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		к 1	к 2	к 3	к 4	К 5	к 6	A 1	A 2	A 3	A 4	G *	Total		A2		G*	Total
1.	1	3	3	3				3	3			3	18		3		3	6
Emergency & Abnormal Plant	2	2	1	2		N/A		2	1	N	/A	1	9	 	2	1	2	4
Evolutions	Tier Totals	5	4	5				5	4			4	27		5		5	10
	1	3	2	3	3	3	2	2	3	2	3	2	28		3		2	5
2. Plant	2	1	1	1	1	1	0	1	1	1	1	1	10	2			1	3
Systems	Tier Totals	4	3	4	4	4	2	3	4	3	4	3	38		5		3	8
3. Generic Ki	-	l Abi	lities	;	1		1	2	3		4	ŀ	10	1	2	3	4	7
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ES-401, REV 9			T10	G1 PWR EXAMINATION OUTLINE	FORM ES-4		
KA	NAME / SAFETY FUNCTION:	IF	२	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RO	SRC	C			
008AK2.02	Pressurizer Vapor Space Accident / 3	2.7	2.7		Sensors and detectors		
011EK2.02	Large Break LOCA / 3	2.6	2.7		Pumps		
022AA2.01	Loss of Rx Coolant Makeup / 2	3.2	3.8		Whether charging line leak exists		
025AG2.1.7	Loss of RHR System / 4	4,4	4.7		Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior and instrument interpretation.		
26AK3.01	Loss of Component Cooling Water / 8	3.2	3.5		The conditions that will initiate the automatic opening and closing of the SWS isolation valves to the CCWS coolers		
27AA1.05	Pressurizer Pressure Control System Malfunction / 3	3.3	3.2		Transfer of heaters to backup power supply		
29EG2.4.31	ATWS / 1	4.2 4	4.1		Knowledge of annunciators alarms, indications or response procedures		
38EA1.17	Steam Gen. Tube Rupture / 3	3.2 3	3.2		S/G sample isolation valve indicators		
54AA2.07	Loss of Main Feedwater / 4	3.4 3	3.9		Reactor trip first-out panel indicator		
56AA2.56	Loss of Off-site Power / 6	3.6 3	3.7		RCS T-ave		
57AK3.01	Loss of Vital AC Inst. Bus / 6	4.1 4	.4		Actions contained in EOP for loss of vital ac electrical		

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ES-401, REV 9		-	T1G1 PWR EXAMINATION OUTLINE	FORM ES-401-	
KA	NAME / SAFETY FUNCTION:	IR	R K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO S	SRO		
058AK1.01	Loss of DC Power / 6	2.8 3	3.1 🖌 🗌 🗌 🗌 🗌 🗌 🗌 🗌 🗌 🗌	Battery charger equipment and instrumentation	
062AA1.07	Loss of Nuclear Svc Water / 4	2.9 3			
		2.3		Flow rates to the components and systems that are serviced by the SWS; interactions among the components	
077AK1.03	Generator Voltage and Electric Grid Disturbances / 6	3.3 3	3.4	Under-excitation	
WE04EK1.1	LOCA Outside Containment / 3	3.5 3	3.9 🔽 🗌 🗌 🗌 🗌 🔲 🔲 🔲 🗌 🗌	Components, capacity, and function of emergency systems.	
WE05EK3.2	Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4	3.7 4	4.1	Normal, abnormal and emergency operating procedures associated with (Loss of Secondary Heat Sink).	
WE11EK2.1	Loss of Emergency Coolant Recirc. / 4	3.6 3	3.9	Components and functions of control and safety systems, including instrumentation, signals, interlocks, failure modes and automatic and manual features.	
we12EG2.4.3	Steam Line Rupture - Excessive Heat Transfer / 4	3.7 3	3.9	Ability to identify post-accident instrumentation.	

ES-401, REV 9			T10	52 PWR EXAMINATION OUTLINE	FORM ES-401	
KA	NAME / SAFETY FUNCTION:	i	IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRC)		
028AK2.02	Pressurizer Level Malfunction / 2	2.6	2.7		Sensors and detectors	
032AK3.02	Loss of Source Range NI / 7	3.7	4.1		Guidance contained in EOP for loss of source-range nuclear instrumentation	
033AG2.4.30	Loss of Intermediate Range NI / 7	2.7	4.1		Knowledge of events related to system operations/status that must be reported to internal orginizations or outside agencies.	
036AK1.01	Fuel Handling Accident / 8	3.5	4.1		Radiation exposure hazards	
068AA1.06	Control Room Evac. / 8	4.1	4.2		Charging pump	
074EA1.23	Inad. Core Cooling / 4	3.9	4		PORV block valve indicators, switches, controls (for both RCS and S/G).	
076AK3.06	High Reactor Coolant Activity / 9	3.2	3.8		Actions contained in EOP for high reactor coolant activity	
WE14EA2.1	Loss of CTMT Integrity / 5	3.3	3.8		Facility conditions and selection of appropriate procedures during abnormal and emergency operations.	
WE15EK1.2	Containment Flooding / 5	2.7	2.9		Normal, abnormal and emergency operating procedures associated with (Containment Flooding).	

ES-401, REV 9			T2G	1 PWR EXAMINATION OUTLINE	FORM ES-40	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRO			
003A4.03	Reactor Coolant Pump	2.8	2.5		RCP lube oil and lift pump motor controls	
004K4.08	Chemical and Volume Control	2.8	3.2		Hydrogen control in RCS	
004K5.29	Chemical and Volume Control	2.6	3.3		Reason for sampling for chloride, fluoride, sodium and solids in RCS	
005G2.4.46	Residual Heat Removal	4.2	4.2		Ability to verify that the alarms are consistent with the plant conditions.	
006K2.04	Emergency Core Cooling	3.6	3.8		ESFAS-operated valves	
007K1.03	Pressurizer Relief/Quench Tank	3.0	3.2		RCS	
008A2.08	Component Cooling Water	2.5	2.7		Effects of shutting (automatically or otherwise) the isolation valves of the letdown cooler	
D08K1.04	Component Cooling Water	3.3	3.3		RCS, in order to determine source(s) of RCS leakage into the CCWS	
010K4.02	Pressurizer Pressure Control	3.0	3.4		Prevention of uncovering PZR heaters	
)12A1.01	Reactor Protection	2.9	3.4		Trip setpoint adjustment	
)13K5.02	Engineered Safety Features Actuation	2.9	3.3		Safety system logic and reliability	

ES-401, REV 9			T20	31 PWR EXAMINATION OUTLINE	FORM ES-401	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRC)		
013K6.01	Engineered Safety Features Actuation	2.7	3.1		Sensors and detectors	
022K3.01	Containment Cooling	2.9	3.2		Containment equipment subject to damage by high or low temperature, humidity and pressure	
025K5 02	Ice Condenser	2.6	2.8		Heat transfer	
026A3.01	Containment Spray	4.3	4.5		Pump starts and correct MOV positioning	
026A4.01	Containment Spray	4.5	4.3		CSS controls	
039A3.02	Main and Reheat Steam	3.1	3.5		Isolation of the MRSS	
059K3.02	Main Feedwater	3.6	3.7		AFW system	
059K3.03	Main Feedwater	3.5	3.7.		S/GS	
061K6.02	Auxiliary/Emergency Feedwater	2.6	2.7		Pumps	
062G2.4.35	AC Electrical Distribution	3.8	4.0		Knowledge of local auxiliary operator tasks during emergency and the resultant operational effects	
063A4.01	DC Electrical Distribution	2.8	3.1		Major breakers and control power fuses	

ES-401, REV 9			T20	31 PWR EXAMINATION OUTLINE	FORM ES-401-2
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRC)	
064A2.05	Emergency Diesel Generator	3.1	3.2		Loading the ED/G
073A1.01	Process Radiation Monitoring	3.2	3.5		Radiation levels
073K1.01	Process Radiation Monitoring	3.6	3.9		Those systems served by PRMs
076K2.04	Service Water	2.5	2.6		Reactor building closed cooling water
078K4.02	Instrument Air	3.2	3.5		Cross-over to other air systems
103A2.04	Containment	3.5	3.6		Containment evacuation (including recognition of the alarm)

ES-401, REV 9		T2G	2 PWR EXAMINATION OUTLINE	FORM ES-4	
KA	NAME / SAFETY FUNCTION:	IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO SRO			
001K2.03	Control Rod Drive	2.7 3.1		One-line diagram of power supplies to logic circuits	
002A4.02	Reactor Coolant	4.3 4.5		Indications necessary to verify natural circulation from appropriate level, flow and temperature indications and valve positions upon loss of forced circulation	
017K3.01	In-core Temperature Monitor	3.5 3.7		Natural circulation indications	
027K5.01	Containment lodine Removal	3.1 3.4		Purpose of charcoal filters	
028A1.02	Hydrogen Recombiner and Purge Control	3.4 3.7		Containment pressure	
034K4.03	Fuel Handling Equipment	2.6 3.3		Overload protection	
035G2.2.40	Steam Generator	3.4 4.7		Ability to apply technical specifications for a system.	
055K1.06	Condenser Air Removal	2.6 2.6		PRM system	
071A3.02	Waste Gas Disposal	2.8 2.8		Pressure-regulating system for waste gas vent header	
75A2.03	Circulating Water	2.5 2.7		Safety features and relationship between condenser vacuum, turbine trip and steam dump	

ES-401, REV 9			T	3 PWR EXAMINATION OUTLINE	FORM ES-401
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRC)	
G2.1.38	Conduct of operations	3.7	3.8		Knowledge of the stations requirements for verbal communication when implamenting procedures
G2.1.7	Conduct of operations	4.4	4.7		Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior and instrument interpretation.
G2.2.11	Equipment Control	2.3	3.3		Knowledge of the process for controlling temporary design changes.
G2.2.19	Equipment Control	2.3	3.4		Knowledge of maintenance work order requirements.
G2.3.13	Radiation Control	3.4	3.8		Knowledge of radiological safety procedures pertaining to licensed operator duties
G2.3.15	Radiation Control	2.9	3.1		Knowledge of radiation monitoring systems
32.3.4	Radiation Control	3.2	3.7		Knowledge of radiation exposure limits under normal and emergency conditions
32.4.28	Emergency Procedures/Plans	3.2	4.1		Knowledge of procedures relating to emergency response to sabotage.
52.4.31	Emergency Procedures/Plans	4.2	4.1		Knowledge of annunciators alarms, indications or response procedures
62.4.6	Emergency Procedures/Plans	3.7	4.7		Knowledge symptom based EOP mitigation strategies.

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SRO T1G1 PWR EXAMINATION OUTLINE

FORM ES-401-2

KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
(RO	SRO			
029EA2.06	ATWS / 1	3.8	3.9		Main turbine trip switch position indication	
038EG2.2.40	Steam Gen. Tube Rupture / 3	3.4	4.7		Ability to apply technical specifications for a system.	
054AG2.4.30	Loss of Main Feedwater / 4	2.7	4.1		Knowledge of events related to system operations/status that must be reported to internal orginizations or outside agencies.	
055EA2.03	Station Blackout / 6	3.9	4.7		Actions necessary to restore power	
058AG2.4.18	Loss of DC Power / 6	3.3	4.0		Knowledge of the specific bases for EOPs.	
077AA2.08	Generator Voltage and Electric Grid Disturbances / 6	4.3	4.4		Criteria to trip the turbine or reactor	

ES-401, REV 9			RO T	1G2 PWR EXAMINATION OUTLINE	FORM ES-401-	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRO			
001AA2.02	Continuous Rod Withdrawal / 1	4.2	4.2		Position of emergency boration valve	
060AG2.4.1	Accidental Gaseous Radwaste Rel. / 9	4.6	4.8		Knowledge of EOP entry conditions and immediate action steps.	
061AA2.01	ARM System Alarms / 7	3.5	3.7		ARM panel displays	
069AG2.4.21	Loss of CTMT Integrity / 5	4.0	4.6		Knowledge of the parameters and logic used to assess the status of safety functions	

ES-401, REV 9		SRO T2G1 PWR EXAMINATION OUTLINE			FORM ES-401-2	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRC)		
012A2.01	Reactor Protection	3.1	3.6		Faulty bistable operation	
039A2.03	Main and Reheat Steam	3.4	3.7		Indications and alarms for main steam and area radiation monitors (during SGTR)	
076G2.4.3	Service Water	3.7	3.9		Ability to identify post-accident instrumentation.	
078A2.01	Instrument Air	2.4	2.9		Air dryer and filter malfunctions	
103G2.4.3	Containment	3.7	3.9		Ability to identify post-accident instrumentation.	

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ES-401, REV 9		SRO T2G2 PWR EXAMINATION OUTLINE			FORM ES-401-2	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRC)		
028A2.01	Hydrogen Recombiner and Purge Control	3.4	3.6		Hydrogen recombiner power setting, determined by using plant data book	
068A2.04	Liquid Radwaste	3.3	3.3		Failure of automatic isolation	
071G2.2.36	Waste Gas Disposal	3.1	4.2		Ability to analyze the effect of maintenance activities, such as degraded power sources, on the status of limiting conditions of operations	

ES	-401,	REV	9
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G2.4.46

Emergency Procedures/Plans

4.2 4.2

SRO T3 PWR EXAMINATION OUTLINE FORM ES-401-2 KA NAME / SAFETY FUNCTION: K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G IR TOPIC: RO SRO G2.1.27 Conduct of operations 3.9 4 Knowledge of system purpose and or function. G2.2.18 Equipment Control 2.6 3.8 Knowledge of the process for managing maintenance activities during shutdown operations. G2.2.40 Equipment Control 3.4 4.7 Ability to apply technical specifications for a system. G2.3.12 Radiation Control 3.2 3.7 Knowledge of radiological safety principles pertaining to licensed operator duties G2.3.14 Radiation Control 3.4 3.8 Knowledge of radiation or contamination hazards that may arise during normal, abnormal, or emergency conditions or activities G2.4.30 Emergency Procedures/Plans 2.7 4.1 Knowledge of events related to system operations/status that must be reported to internal orginizations or outside agencies.

Ability to verify that the alarms are consistent with the

plant conditions.

ES-301

Administrative Topics Outline

Form ES-301-1

Facility:Sequoyah Nuclea Examination Level: RO	r Station 1 & 2 SRO X	Date of Examination:09/07/2010 Operating Test Number: _ <u>2010302</u>	
Administrative Topic (see Note)	Type Code*	Describe activity to be performed	
Conduct of Operations	M, R, P	2.1.5 Ability to use procedures related to shift staffing, such as minimum crew compliment, overtime limitations, etc. Evaluate Overtime Restrictions	
Conduct of Operations	N, R	2.1.25 Ability to interpret reference materials, such as graphs, curves, tables, etc. Review of Estimated Critical Position Calculation	
Equipment Control	M, R	2.2.43 Knowledge of the process used to track inoperable alarms. Review and Approve a Disabled Alarm Checklist	
Radiation Control	D, R	2.3.4 Knowledge of radiation exposure limits under normal and emergency conditions. Evaluate Worker Exposure	
Emergency Procedures/Plan	D, R	 2.4.38 Ability to take actions called for in the facility emergency plan, including supporting or acting as emergency coordinator if required. Classify the Event per the REP (SGTR with Failed S/G Safety) 	
NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when all 5 are required.			
 * Type Codes & Criteria: (C)ontrol room, (S)imulator, or Class(R)oom (D)irect from bank (≤ 3 for ROs; ≤ 4 for SROs & RO retakes) (N)ew or (M)odified from bank (≥ 1) (P)revious 2 exams (≤ 1; randomly selected) 			

Admin 1.a – This JPM has the candidate review several different work schedules and determine if any of the required work-hour rules were violated. This is a Bank JPM that was on the 2009 NRC exam but has been Modified.

Admin 1.b – This JPM has the candidate review a completed ECP calculation and identify any and all errors that were committed during the development of the ECP. This is a New JPM.

Admin 2 – This JPM has the candidate review a Disabled Annunciator request and identify any and all errors that may have committed during the development of the request. This is a Modified Bank JPM.

Admin 3 – This JPM has the candidate review projected worker's dose for performing a job in the RCA and determine if any Federal or Administrative dose limits may be exceeded by any of the workers and which notifications would need to be made if the actual doses were to reach the estimated values. This is a Bank JPM.

Admin 4 – This JPM has the candidate review the data presented during a proposed accident and determine the Emergency Event Classification and report any Protective Action Recommendations (PARs) which would appropriate for the accident. This is a Bank JPM.

ES-301

Form ES-301-1

Facility:Sequoyah Nuclear S Examination Level: RO ${f X}$	Station 1 & 2_	Date of Examination:09/07/2010 Operating Test Number:2010302		
Administrative Topic (see Note)	Type Code*	Describe activity to be performed		
Conduct of Operations	M, R, P	2.1.5 Ability to use procedures related to shift staffing, such as minimum crew compliment, overtime limitations, etc		
		A.1.a Evaluate Overtime Restrictions		
Conduct of Operations	N, R [.]	2.1.7 Ability to interpret reference materials, such as graphs, curves, tables, etc.		
		A.1.b Perform RCS Deboration Calculation		
	D, R	2.2.12 Knowledge of surveillance procedures		
Equipment Control		Boric Acid Storage Tank Level Operability Determination		
Radiation Control	M, R	2.3.4 Knowledge of radiation exposure limits under normal and emergency conditions.		
		Determine Maximum Stay Time		
Emergency Procedures/Plan				
NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when all 5 are required.				
* Type Codes & Criteria: (C)ontrol room, (S)imulator, or Class(R)oom (D)irect from bank (≤ 3 for ROs; ≤ 4 for SROs & RO retakes) (N)ew or (M)odified from bank (≥ 1) (P)revious 2 exams (≤ 1; randomly selected)				

Admin 1.a – This JPM requires the candidate to evaluate different schedules and determine if any of the required work-hour rules were violated. This is a Bank JPM that was used on the 2009 NRC exam which has been Modified.

Admin 1.b – This JPM requires the candidate to calculate the amount time that a de-borating demineralizer needs to be placed in service to reduce the desired RCS boron concentration during normal EOL plant operation. This is a New JPM.

Admin 2 – This JPM requires the candidate to evaluate the conditions of a Boric Acid Storage Tank to determine if it meets the Tech Spec requirements. This is a Bank JPM.

Admin 3 – This JPM requires the candidate to determine the expected dose they would receive while conducting tagging operations in the RCA. This is a Modified Bank JPM from a previous Watts Bar exam.

ES-301

Control Room/In-Plant Systems Outline

Form ES-301-2

1		Date of Examination: perating Test No.:	
Contro	bl Room Systems $^{@}$ (8 for RO); (7 for SRO-I); (2 or 3 for SRO	-U, including 1 ESF)	
	System / JPM Title	Type Code*	Safety Function
a.	001 Control Rod Drive System; A2.17 (3.3/3.8) 038-AP3 – Shutdown Bank Withdrawal	M, A, S, L	1
b.	006 Emergency Core Cooling System; A1.13 (3.5/3.7) 097-2AP – Refill #3 CLA to within Normal Range	D, A, S	2
C.	007 Pressurizer Relief Tank System A2.02 (2.6/3.2) 036 – Return PRT to Normal	D, S	5
d.	003 Reactor Coolant Pump System; A1.02 (2.9/2.9) Sim. D – Respond to Loss of Flow to RCP Oil Cooler	N, S	4P
e.	064 Emergency Diesel Generation; A4.06 (3.9/3.9) 046-1 – Shutdown the Diesel Generators Following Auto Start	D, S	6
f.	015 Nuclear Instrumentation System; A4.02 (3.9/3.9) 022-AP2 – Calibrate Power Range Nis	D, A, S	7
g.	036 Fuel Handling Incidents AA2.02 (3.4/4.1) 104-AP – Initiate Makeup to the Refueling Cavity	M, A, S, L	8
h.	W/E14 High Containment Pressure; EA1.1 (3.7/3.7) 057-AP1 – Respond to HI CNMT Pressure, Place RHR Spray In Service	D, A, S	3
In-Plar	nt Systems [@] (3 for RO); (3 for SRO-I); (3 or 2 for SRO-U)		
i.	063 D.C. Electrical Distribution; A4.01 (2.7/3.0) 071 – Spare out a Vital Batt Charger	D	6
j.	061 Auxiliary/Emergency Feedwater System; A2.07 (3.4/3. 024AP – Local control of MD AFW Pump Flow	.5) D, A, R	48
k.	002 Reactor Coolant System; A3.01 (3.7/3.8) 072-2 – Local Alignment of 2-RM-90-112 to Lower CNMT	D, R	8

All RO and SRO-I control room (and in-plant) systems must be different and serve different safety functions; all 5 SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room.			
* Type Codes	Criteria for RO / SRO-I / SRO-U		
 (A)Iternate path (C)ontrol room (D)irect from bank (E)mergency or abnormal in-plant (EN)gineered safety feature (L)ow-Power / Shutdown (N)ew or (M)odified from bank including 1(A) (P)revious 2 exams (R)CA (S)imulator 	$4-6 / 4-6 / 2-3$ $\leq 9 / \leq 8 / \leq 4$ $\geq 1 / \geq 1 / \geq 1$ $- / - / \geq 1 \text{ (control room system)}$ $\geq 1 / \geq 1 / \geq 1$ $\geq 2 / \geq 2 / \geq 1$ $\leq 3 / \leq 3 / \leq 2 \text{ (randomly selected)}$ $\geq 1 / \geq 1 / \geq 1$		

Sim A – This JPM has the candidate identify misaligned control rods while attempting to withdraw the Shutdown banks in preparation for a reactor start-up. This is a Modified Bank, Low Power, Alternate Path JPM.

Sim B – This JPM has the candidate refill a cold leg accumulator to its normal level, and then has the fill valve stick open requiring alternate methods to stop the fill. This is an Alternate Path, Bank JPM

Sim C – This JPM has the candidate return the PRT to normal temperature and pressure following a leaking PZR PORV. This is Bank JPM.

Sim D – This JPM has the candidate determine that a loss of cooling has occurred to the RCP oil coolers which will require them to stop the affected RCP. This is a New JPM.

SIM E – This JPM has the candidate shutdown an EDG that has been running unloaded for an extended period of time following an auto start. This will require the diesel be paralleled and loaded prior to shutdown. This is a bank JPM.

SIM F – This JPM has the candidate review the NIS readings against a calorimetric power calculation. The Alternate path is that a calibration adjustment is required to be made to a power range NI. This is a Alternate Path, Bank JPM.

SIM G – This JPM has the candidate respond to lowering level in the Refueling Cavity. The Alternate path is that the normal supply of fill from the charging pumps will be unavailable and will require re-alignment of the suction of RHR pumps to provide the fill. This is a Modified Bank, Low Power, Alternate path JPM.

SIM H – This JPM directs that candidate to respond to continued High CNMT pressure following a LOCA by aligning RHR to provide CNMT spray. The Alternate path develops when the normal spray supply valve will not open. This is a Alternate Path, Bank JPM.

In-plant I – This JPM has the candidate walk through process of placing the spare battery charger in service and removing the normal charger. This is Bank JPM.

In-plant J – This JPM has the candidate take local control of AFW supply to #3 SG due to valve not responding from the control room. This is an Alternate Path, Bank JPM performed in the RCA.

In-plant K – This JPM directs the candidate to locally realign 2-RM-90-112 to monitor Lower Containment as part of the Unit 2 RCS leak detection system. This is a Bank JPM performed in the RCA.