 | STD,VCS | PT07 | | SEC A | <p>COLA Part 7, Section A, STD and VCS Departures, will be revised to add the following departure:</p> <table border="0" data-bbox="751 1443 1262 1471"> <tr> <td style="text-align: center;">Departure Number</td> <td style="text-align: center;">Description</td> </tr> </table> | Departure Number | Description | <p>VEGP-VOL-CH08 response to STD-VOL-08.03-002 item 4 SNC Ltr ND-10-2005</p> |
| Departure Number | Description | | | | | | | |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change | | | | |
|---------------------|---|---------------------|----------------|---------------------|--|---|-------------|---------------|---|---|
| | | | | | <p>STD DEP 8.3-1 Class 1E voltage regulating transformer current limiting features</p> | <p>(SCE&G Endorsement Letter NND-10-0381)</p> | | | | |
| 8041 | STD,VCS | PT07 | | SEC A | <p>COLA Part 7, Section A, will be revised to add the following departure:</p> <table border="0" data-bbox="772 384 1470 475"> <tr> <td style="text-align: center;">Departure Number</td> <td style="text-align: center;">Description</td> </tr> <tr> <td style="text-align: center;">STD DEP 8.3-1</td> <td style="text-align: center;">Class 1E voltage regulating transformer current limiting features</td> </tr> </table> <p>Departure Number: STD DEP 8.3-1 Affected DCD/FSAR Sections: 8.3.2.2</p> <p>Summary of Departure:</p> <p>The DCD states that the Class 1E battery chargers and Class 1E voltage regulating transformers are designed to limit the input (ac) current to an acceptable value under faulted conditions on the output side. However, the AP1000 voltage regulating transformers do not have active components to limit current.</p> <p>Scope/Extent of Departure:</p> <p>This departure is identified in FSAR Section 8.3.2.2.</p> <p>Departure Justification:</p> <p>DCD section 8.3.2.2 states that the Class 1E voltage regulating transformers have built-in circuit breakers at the input and output sides for protection and isolation. The circuit breakers are coordinated and periodically tested to verify their designed coordination and isolation function. They are qualified as isolation devices between Class 1E and non-Class 1E circuits in accordance with IEEE 384 and Regulatory Guide 1.75. Since the isolation and protection function is provided by the breakers, there is no need for the voltage regulating transformers to have current limiting capability. This departure does not adversely affect any safety-related system, nor does it conflict with applicable regulatory guidance.</p> <p>Departure Evaluation:</p> <p>This Tier 2 departure is associated with isolation between Class 1E loads and the non-Class 1E ac power source. The departure results in a change to the DCD that does not impact the required design function (i.e., isolation). Accordingly, it does not:</p> <ol style="list-style-type: none"> 1. Result in more than a minimal increase in the frequency of occurrence of an accident previously evaluated in the plant-specific DCD; 2. Result in more than a minimal increase in the likelihood of occurrence of a malfunction of a structure, system, or component (SSC) important to safety and previously evaluated in the plant-specific DCD; 3. Result in more than a minimal increase in the consequences of an accident previously evaluated in the plant-specific DCD; 4. Result in more than a minimal increase in the consequences of a malfunction of an SSC important to safety previously evaluated in the plant-specific DCD; 5. Create a possibility for an accident of a different type than any evaluated previously in the plant-specific DCD; | Departure Number | Description | STD DEP 8.3-1 | Class 1E voltage regulating transformer current limiting features | <p>VEGP-VOL-CH08 response to STD-VOL-08.03-002 item 5 SNC Ltr ND-10-2005 (SCE&G Endorsement Letter NND-10-0381)</p> |
| Departure Number | Description | | | | | | | | | |
| STD DEP 8.3-1 | Class 1E voltage regulating transformer current limiting features | | | | | | | | | |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|-----------------------|----------|---------------|-------------|------------------|--|--|
| | | | | | <p>6. Create a possibility for a malfunction of an SSC important to safety with a different result than any evaluated previously in the plant-specific DCD;</p> <p>7. Result in a design basis limit for a fission product barrier as described in the plant-specific DCD being exceeded or altered; or</p> <p>8. Result in a departure from a method of evaluation described in the plant-specific DCD used in establishing the design bases or in the safety analyses.</p> <p>This Tier 2 departure does not affect resolution of an ex-vessel severe accident design feature identified in the plant-specific DCD.</p> <p>Therefore, this departure has no safety significance.</p> <p>NRC Approval Requirement:</p> <p>This departure does not require NRC approval pursuant to 10 CFR Part 52, Appendix D, Section VIII.B.5.</p> | |
| PT10 - (empty) | | | | | 22 COLA Changes | |
| 8711 | VCS | PT10 | | All LCs | Format all license condition headings such that they are all capitalized. | Consistency with formatting of license condition headings throughout Part 10. |
| 8111 | STD,VCS | PT10 | | LC 02 | <p>COLA Part 10, Proposed License Conditions, including ITAAC, proposed License Condition #2, item 14.4-3, Conduct of Test Program, will be revised from:</p> <p>14.4-3 Conduct of Test Program 14.4.3 Prior to initiating test program A site-specific startup administration manual (procedure), which contains the administration procedures and requirements that govern the activities associated with the plant initial test program, as identified in FSAR Section 14.2, is provided prior to initiating the plant initial test program.</p> <p>To read:</p> <p>14.4-3 Conduct of Test Program 14.4.3 NOTE -addressed by proposed License Conditions #3 and #6.</p> | VEGP-VOL-CH14 response to item 1 SNC Ltr ND-10-1993 (SCE&G Endorsement Letter NND-10-0395) |
| 8112 | STD,VCS | PT10 | | LC 02 | <p>COLA Part 10, Proposed License Conditions, including ITAAC, proposed License Condition #2, item 14.4-4, Review and Evaluation of Test Results, will be revised from:</p> <p>14.4-4 Review and Evaluation of Test Results 14.4.4 Prior to initial fuel load The Combined License holder is responsible for review and evaluation of individual test results as well as final review of overall test results and for review of selected milestones or hold points within the test phases. Test exceptions or results which do not meet acceptance criteria are identified to the affected and responsible design organizations, and corrective actions and retests, as required, are performed.</p> <p>To read:</p> <p>14.4-4 Review and Evaluation of Test Results 14.4.4 NOTE - addressed by proposed License Condition #9.</p> | VEGP-VOL-CH14 response to item 2 SNC Ltr ND-10-1993 (SCE&G Endorsement Letter NND-10-0395) |
| 8113 | STD,VCS | PT10 | | LC 02 | <p>COLA Part 10, Proposed License Conditions, including ITAAC, proposed License Condition #2, item 14.4-6, First-Plant-Only and Three-Plant-Only Tests, will be revised from:</p> <p>14.4-6 First-Plant-Only and Three-Plant-Only Tests 14.4.6 Prior to preoperational testing</p> | VEGP-VOL-CH14 response to item 3 SNC Ltr ND-10-1993 (SCE&G Endorsement Letter NND-10-0395) |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|---------------------|-------------|---------------------|----------------|---------------------|---|---|
| | | | | | <p>The COL holder for the first plant and the first three plants will perform the tests listed in subsection 14.2.5. For subsequent plants, either tests listed in subsection 14.2.5 shall be performed, or the COL applicant shall provide a justification that the results of the first-plant-only tests or firstthree-plant tests are applicable to the subsequent plant.</p> <p>The Combined License holder(s) for the first AP1000 plant (or first three plants) available for testing will perform the tests defined during preoperational and startup testing as identified in subsections 14.2.9 and 14.2.10. Combined License holders referencing the results of the tests will provide the report as necessary. The schedule for providing this information will be provided prior to preoperational testing.</p> <p>To read:</p> <p>14.4-6 First-Plant-Only and Three-Plant-Only Tests 14.4.6 NOTE -addressed by proposed License Conditions #7 and #9.</p> | |
| 8115 | STD,VCS | PT10 | | LC 07 | <p>COLA Part 10, Proposed License Conditions, including ITAAC, proposed License Condition #7, First-Plant-Only and First-Three-Plant-Only Testing, will be revised from:</p> <p>7. First-Plant-Only and First-Three-Plant-Only Testing Certain design features of the AP1000 plant will be subjected to special tests to establish unique phenomenological performance parameters of the AP1000 design. Because of the standardization of the AP1000 design, these special tests (designated as first-plant-only tests and first-three-plant-only tests) are not required on subsequent plants. These tests will be controlled through license conditions to ensure that relevant test results are reviewed, evaluated, and approved by the designated licensee management before proceeding with the next testing phase. Accordingly, the following license condition is proposed: First-Plant-Only and First-Three-Plant-Only Testing Following completion of the testing, the licensee completing the testing shall review and evaluate individual test results. Test exceptions or results which do not meet acceptance criteria are identified to the affected and responsible organizations, and corrective actions and tests, as required, are performed.</p> <p>Additionally, the licensee completing the testing shall also provide written notification of completion of the testing to the Director of the Office of New Reactors.</p> <ol style="list-style-type: none"> 1. For testing completed during pre-critical testing, criticality testing, and during low-power testing, these reports may be in conjunction with the test completion reports required for the power ascension test phase as identified below. 2. For tests completed during operation above 5% RTP, the reports shall be provided for each individual test within thirty (30) calendar days of the licensee confirmation of completion of the testing. <p>Subsequent plant licensees crediting completion of testing by the first-plant or by the first-three plants shall provide a report referencing the written notification of completion submitted by the plant(s) completing the testing to the Director of the Office of New Reactors.</p> <p>To read:</p> <p>7. First-Plant-Only and First-Three-Plant-Only Testing Certain design features of the AP1 000 plant will be subjected to special tests to establish unique phenomenological performance parameters of the AP1000 design. Because of the standardization of the AP1000 design, these special tests (designated as first-plant-only tests and first-three-plant-only tests) are not required on subsequent plants. Once these tests are completed by the first plant (or first three plants) and appropriate documentation identified, the subsequent plants need only reference the applicable documentation to show that the first plant (or first three plants) completed the required testing. Accordingly, the following license condition is proposed: First-Plant-Only and First-Three-Plant-Only Testing.</p> <p>A licensee shall provide written identification of the applicable references for documentation for the</p> | <p>VEGP-VOL-CH14 response to item 5 SNC Ltr ND-10-1993 (SCE&G Endorsement Letter NND-10-0395)</p> |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|---------------------|-------------|---------------------|----------------|---------------------|--|--|
| | | | | | <p>completion of the testing to the Director of the Office of New Reactors (or equivalent NRC management) within thirty (30) calendar days of the licensee confirmation of acceptable test results.</p> <p>Subsequent plant licensees crediting completion of testing by the first-plant or by the first-three plants shall provide a report referencing the applicable documentation identified by the first (or first three) plant(s) confirming the testing to the Director of the Office of New Reactors (or equivalent NRC management).</p> <p>This report shall be provided to NRC either prior to initiation of pre-operational testing, or within sixty (60) days of the identification of the documentation for the completion of the testing by the first plant (or third plant, as appropriate), whichever is later.</p> | |
| 8116 | STD,VCS | PT10 | | LC 09 | <p>COLA Part 10, Proposed License Conditions, including ITAAC, proposed License Condition #9, Power-Ascension Test Phase, will be revised to address the complete startup testing program, with additions for pre-operational testing, and for above 5% up to and including 100% RTP from:</p> <p>9. Power-Ascension Test Phase</p> <p>Certain milestones within the startup testing phase of the initial test program (i.e., pre-critical testing, criticality testing, and low-power «5% RTP testing) are controlled through license conditions to ensure that relevant test results are reviewed, evaluated, and approved by the designated licensee management before proceeding with the power ascension test phase.</p> <p>Accordingly, the following license conditions are proposed:</p> <p>Pre-critical and Criticality Testing</p> <p>1. Following completion of pre-critical and criticality testing, the licensee shall review and evaluate individual test results. Test exceptions or results which do not meet acceptance criteria are identified to the affected and responsible organizations, and corrective actions and retests, as required, are performed.</p> <p>2. The licensee shall provide written notification to the Director of the Office of New Reactors within fourteen (14) calendar days of completion of the pre-critical and criticality testing.</p> <p>Low-Power «5% RTP Testing</p> <p>1. Following completion of low-power «5% RTP testing, the licensee shall review and evaluate individual test results. Test exceptions or results which do not meet acceptance criteria are identified to the affected and responsible organizations, and corrective actions and retests, as required, are performed.</p> <p>2. The licensee shall provide written notification to the Director of the Office of New Reactors within fourteen (14) calendar days of completion of the low power testing.</p> <p>To read:</p> <p>9. Startup Program Test Results</p> <p>Certain milestones within the startup testing phase of the initial test program (i.e., pre-critical testing, criticality testing, and low-power «5% RTP) testing) are controlled through license conditions to ensure that relevant test results are reviewed, evaluated, and approved by the deSignated licensee management before proceeding with the power ascension test phase.</p> <p>Accordingly, the following license conditions are proposed: Pre-operational Testing Following completion of pre-operational testing, the licensee shall review and evaluate individual test results. Test exceptions or results which do not meet acceptance criteria are identified to the affected and responsible organizations, and corrective actions and retests, as required, are performed.</p> | VEGP-VOL-CH14 response to item 6 SNC Ltr ND-10-1993 (SCE&G Endorsement Letter NND-10-0395) |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|---------------|----------|---------------|-------------|------------------|---|---|
| | | | | | <p>Pre-critical and Criticality Testing</p> <p>1. Following completion of pre-critical and criticality testing, the licensee shall review and evaluate individual test results. Test exceptions or results which do not meet acceptance criteria are identified to the affected and responsible organizations, and corrective actions and retests, as required, are performed.</p> <p>2. The licensee shall provide written notification to the Director of the Office of New Reactors (or equivalent NRC management) within fourteen (14) calendar days of completion of the pre-critical and criticality testing.</p> <p>Low-Power «5% RTP Testing</p> <p>1. Following completion of low-power testing «5% RTP, the licensee shall review and evaluate individual test results. Test exceptions or results which do not meet acceptance criteria are identified to the affected and responsible organizations, and corrective actions and retests, as required, are performed.</p> <p>2. The licensee shall provide written notification to the Director of the Office of New Reactors (or equivalent NRC management) within fourteen (14) calendar days of completion of the low-power testing.</p> <p>At-Power (5%-100% RTP) Testing</p> <p>1. Following completion of at-power testing (at or above 5% RTP up to and including testing at 100% RTP), the licensee shall review and evaluate individual test results. Test exceptions or results which do not meet acceptance criteria are identified to the affected and responsible organizations, and corrective actions and retests, as required, are performed.</p> <p>2. The licensee shall provide written notification to the Director of the Office of New Reactors (or equivalent NRC management) within fourteen (14) calendar days of completion of the at-power testing.</p> | |
| 8276 | STD,VCS | PT10 | | LC#01 | <p>COLA Part 10, Proposed License Condition #1, ITAAC, introductory statements will be revised from:</p> <p>There are several ITAAC identified in the COLA. Once incorporated into the COL, the regulations identify the requirements that must be met.</p> <p>To read:</p> <p>There are several ITAAC identified in the COLA. Once incorporated into the COL, the regulations identify the requirements that must be met. The incorporation below includes the sensitive unclassified non-safeguards information (including proprietary information), and safeguards information referenced in the AP1000 DCD. Such DCD information is included in this combined license application in the same manner as it is included in the AP1000 DCD, i.e., references in the DCD are included as references in the FSAR, and material incorporated by reference into the DCD is incorporated by reference into the FSAR. Appropriate agreements are in place to provide access to the withheld sensitive unclassified non-safeguards information (including proprietary information), and safeguards information referenced in the AP1000 DCD.</p> | <p>VEGP-VOL-CH01 IBR of PI & SGI response item 2 SNC Ltr ND-10-2207 (SCE&G Endorsement Letter NND-10-0429)</p> |
| 8237 | STD,VCS | PT10 | | LC#02, 15.0-1 | <p>COLA Part 10 (Rev. 3), Proposed License Conditions, LC#2, COL Holder Items, COL Item No. 15.0-1, will be revised from:</p> <p>15.0-1 Documentation of Plant Calorimetric Uncertainty Methodology 15.0.15.1 Prior to initial fuel load</p> <p>Confirm the plant operating instrumentation installed for feedwater flow measurement is a Caldon (Cameron) LEFM CheckPlus™ System.</p> | <p>COL-SER-OI-CH15 S3 response to SER-OI-15.00-001 item 3 SNC Ltr ND-10-2091 (SCE&G Endorsement Letter NND-10-0407). Note: this response supersedes QB item 8117.</p> |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|---------------|----------|---------------|-------------|-------------------|---|---|
| | | | | | <p>To read:</p> <p>15.0-1 Documentation of Plant Calorimetric Uncertainty Methodology 15.0.15.1</p> <p>Note -addressed by proposed ITAAC Table 2.5.4-2, item 4.</p> | |
| 8110 | STD,VCS | PT10 | | LC#02, 19.59.10-5 | <p>COLA Part 10, License Conditions and ITAAC, Section 2, COL Item No. 19.59.10-1, Item 1 will be revised from:</p> <p>1. Specific minimum seismic requirements consistent with those used to define the Table 19.55-1 HCLPF values. This includes the known frequency range used to define the HCLPF by comparing the required response spectrum (RRS) and test response spectrum (TRS). The range of frequency response that is required for the equipment with its structural support is defined.</p> <p>To read:</p> <p>1. Specific minimum seismic requirements consistent with those used to define the Table 19.55-1 HCLPF values. This includes the known frequency range used to define the HCLPF by comparing the required response spectrum (RRS) and test response spectrum (TRS). The test response spectra are chosen so as to demonstrate that no more than one percent rate of failure is expected when the equipment is subjected to the applicable seismic margin ground motion for the equipment identified to be applicable in the seismic margin insights of the site-specific PRA. The range of frequency response that is required for the equipment with its structural support is defined.</p> | VEGP-VOL-CH19 PRA item 6 SNC Ltr ND-10-1811 (SCE&G Endorsement Letter NND-0362) |
| 8118 | STD,VCS | PT10 | | LC#03 | <p>COLA Part 10, proposed License Condition 3, Operational Program Implementation, item C, Receipt of Materials, will be revised from:</p> <p>C.4 -Emergency Planning (applicable portions)</p> <p>To read:</p> <p>C.4 -Deleted</p> | VEGP-RAI-LTR-062 response to RAI 01.05-001 item 3 SNC Ltr ND-10-2002 (SCE&G Endorsement Letter NND-10-0440) |
| 8400 | STD,VCS | PT10 | | LC#03 C.7 | <p>COLA Part 10, Proposed License Conditions, LC#3.C, Operational Program Implementation, Receipt of Materials, will be revised to include a new line item for implementation of an SNM material control and accounting program, as follows (where # is the next appropriate letter designation):</p> <p>C.#. SNM Material Control and Accounting Program</p> | VEGP-RAI-LTR-064 response to RAI 01.05-003 item 6 SNC Ltr ND-10-2257 (SCE&G Endorsement NND-10-0448) |
| 8107 | STD,VCS | PT10 | | LC#06 | <p>COLA Part 10, Proposed License Conditions, will be revised to add a new standard item to proposed license condition 6 to read (where # is the next appropriate letter):</p> <p>#. the implementation of construction and inspection procedures for concrete filled steel plate modules activities before and after concrete placement, use of construction mock-ups, and inspection of modules before and after concrete placement as discussed in DCD Subsection 3.8.4.8.</p> | VEGP-VOL-CH03 Const Procedures response to STD-COL-03.08-006 item 3 SNC Ltr ND-10-1900 (SCE&G Endorsement Letter NND-10-0379) |
| 8114 | STD,VCS | PT10 | | LC#06 | <p>COLA Part 10, Proposed License Conditions, including ITAAC, proposed License Condition #6, Operational Program Readiness, will be revised from:</p> <p>c. the approved preoperational and startup test procedures in accordance with FSAR Subsection 14.2.3.</p> <p>To read:</p> <p>c. the approved preoperational and startup test procedures (including the site-specific startup adm</p> | VEGP-VOL-CH14 response to item 4 SNC Ltr ND-10-1993 (SCE&G Endorsement Letter NND-10-0395) |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|---------------|----------|---------------|-------------|---------------------|---|--|
| | | | | | instration manual (procedure) prior to initiating the plant initial test program) in accordance with FSAR Subsection 14.2.3. | |
| 8236 | STD,VCS | PT10 | | LC#06 | COLA Part 10, Proposed License Conditions, LC#6, Operational Program Readiness, will be revised to include a new line item for availability of documentation of plant calorimetric uncertainty methodology as follows (where # is the next appropriate letter designation): #. the availability of documented instrumentation uncertainties to calculate a power calorimetric uncertainty, prior to initial fuel load. #. the availability of administrative controls to implement maintenance and contingency activities related to the power calorimetric uncertainty instrumentation, prior to initial fuel load. | COL-SER-OI-CH15 S3 response to SER-OI-15.00-001 item 4 SNC Ltr ND-10-2091 (SCE&G Endorsement Letter NND-10-0407) |
| 8117 | STD,VCS | PT10 | | LC#2 | COLA Part 10, Proposed License Conditions, LC#2, will be revised to include a new line item for COL item 15.0-1 as follows: 15.0-1 Documentation of Plant Calorimetric Uncertainty Methodology 15.0.15.1 Prior to initial fuel load Confirm the plant operating instrumentation installed for feedwater flow measurement is a Caldon [Cameron] LEFM CheckPlus™ System. Additionally, confirm that administrative controls are in place to implement maintenance and contingency activities related to the system. | COL-SER-OI-Ch15 S2 response to SER OI 15.00-001 item 3 SNC Ltr ND-10-1527 (SCE&G Endorsement Letter NND-10-0332) |
| 8238 | STD,VCS | PT10 | | LC#AppB | COLA Part 10, Appendix B, will be revised to include a new Plant-Specific ITAAC line item for COL item 15.0-1 as follows: Add the following information to the information provided in the referenced DCD Tier 1 Section 2.5.4, as a new item 4 under the Design Description section: 4. The plant operating instrumentation installed for feedwater flow measurement is one that has been specifically approved by the NRC; the power calorimetric uncertainty calculation includes uncertainties for the associated instrumentation based on an NRC approved methodology; and the calculated calorimetric values are bounded by the uncertainty value assumed for the initial reactor power in the safety analysis. | COL-SER-OI-CH15 S3 response to SER-OI-15.00-001 item 5 SNC Ltr ND-10-2091 (SCE&G Endorsement Letter NND-10-0407) |
| 8239 | STD,VCS | PT10 | | LC#AppB | COLA Part 10, Appendix B, will be revised to include a new Plant-Specific ITAAC line item for COL item 15.0-1 as follows: Add the following information to the information provided in the referenced DCD Tier 1 Section 2.5.4, as a new, final line item in Table 2.5.4-2. Refer to the final response letter ND-10-2091 for the complete change. | COL-SER-OI-CH15 S3 response to SER-OI-15.00-001 item 6 SNC Ltr ND-10-2091 (SCE&G Endorsement Letter NND-10-0407) |
| 7982 | VCS | PT10 | | Table 3.8-1, Item 1 | VCSNS Part 10, Table 3.8-1, Emergency Planning ITAAC, Acceptance Criteria 1.1 will be revised in a future COLA update as follows: 1.1 The specified parameters as listed in DCD Table 7.5-1 and FSAR Table 7.5-201 are retrievable in the Control Rooms, TSC and EOF, and the ranges of the displays encompass the values specified in the Emergency Classification and EAL Technical Basis Document. | SCE&G Letter NND-10-0312 |
| 7983 | VCS | PT10 | | Table 3.8-1, Item 5 | VCSNS Part 10, Table 3.8-1, Emergency Planning ITAAC, Acceptance Criteria 5.1.5 will be revised in a future COLA update as follows: 5.1.5 The TSC has the means to receive, store, process, and display plant and environmental information as listed in DCD Table 7.5-1 and FSAR Table 7.5-201, and to initiate emergency measures and conduct emergency assessment. | SCE&G Letter NND-10-0312 |
| 7984 | VCS | PT10 | | Table 3.8-1, | VCSNS Part 10, Table 3.8-1, Emergency Planning ITAAC, Acceptance Criteria | SCE&G Letter NND-10-0312 |

| QB Change ID# | COLA REP | COLA Part REP | Chapter REP | Section/Page REP | Complete Change Description | Basis for Change |
|-----------------------|----------|---------------|-------------|---------------------|---|---|
| | | | | Item 5 | 5.2.4 will be revised in a future COLA update as follows: 5.2.4 Radiological data identified in the EP Unit Annex, meteorological data, and plant system data pertinent to determining offsite protective measures as listed in DCD Table 7.5-1 and FSAR Table 7.5-201 are available and displayed when activated in the EOF. | |
| 7985 | VCS | PT10 | | Table 3.8-1, Item 8 | VCSNS Part 10, Table 3.8-1, Emergency Planning ITAAC, Acceptance Criteria 8.1.3 will be revised in a future COLA update as follows: 8.1.3 The exercise was completed within the specified time periods of Appendix E to 10 CFR Part 50, offsite exercise objectives were met, and there were no uncorrected offsite exercise deficiencies, or a license condition requires offsite deficiencies to be corrected prior to operation above 5% of rated power as described in 10 CFR 50.54(gg). | SCE&G Letter NND-10-0312 |
| 8279 | VCS | PT10 | | Table 3.8-1, Item 8 | COLA Part 10, Table 3.8-1, Emergency Planning ITAAC 8.1.1.D.2 will be revised to add the following item: c. Demonstrate the capability of TSC and EOF equipment and data displays to clearly identify and reflect the affected unit. | VEGP-P10-VOL-CH18 TSC ACRS item 3 SNC Ltr ND-10-2086 (SCE&G Endorsement Letter NND-10-0434) |
| Supp - (empty) | | | | | | 2 COLA Changes |
| 8401 | VCS | Supp | | Part 16 | New COLA Part 16, Special Nuclear Material (SNM) Material Control and Accounting Program Description will be provided. See Letter NND-10-0448 for program. | SCE&G Letter NND-10-0448 |
| 8454 | VCS | Supp | | Part 16 | Brackets removed from Special Nuclear Material (SNM) Material Control and Accounting (MC&A) Program Description in COLA Part 16. | Formatting Correction to SCE&G Letter NND-10-0448 |

SUMMARY

| COLA Part REP | Chapter REP | Number of COLA Changes |
|---------------|-------------|------------------------|
| PT01 | (empty) | 3 |
| PT02 | FSAR01 | 25 |
| PT02 | FSAR03 | 11 |
| PT02 | FSAR05 | 4 |
| PT02 | FSAR06 | 9 |
| PT02 | FSAR07 | 1 |
| PT02 | FSAR08 | 2 |
| PT02 | FSAR09 | 3 |
| PT02 | FSAR11 | 2 |
| PT02 | FSAR12 | 2 |
| PT02 | FSAR13 | 8 |
| PT02 | FSAR14 | 4 |

| COLA Part REP | Chapter REP | Number of COLA Changes |
|---------------------------|--------------------|-------------------------------|
| PT02 | FSAR15 | 2 |
| PT02 | FSAR17 | 3 |
| PT02 | FSAR18 | 2 |
| PT02 | FSAR19 | 6 |
| PT04 | <i>(empty)</i> | 2 |
| PT05 | <i>(empty)</i> | 3 |
| PT07 | <i>(empty)</i> | 5 |
| PT10 | <i>(empty)</i> | 22 |
| Supp | <i>(empty)</i> | 2 |
| TOTALS (21 groups) | | 121 |