

Admitted: 01/25/2011 Rejected:

Withdrawn: Stricken:

			NRC000061		
NRC FORM 374	U.S. NUCLEAR REGULA	TORY COMMISSION	Page 1 of 7		
	MATERIALS LICENSE				
Pursuant to the Atomic Energy Act of 1 Title 10, Code of Federal Regulations statements and representations hereter receive, acquire, possess, and transfer to for the purpose(s) and at the place(s) d it in accordance with the regulations of the in Section 183 of the Atomic Energy Act of the Nuclear Regulatory Commission	954, as amended, the s, Chapter I, Parts 30, ofore made by the lice byproduct, source, and esignated below; to del he applicable Part(s). T to of 1954, as amended now or hereafter in eff	Energy Reorganization 31, 32, 33, 34, 35, 36 nsee, a license is here special nuclear material iver or transfer such ma his license shall be deen , and is subject to all ap ect and to any condition	Act of 1974 (Public Law 93-438), and c, 39, 40, and 70, and in reliance on by issued authorizing the licensee to designated below; to use such material terial to persons authorized to receive med to contain the conditions specified plicable rules, regulations, and orders as specified below.		
Licensee					
1. Louisiana Energy Services, L.P. 3. License Number: SNM-2010, Amendment 6					
2. P.O. Box 1789		4. Expiration Date	e: See Condition 13		
Eunice, New Mexico 88231		5. Docket No. 70	-3103		
6. Source and/or Special Nuclear Material	7. Chemical and/o Form	or Physical 8.	Maximum amount that Licensee May Possess at Any One Time		
	Under This Lic	ense			
A. Uranium (natural and depleted) and daughter products	A.1 Physical: and Gas	Solid, Liquid,	A. 136,120,000 kg		
SIS	A.2 Chemical UO ₂ F ₂ , ox other com	UF ₆ , UF ₄ , ides and pounds	the MA		
B. Uranium enriched in isotope U-235 up to	B.1 Physical: and Gas	Solid, Liquid,	B. 545,000 kg		
5 percent by weight and uranium daughters	B.2 Chemical UO ₂ F ₂ , ox	UF ₆ , UF₄,	5		
1	and other	compounds -	0		
C. Tc-99, transuranic isotopes and other	C. Any 👊	000	C. Amount that exists as contamination as a		
contamination	**	×××	consequence of the historical feed of recycled uranium at other facilities		
D. CI-36	D. Unsealed	any form	D. 2.26E-1 uCi		
E. Cr-51	E. Sealed pe	r §30.32(g)(1)	E. 1.00E+1 uCi		
F. Co-57	F. Sealed pe	r §30.32(g)(1)	F. 1.00E+4 uCi		
G. Co-60	G. Sealed pe	r §30.32(g)(1)	G. 1.00E+1 uCi		
H. Ni-63	H. Unsealed	any form	H. 1.00E+1 uCi		
I. Sr-85	I. Sealed pe	r §30.32(g)(1)	I. 1.00E+1 uCi		
J. Y-88	J. Sealed pe	r §30.32(g)(1)	J. 1.00E+1 uCi		

RC FORM 374A	U.S. NUCLEAR REGUL	ATORY COMMISSION	License Number	Page 2 of
MATERIALS LICENSE SUPPLEMENTARY SHEET			Docket or Referen	ce Number
K. Sr-90	К.	Unsealed, any forr	n K.	5.00E+0 uCi
L. Y-90	L.	Unsealed, any forr	n L.	5.00E+0 uCi
M. Tc-99	М.	Unsealed, any forr	л G , М.	1.00E+1 uCi
N. Cd-109	N.	Sealed per §30.32	(g)(1) N.	1.00E+3 uCi
O. Sn-113	О.	Sealed per §30.32	(g)(1) O.	1.00E+1 uCi
P. Te-123m	🤊 Р.	Sealed per §30.32	(g)(1) P.	1.00E+1 uCi
Q. Cs-137	Q.	Sealed per §30.32	(g)(1) Q.	5.00E+4 uCi
R. Eu-152 (13	y) R.	Sealed per §30.32	(g)(1) R.	2.00E+0 uCi
S. Po-210	S. (Unsealed, any form	n S.	1.00E+1 uCi
T. Th-230	Т. 7	Unsealed, any forr	n Т.	1.00E+0 uCi
U. U-232	U.	Unsealed, any form	n	1.00E+0 uCi
V. U-233	V.V.	Sealed per §30.32	(g)(1) V.	1.00E+5 uCi
W. U-234	W.	Unsealed, any forr	n W.	1.00E+0 uCi
X. U-235	X.	Unsealed, any form	n X.	1.00E+0 uCi
Y. U-236	7 Y.	Sealed per §30.32	(g)(1) Y.	1.00E+5 uCi
Z. U-238	7 Z.	Unsealed, any forr	n 🔽.	1.00E+0 uCi
AA. Am-241	AA.	Sealed per §30.32	(g)(1) 🗡 🗛	5.00E+4 uCi
BB. Cf-252	BB.	Sealed per §30.32	(g)(1) BB	. 5.00E+4 uCi

- 9. Authorized place of use: National Enrichment Facility (NEF), located 5 miles east of Eunice, New Mexico on Highway 176 in Lea County, New Mexico.
- 10. The licensee shall conduct authorized activities at the NEF in accordance with the statements, representations, and conditions, or as revised in accordance with Section 19 of the Quality Assurance Program Description, 10 CFR 40.35(f), 10 CFR 51.22, 10 CFR 70.32, 10 CFR 70.72, or 10 CFR 95.19 in:
 - a. Application for Material License, NRC Form 313 dated December 12, 2003.
 - b. Safety Analysis Report dated December 12, 2003, as revised by letters dated February 27, 2004; July 30, 2004; September 30, 2004; April 22, 2005; April 29, 2005; May 25, 2005; June 10, 2005; February

NRC FORM 374A U.S. NUCLEAR REGULATORY COMMISSION Page 3 of 7 License Number SNM-2010, Amendment 6 MATERIALS LICENSE Docket or Reference Number SUPPLEMENTARY SHEET 70-3103 16, 2006; February 28, 2006; March 16, 2006; March 24, 2006; January 29, 2007; April 10, 2007; July 30, 2007, October 12, 2007; November 12, 2007; and November 30, 2007. c. Environmental Report dated December 12, 2003, as revised by letters dated February 27, 2004; July 30, 2004; September 30, 2004; April 22, 2005; June 10, 2005; March 16, 2006; March 24, 2006; January 29, 2007; April 10, 2007; July 30, 2007, and November 30, 2007. d. Physical Security Plan dated December 12, 2003, as revised by letters dated May 12, 2004; July 30, 2004; December 10, 2004; and January 12, 2005. e. Fundamental Nuclear Material Control Plan dated December 12, 2003, as revised by letters dated February 27, 2004; July 30, 2004; October 7, 2004; October 15, 2004; December 7, 2004; April 22, 2005; October 23, 2006; and November 30, 2007. f. Quality Assurance Program Description dated April 9, 2004, as revised by letter dated April 22, 2005; October 23, 2006; November 12, 2007; July 30, 2007, October 12, 2007, and November 12, 2007. Emergency Plan dated December 12, 2003, as revised by letters dated July 30, 2004; September 30, g. 2004; April 22, 2005; October 23, 2006; and July 30, 2007. Standard Practice Procedure Plan for the Protection of Classified Matter dated December 12, 2003, as h. revised by letters dated July 30, 2004; March 16, 2006; November 21, 2006; November 22, 2006; March 20, 2007; April 27, 2007; July 19, 2007; and November 30, 2007. Standard Practice Procedure Plan for the Protection of Classified Matter at the ETUS Location of the i. National Enrichment Facility dated October 11, 2007, as revised by letter dated December 3, 2007. 11. Introduction of UF₆ into any module of the NEF shall not occur until the Commission completes an operational readiness and management measures verification review to verify that management measures that ensure compliance with the performance requirements of 10 CFR 70.61 have been implemented and confirms that the facility has been constructed and will be operated safely and in accordance with the requirements of the license. The licensee shall provide the Commission with 120 days advance notice of its plan to introduce UF₆ in any module of the NEF. 12. The licensee is hereby granted the special authorizations and exemptions identified in Section 1.2.3.6 of the National Enrichment Facility Safety Evaluation Report, dated June 2005. 13. This license will expire 30 years after the date of license issuance. 14. For the disposition of depleted UF₆, the licensee shall not use a depleted UF₆ deconversion facility that employs a process that results in the production of anhydrous hydrofluoric acid. 15. a. The licensee shall provide proof of \$5 million liability insurance, as required under 10 CFR 140.13b, at least 30 days prior to the planned date for obtaining possession of test material (less than or equal to 50 kg) of depleted or natural uranium hexafluoride. b. The licensee shall provide proof of full liability insurance, as required under 10 CFR 140.13b, at least 30 days prior to the planned date for obtaining feed material (greater than 50 kg uranium hexafluoride). If the licensee is proposing to provide less than \$300 million of liability insurance coverage, the licensee shall provide, to the NRC for review and approval, an evaluation supporting liability insurance coverage in amounts less than \$300 million, at least 120 days prior to the planned date for obtaining feed material.

NRC FORM 374A	U.S. NUCLEAR REGULATORY COMMISSION		Page 4 of 7
		License Number SNM-2010, Amendment 6	
	MATERIALS LICENSE SUPPLEMENTARY SHEET	Docket or Reference Number 70-3103	

- 16. a. The licensee shall provide an updated Decommissioning Funding Plan cost estimate update and final copies of the proposed financial assurance instruments to NRC for review at least six months prior to the planned date for obtaining test material (less than or equal to 50 kilograms of uranium hexafluoride), and provide to NRC final executed copies of the reviewed financial assurance instruments at least 21 days prior to the receipt of test material. In this Decommissioning Funding Plan update, the licensee shall provide full funding for decontamination and decommissioning of the Centrifuge Test Facility, the Post Mortem Facility, the Cylinder Receipt and Dispatch Building, and all other plant areas where licensed material is used. The amount of the financial assurance instrument shall be updated to current year dollars and include any applicable changes to the decommissioning cost estimate.
 - b. The licensee shall provide an updated Decommissioning Funding Plan cost estimate update and final copies of the proposed financial assurance instruments to NRC for review at least six months prior to the planned date for obtaining feed material (greater than 50 kilograms of uranium hexafluoride) for initial production in Separations Building Module (SBM) 1001, and provide to NRC final executed copies of the reviewed financial assurance instruments at least 21 days prior to the receipt of feed material. In this Decommissioning Funding Plan update, the licensee shall provide full funding for decontamination and decommissioning of SBM 1001 and all other plant areas where licensed material is used.

In addition, the licensee shall provide funding for the disposition of depleted uranium tails in an amount needed to disposition the first three years of depleted uranium tails generation. The decommissioning cost estimate shall include an update to the U.S. Department of Energy (DOE) depleted uranium disposition cost estimate. The total amount funded for depleted uranium disposition shall be no less than the updated DOE cost estimate. The amount of the financial assurance instrument shall be updated to current year dollars and include any applicable changes to the decommissioning cost estimate.

- c. The licensee shall provide an updated Decommissioning Funding Plan cost estimate update and final copies of the proposed financial assurance instruments to NRC for review at least six months prior to introducing feed material in SBM 1003, and provide to NRC final executed copies of the reviewed financial assurance instruments at least 21 days prior to introducing feed material into SBM 1003. In this Decommissioning Funding Plan update, the licensee shall provide full funding for decontamination and decommissioning of SBM 1003 and all other plant areas where licensed material is used. The amount of the financial assurance instrument shall be updated to current year dollars and include any applicable changes to the decommissioning cost estimate.
- d. The licensee shall provide an updated Decommissioning Funding Plan cost estimate update and final copies of the proposed financial assurance instruments to NRC for review at least six months prior to introducing feed material in SBM 1005, and provide to NRC final executed copies of the reviewed financial assurance instruments at least 21 days prior to introducing feed material into SBM 1005. In this Decommissioning Funding Plan update, the licensee shall provide full funding for decontamination and decommissioning of SBM 1005 and all other plant areas where licensed material is used. The amount of the financial assurance instrument shall be updated to current year dollars and include any applicable changes to the decommissioning cost estimate.

		License Number SNM-2010, Amendment 6
	MATERIALS LICENSE SUPPLEMENTARY SHEET	Docket or Reference Number 70-3103
e.	Subsequent updated decommissioning funding estimat decommissioning shall be provided to NRC for review, SBM 1003 or SBM 1005 is delayed, the current decom updated and provided to NRC for review, at a minimum	es and revised funding instruments for facility at a minimum, every three years. If operation on missioning funding cost estimate shall be , every three years.
f.	After the first three years of initial plant production, sub- estimates and revised funding instruments for depleted on a forward-looking basis to reflect projections of depl depleted uranium disposition cost estimate shall include disposition cost estimate. The total amount funded for than the updated DOE cost estimate.	sequent updated decommissioning cost uranium disposition shall be provided annually eted uranium byproduct generation. The e an update to the DOE depleted uranium depleted uranium disposition shall be no less
17. De	eleted	P
18 De	eleted U	
"IF ins	ROFS Boundary Definitions." Completed IROFS boundar	ing for all IDOEC shall be available for
20. Cu pro co de Co to co an An	spection at the time of the operational readiness review. urrently, there are no IROFS that have been specified as ogrammable logic controllers, and/or any digital device, ir mmunication protocols (such as fieldbus devices and Loo sign of any IROFS be changed to include any of the prec ommission approval prior to implementing the change(s). accepted best practices in software and hardware engine ntrols as discussed in the Quality Assurance Program De d the applicable guidance of the following industry standa halysis Report Chapter 3:	using software, firmware, microcode, icluding hardware devices which implement da cal Area Network controllers), etc. Should the eding features, the licensee shall obtain The licensee's design change(s) shall adhere earing, including software quality assurance escription throughout the development process ards and regulatory guides as specified in Safe
20. Cu pro co de Co to co an An a.	spection at the time of the operational readiness review. urrently, there are no IROFS that have been specified as ogrammable logic controllers, and/or any digital device, ir mmunication protocols (such as fieldbus devices and Loc sign of any IROFS be changed to include any of the preco- ommission approval prior to implementing the change(s). accepted best practices in software and hardware engine ntrols as discussed in the Quality Assurance Program De d the applicable guidance of the following industry standar halysis Report Chapter 3: American Society of Mechanical Engineers (ASME) NG Assurance Requirements of Computer Software for Nu 1a-1995 Addenda of NQA-1-1994 and ASME NQA-1-1 Requirements for Computer Program Testing." (Refer	using software, firmware, microcode, icluding hardware devices which implement da cal Area Network controllers), etc. Should the eding features, the licensee shall obtain The licensee's design change(s) shall adhere earing, including software quality assurance escription throughout the development process ards and regulatory guides as specified in Safe QA-1-1994, Part II, subpart Part 2.7, "Quality clear Facility Applications," as revised by NQA- 994, Part 1, Supplement 11S-2, "Supplementa to SAR Chapter 11, Appendix A, Section 3.)
20. Cu pro co de Co to co an An a.	spection at the time of the operational readiness review. urrently, there are no IROFS that have been specified as ogrammable logic controllers, and/or any digital device, ir mmunication protocols (such as fieldbus devices and Loc sign of any IROFS be changed to include any of the preco- ommission approval prior to implementing the change(s). accepted best practices in software and hardware engine ntrols as discussed in the Quality Assurance Program De d the applicable guidance of the following industry standa- nalysis Report Chapter 3: American Society of Mechanical Engineers (ASME) NG Assurance Requirements of Computer Software for Nu 1a-1995 Addenda of NQA-1-1994 and ASME NQA-1-1 Requirements for Computer Program Testing." (Refer Electric Power Research Institute (EPRI) NP-5652, "Gu Items in Nuclear Safety Grade Applications," June 1988	A-1-1994, Part II, subpart Part 2.7, "Quality clear Facility Applications," as revised by NQA- 994, Part 1, Supplement 11S-2, "Supplementation to SAR Chapter 11, Appendix A, Section 3.) ideline for the Utilization of Commercial Grade 3.
20. Cu pro co de Co to co an An a. b.	spection at the time of the operational readiness review. Irrently, there are no IROFS that have been specified as ogrammable logic controllers, and/or any digital device, ir mmunication protocols (such as fieldbus devices and Loc sign of any IROFS be changed to include any of the preco ommission approval prior to implementing the change(s). accepted best practices in software and hardware engine ntrols as discussed in the Quality Assurance Program De d the applicable guidance of the following industry standar alysis Report Chapter 3: American Society of Mechanical Engineers (ASME) NG Assurance Requirements of Computer Software for Nu 1a-1995 Addenda of NQA-1-1994 and ASME NQA-1-1 Requirements for Computer Program Testing." (Refer Electric Power Research Institute (EPRI) NP-5652, "Gu Items in Nuclear Safety Grade Applications," June 1988 EPRI Topical Report (TR) -102323, "Guidelines for Elec Plants," Revision 1, December 1996.	using software, firmware, microcode, icluding hardware devices which implement da cal Area Network controllers), etc. Should the eding features, the licensee shall obtain The licensee's design change(s) shall adhere earing, including software quality assurance escription throughout the development process ards and regulatory guides as specified in Safe QA-1-1994, Part II, subpart Part 2.7, "Quality clear Facility Applications," as revised by NQA- 994, Part 1, Supplement 11S-2, "Supplementation to SAR Chapter 11, Appendix A, Section 3.) ideline for the Utilization of Commercial Grade 3.

e. Regulatory Guide 1.152, "Criteria for Digital Computers in Safety Systems in Nuclear Power Plants," Revision 1, January 1996.

NRC FORM 374A	U.S. NUCLEAR REGULATORY COMMISSION

Page 6 of 7

MATERIALS LICENSE SUPPLEMENTARY SHEET

License Number SNM-2010, Amendment 6

Docket or Reference Number 70-3103

- f. Regulatory Guide 1.168, "Verification, Validation, Reviews, and Audits for Digital Software Used in Safety Systems of Nuclear Power Plants," Revision 1, February 2004.
- g. Regulatory Guide 1.169, "Configuration Management Plans for Digital Computer Software Used in Safety Systems of Nuclear Power Plants," September 1997.
- h. Regulatory Guide 1.170, "Software Test Documentation for Digital Computer Software Used in Safety Systems of Nuclear Power Plants," September 1997.
- i. Regulatory Guide 1.172, "Software Requirements Specifications for Digital Computer Software Used in Safety Systems of Nuclear Power Plants," September 1997.
- j. Regulatory Guide 1.173, "Developing Software Life Cycle Processes for Digital Computer Software Used in Safety Systems of Nuclear Power Plants," September 1997.

If any above changes result in IROFS requiring operator actions, a human factors engineering review of the human-system interfaces shall be conducted using the applicable guidance in NUREG-0700, "Human-System Interface Design Review Guidelines," Revision 2, dated May 2002 (NRC, 2002d), and NUREG-0711, "Human Factors Engineering Program Review Model," Revision 2, dated February 2004.

- 21. Onsite storage of DUF₆ generated at the NEF shall be limited to a maximum of 5,016 48Y cylinders (or the equivalent amount of uranium stored in other NRC accepted and Department of Transportation ("DOT") certified cylinder types) of DUF₆. The generation of any additional DUF₆ to be stored onsite by the licensee beyond this limit shall constitute noncompliance with the license. The licensee shall suspend production of any additional DUF₆ for onsite storage until this noncompliance is remedied. In no event shall the licensee store DUF₆ generated at the NEF in New Mexico other than at the NEF.
- 22. Onsite storage of any one cylinder of DUF₆ generated at the NEF shall be limited to a maximum of 15 years, beginning from the date that each cylinder is filled in accordance with the licensee's standard procedures. The storage of any one DUF₆ cylinder beyond this limit by the licensee shall constitute noncompliance with the license. The licensee shall suspend production of any additional DUF₆ for onsite storage until this noncompliance is remedied. In no event shall the licensee store DUF₆ generated at the NEF in New Mexico other than at the NEF.
- 23. The licensee shall provide financial assurance for the offsite disposal of DUF₆ from the NEF using a minimum contingency factor of twenty-five percent (25%).

Upon reaching 4,000 cylinders of DUF_6 in 48Y cylinders (or the equivalent amount of uranium stored in other NRC accepted and DOT certified cylinder types) in onsite storage, the licensee shall immediately increase the financial assurance to provide a fifty percent (50%) contingency factor for disposition of DUF_6 stored at the NEF unless: (a) an application to construct and operate a deconversion facility outside of New Mexico that is specifically designated to deconvert the DUF_6 stored onsite at the NEF has been docketed by the agency responsible for reviewing the application; (b) an application for such a facility has been approved by the agency responsible for reviewing the application; or (c) the licensee is using another alternate method for removing the DUF_6 stored onsite.

			SNM-2010, Amendment 6
	MATERIALS LIC SUPPLEMENTARY	ENSE ' SHEET	Docket or Reference Number 70-3103
In add uraniu shall i dispos (50%) onsite and o DUF ₆ (b) an applic	dition, upon reaching the lim um stored in other NRC accommediately increase the fin sition of DUF_6 stored at NEF). The contingency factor st e is reduced to ninety-eight p perate a deconversion facili stored onsite at the NEF hat application for such a facili cation; or (c) the licensee is	it of 5,016 cylinders of DU epted and DOT certified cy ancial assurance to provid if the contingency factor hall remain at fifty percent bercent (98%) of the 5,016 ty outside of New Mexico the s been docketed by the ag ty has been approved by the using another alternate me	F_6 in 48Y cylinders (or the equivalent amount ylinder types) in onsite storage, the licensee le fifty percent (50%) contingency factor for has not already been increased to fifty percent (50%) until the number of cylinders stored limit and either: (a) an application to construct that is specifically designated to deconvert the gency responsible for reviewing the application he agency responsible for reviewing the ethod for removing the DUF ₆ from New Mexico
Nothii laws a	ng herein shall release the li and regulations.	censee from other financia	al assurance obligations set forth in applicable
24. The li accou pursu for the effect withou effect licens	censee shall maintain and founting and measurement courting and measurement courting ant to 10 CFR 74.33(b). The safeguarding of uranium siveness of the material contract prior approval of the Compares of its material contracted shall submit an application of the contracted shall submit an application.	ollow the Fundamental Nu ntrol of uranium source ma e licensee shall make no o ource material or special r rol and accounting program mission. If the licensee de ol and accounting program on for amendment to its license	clear Material Control Program for control and aterial and special nuclear material at the NEF change to material control procedures essentia nuclear material that would decrease the m implemented pursuant to 10 CFR 74.33(b) esires to make changes that would decrease the n or its measurement control program, the cense pursuant to 10 CFR 70.34.
The li withou furnis an ap	censee shall maintain record at prior Commission approve h to the Director, Division of propriate method listed in 10 ns of the change if it pertain	ds of changes to the mate al a period of five years fro Nuclear Security, Office N 0 CFR 70.5(a), a report co s to uranium enriched less	rial control and accounting program made on the date of the change. The licensee shall Juclear Security and Incident Response, using Intaining a description of each change within si than 20 percent in the uranium-235 isotope.
monu	a are any revisions to the p		
25. If ther letter licens	to NRC describing the chan ee may not implement the c	uclear criticality safety value ges and shall provide the changes in the revised value	dation report, then the licensee shall provide a revised validation report upon request. The dation report until NRC approves the changes.
 If ther letter licens The li exception 	to NRC describing the chan see may not implement the c censee shall not use, proce of provided by applicable per	uclear criticality safety valid ges and shall provide the changes in the revised valid ss, store, reproduce, trans rsonnel and facility clearar	dation report, then the licensee shall provide a revised validation report upon request. The dation report until NRC approves the changes mit, handle, or allow access to classified matter acces as required under 10 CFR Part 95.
25. If ther letter licens 26. The li excep	to NRC describing the chan see may not implement the c censee shall not use, proce of provided by applicable per	uclear criticality safety valid ges and shall provide the changes in the revised valid ss, store, reproduce, trans rsonnel and facility clearar	dation report, then the licensee shall provide a revised validation report upon request. The dation report until NRC approves the changes. mit, handle, or allow access to classified matter aces as required under 10 CFR Part 95.